



Understanding Confinement In Advanced Inductive (AI) Scenario Plasmas – Dependence On Gyroradius And Rotation



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SUMMARY OF OBSERVATIONS AND RESULTS

- Determined the scaling of global energy confinement and local heat transport with normalized plasma size ($\rho^* = \rho/\rho_0$) for AI (hybrid) scenario plasmas.
- Obtained good matches between JET and DIII-D plasmas for
 - global energy confinement scaling
 - identity plasmas (all dimensionless parameters the same)
 - at the extremes of the ρ^* range (all except ρ^* matched)
 - a range of ~ 2.7 ; about the same as from JET to ITER.
- Key results:
 - generally Bohm-like transport,
 - global scaling: $\tau_{E0} \propto \rho_0^{-1.9}$,
 - local transport:
 - matched profiles for $0.45 \leq \rho \leq 0.85$,
 - for $\chi_{i,e}/\chi_B \propto \rho_0^{\alpha_{i,e}}$, obtain $\alpha_i = -0.57$ and $\alpha_e = -0.42$, at $\rho = 0.65$, ($\chi_B = T/eB$)
- Projection to ITER performance will require scaling with v_{*e} , and better understanding of rotation and T_e/T_i dependences.
- Determined the dependence on rotation of global confinement for AI plasmas in DIII-D.
 - over the accessible rotation range (a factor of ~ 4.5) τ_{E0} increases by $\sim 60\%$ and the scaling multipliers H_{89} and H_{98y2} increase by $\sim 30\%$.
 - determined the behavior of the $m/n=3/2$ NTM island as rotation changes and the effect of the NTM on confinement:
 - at low rotation, there is a 10-15% penalty in τ_{E0} due to the NTM;
 - at high rotation, this decreases to 5-10%.

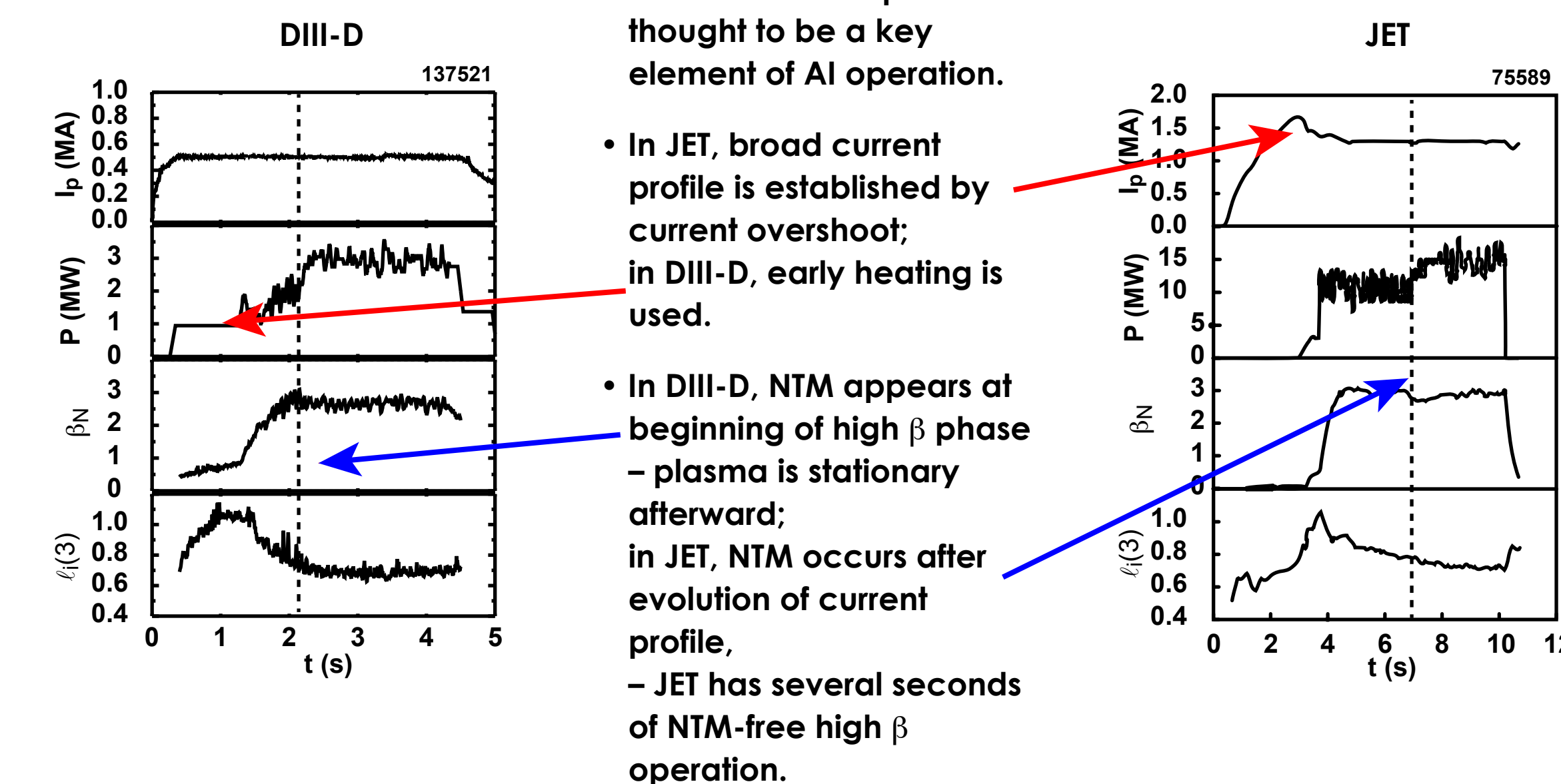
Physics-based extrapolation by nondimensional scaling

- Advanced inductive plasmas are a realization of the ITER hybrid scenario, providing high neutron fluence in a long duration inductive discharge.
- Extrapolation of the performance of AI plasmas depends on the scaling of transport with plasma size.
- Determining the scaling of performance with dimensionless parameters is a robust and rigorous method for development of scaling laws.
- A key parameter for extrapolation to ITER and reactors is $\rho^* = (\text{ion gyroradius})/(\text{minor radius})$.

Using both JET and DIII-D extends ρ^* scaling range

- The accessible range in ρ^* is large:
 - ~ 2.7 from high ρ^* in DIII-D to low ρ^* in JET,
 - about the same as the range from JET to ITER.
- There are three key elements to this study:
 - determining the scaling with ρ^* of the global energy confinement time normalized to the ion gyro-period, $(eB/m)\tau_{E0}$ (or $B\tau_{E0}$ for convenience),
 - demonstration of identity discharges in the two tokamaks, and
 - assessment of the scaling with ρ^* of the local heat diffusivity (normalized to the Bohm diffusivity), $\chi/\chi_B = \chi/(T/eB)$.

DIII-D (high ρ^*) and JET (low ρ^*) use different methods to prepare AI plasmas



Global scaling of energy confinement time is Bohm-like

- Assume that the thermal diffusivity has a power law dependence on ρ_0 : $\chi/\chi_B \propto \rho_0^{\alpha}$

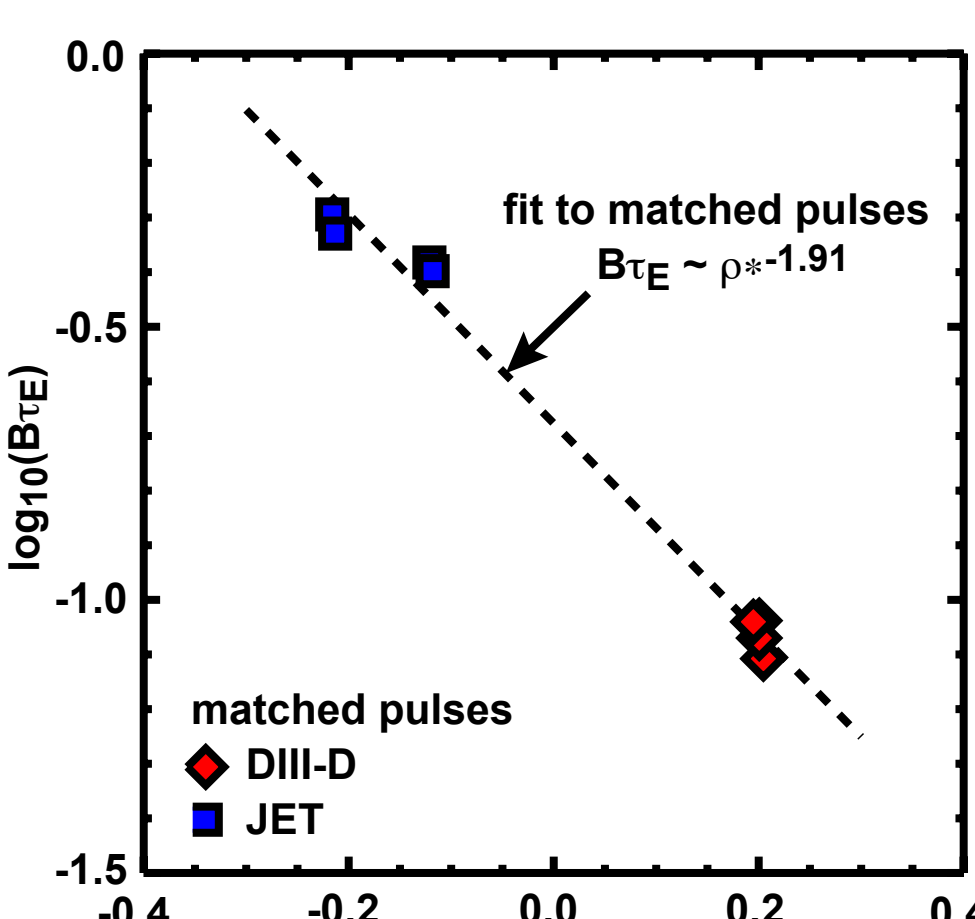
where $\chi_B = T/eB$ then the global energy confinement time will scale like $(eB/m)\tau_{E0} \propto \rho_0^{-(2+\alpha)} F(v_{*e}, \beta, q, T_i/T_e, \epsilon, \dots)$

- If all other parameters are fixed, the dependence of $B\tau_{E0}$ on ρ_0 can be determined.
- Eight matched time slices (4 JET & 4 DIII-D) were selected for this global comparison.
 - good matches of n_e , T_e , and T_i profiles,
 - also good matches of β , v_{*e} , and M .

parameter	JET mean	DIII-D mean	$\Delta\%$
a/R	0.315	0.298	2.7
κ_0	1.51	1.54	0.7
q_{CY1}	2.92	2.68	4.4
q_{CY1}	0.854	0.715	8.9
v_{*e}	0.594	0.563	2.7
β_{T1}	5.58	5.45	1.2
T_i/T_e	1.097	1.222	5.4
H_{98y2}	1.29	1.32	1.2

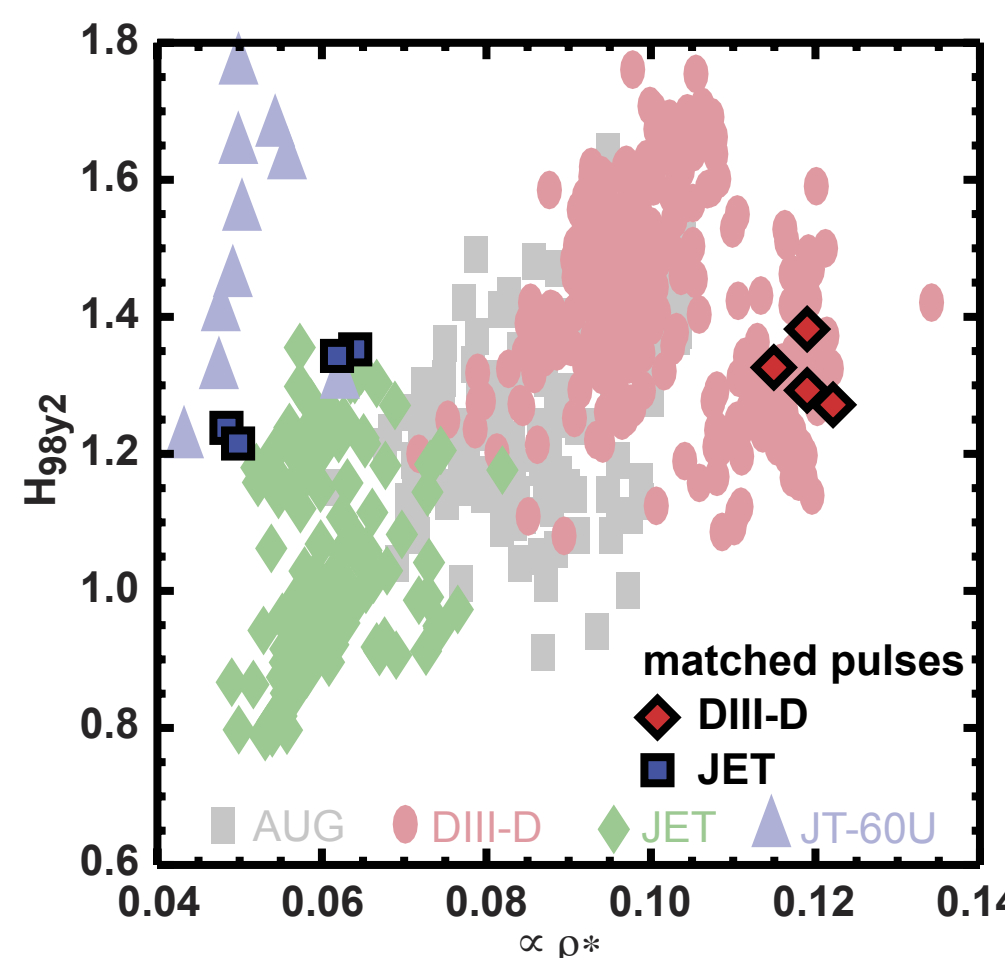
$\kappa_0 = V/2\pi^2 R a^2$
 $q_{CY1} = 5B a^2 \kappa_0 / R$ ($@\rho=0.5$)
 $v_{*e} = n_0 a / T_e^{3/2}$ ($@\rho=0.5$)
 $\beta_{T1} = n_0 (T_e + T_i) / B^2$ ($@\rho=0.5$)
 $\Delta = \text{difference}$
 $\Sigma = \text{sum}$

- The best fit for the eight selected plasmas is $B\tau_{E0} \propto \rho_0^{-1.91}$.

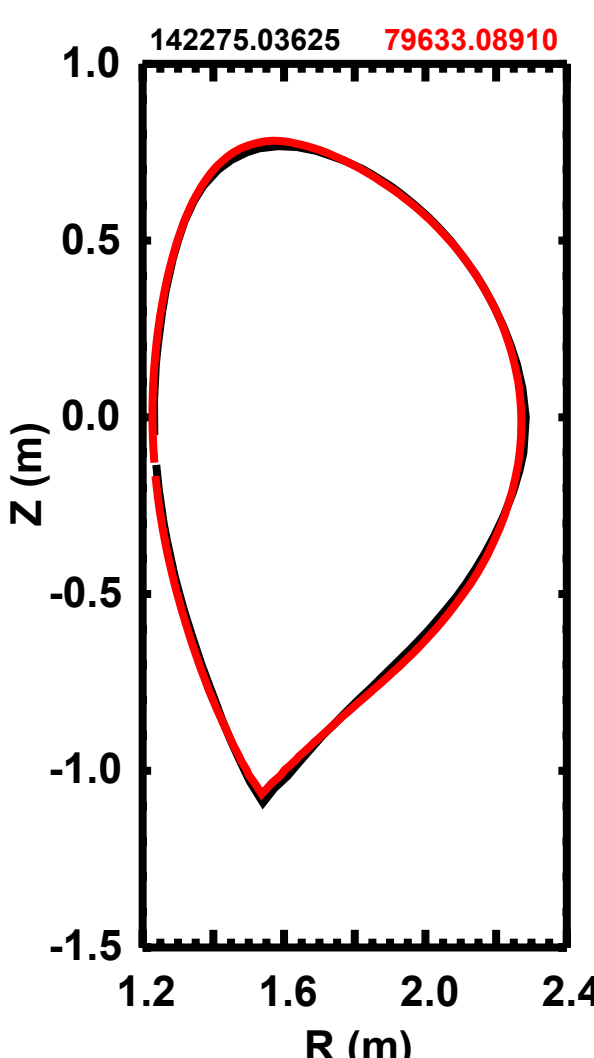


- Here ρ_{*0} is obtained using the target temperature for each point ($T = a^{1/3} B^{2/3}$). Using the volume-averaged T yields $\rho_{*0}^{-2.12}$; using W_{th}/n_e gives $\rho_{*0}^{-2.51}$.
- Can account for small variations in q and β using ITER98(y,2) scaling: $B\tau_{98y2} \propto q^{-3} \beta^{-0.9}$, giving $\rho_{*0}^{-2.30}$.

- The ITPA database of hybrid discharges (AUG, DIII-D, JET, and JT-60U) shows a strong dependence of H_{98y2} on ρ^* . When only the matched plasmas are compared, little or no dependence is seen.



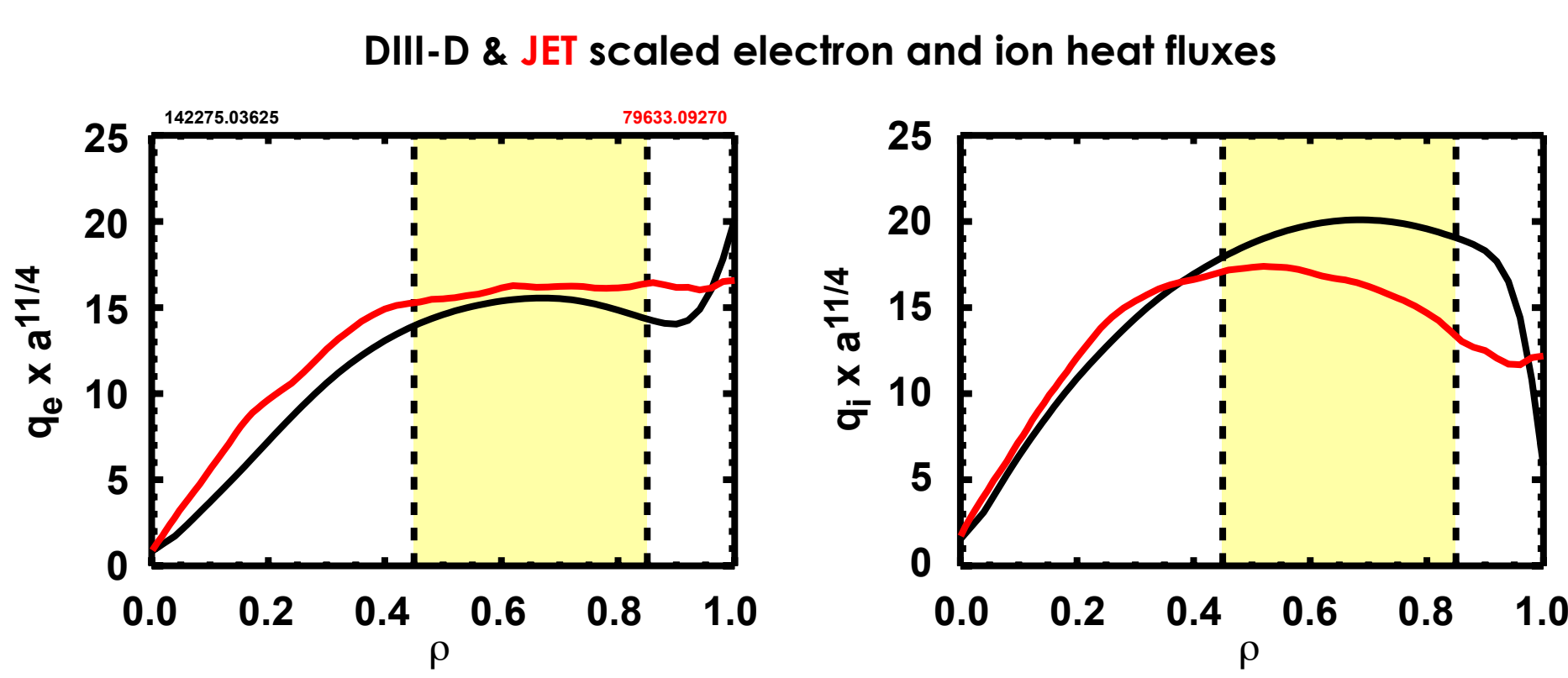
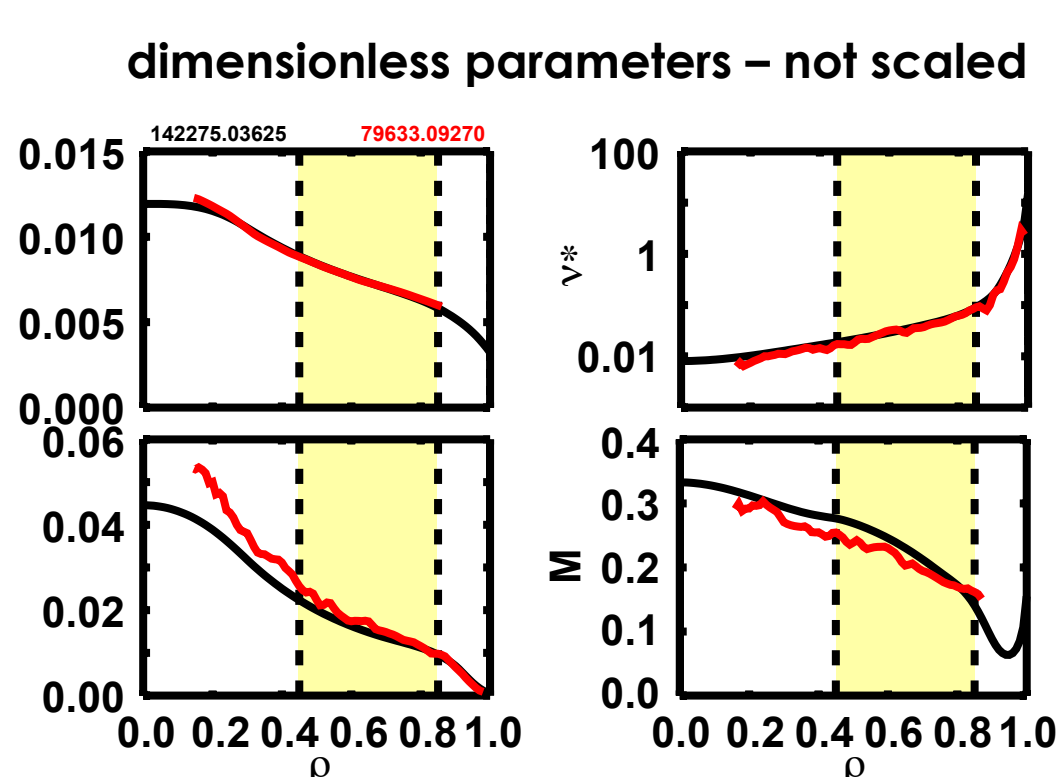
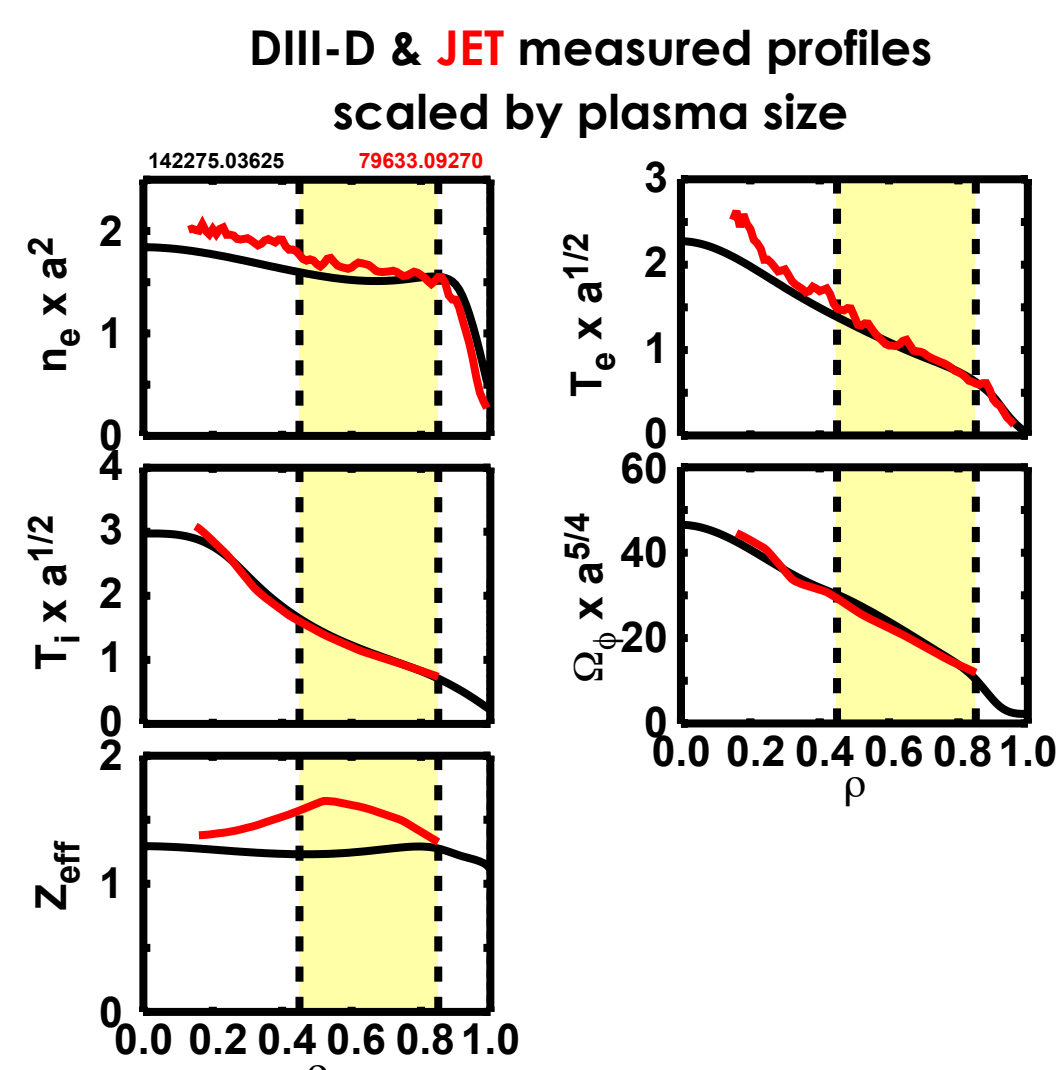
JET and DIII-D shapes are very well matched



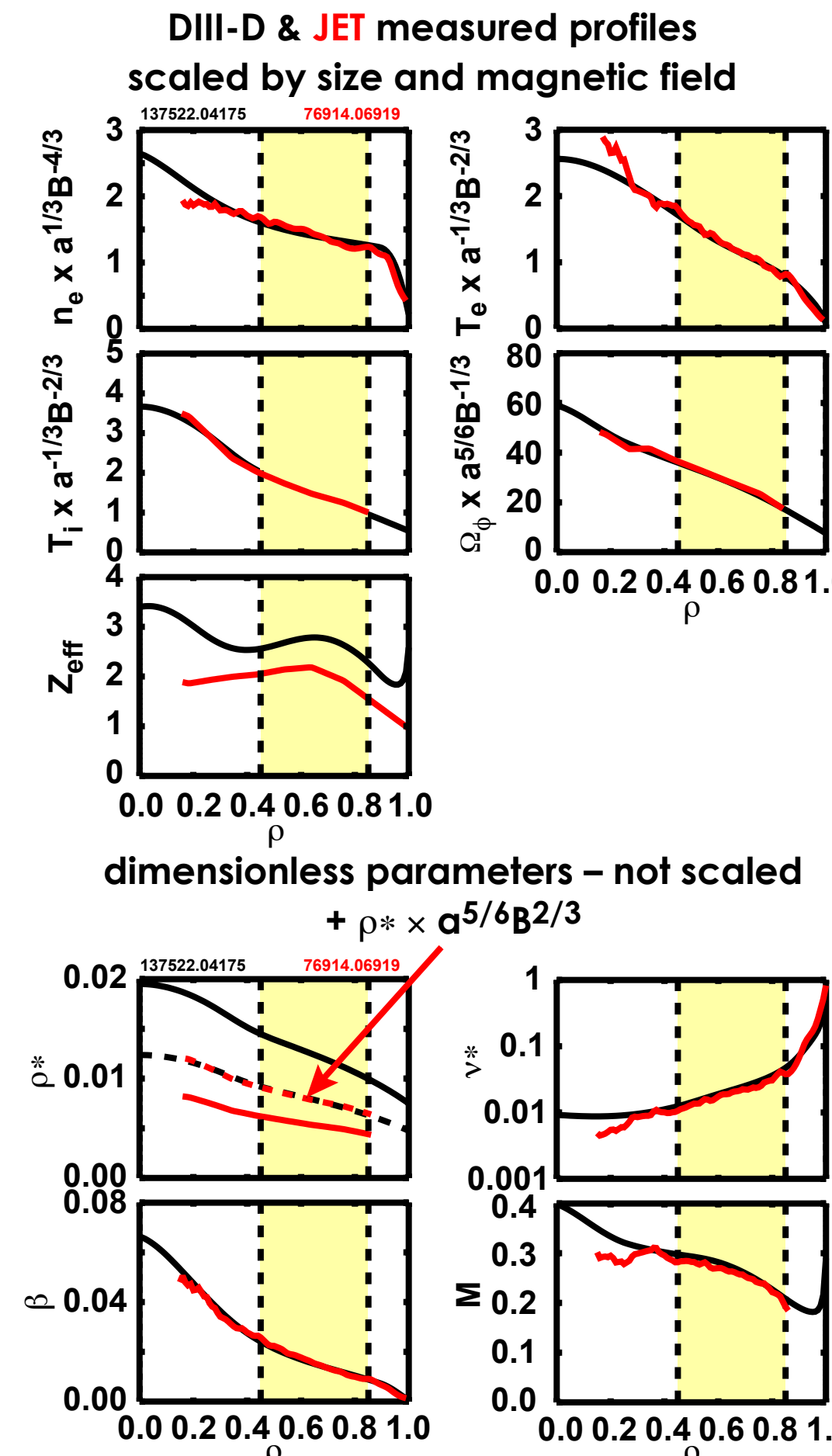
- Discharges from identity comparison data set.
- JET plasma dimensions reduced by 1.675.
- rms gap between shapes is 19 mm – largest deviation is lower-outer squareness.
- Shapes are similar to ITER:
 - $\epsilon = 0.31$ (ITER = 0.32)
 - $\kappa = 1.75$ (ITER = 1.85)
 - $\delta = 0.36$ (ITER = 0.480)

Identity matches confirm physics basis

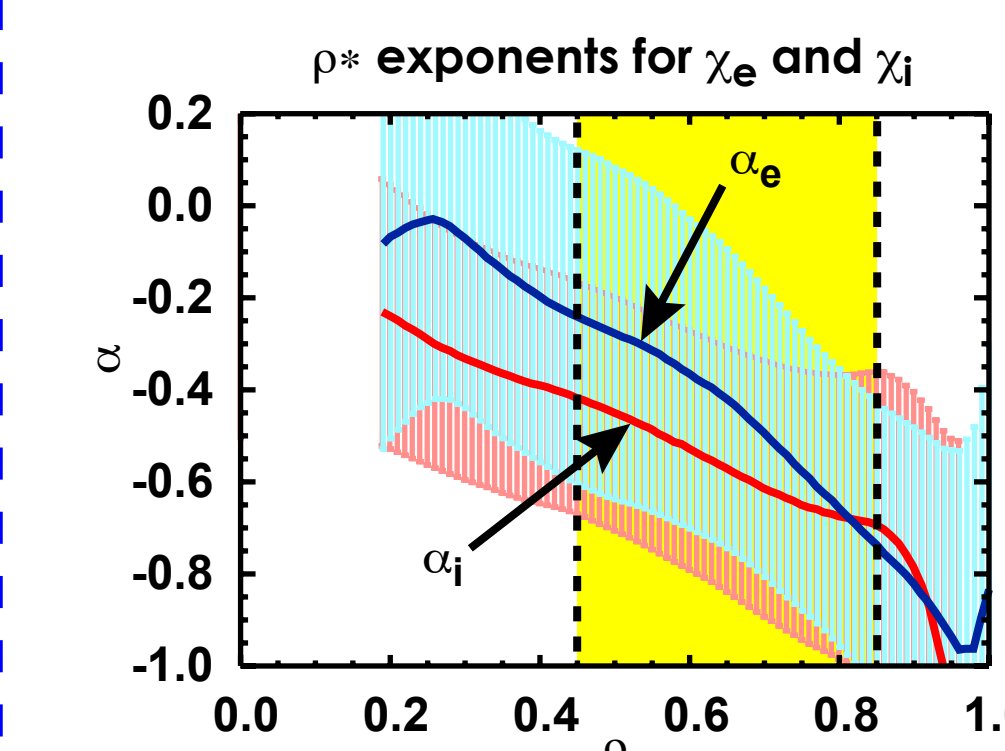
- Identity match:
 - only the plasma dimensions are varied
 - all dimensionless parameters are fixed.
- Good match indicates
 - same physics governs both tokamaks
 - the set of dimensionless parameters represents these processes.
- Profiles compared for $0.45 \leq \rho \leq 0.85$ (52% of plasma volume)
- Good matches found for pairs of time slices in three JET and eight DIII-D discharges.
- Dimensionless parameters (ρ^* , β , v_{*e} , M) match without scaling.
- Test whether same physics is occurring:
 - compare scaled heat flux: $q = -n\chi VT \propto a^{11/4}$, $(a_{JET}/a_{DIII-D})^{11/4} = 4.1$



Matching at extremes of ρ^* range gives local transport scaling – also Bohm-like



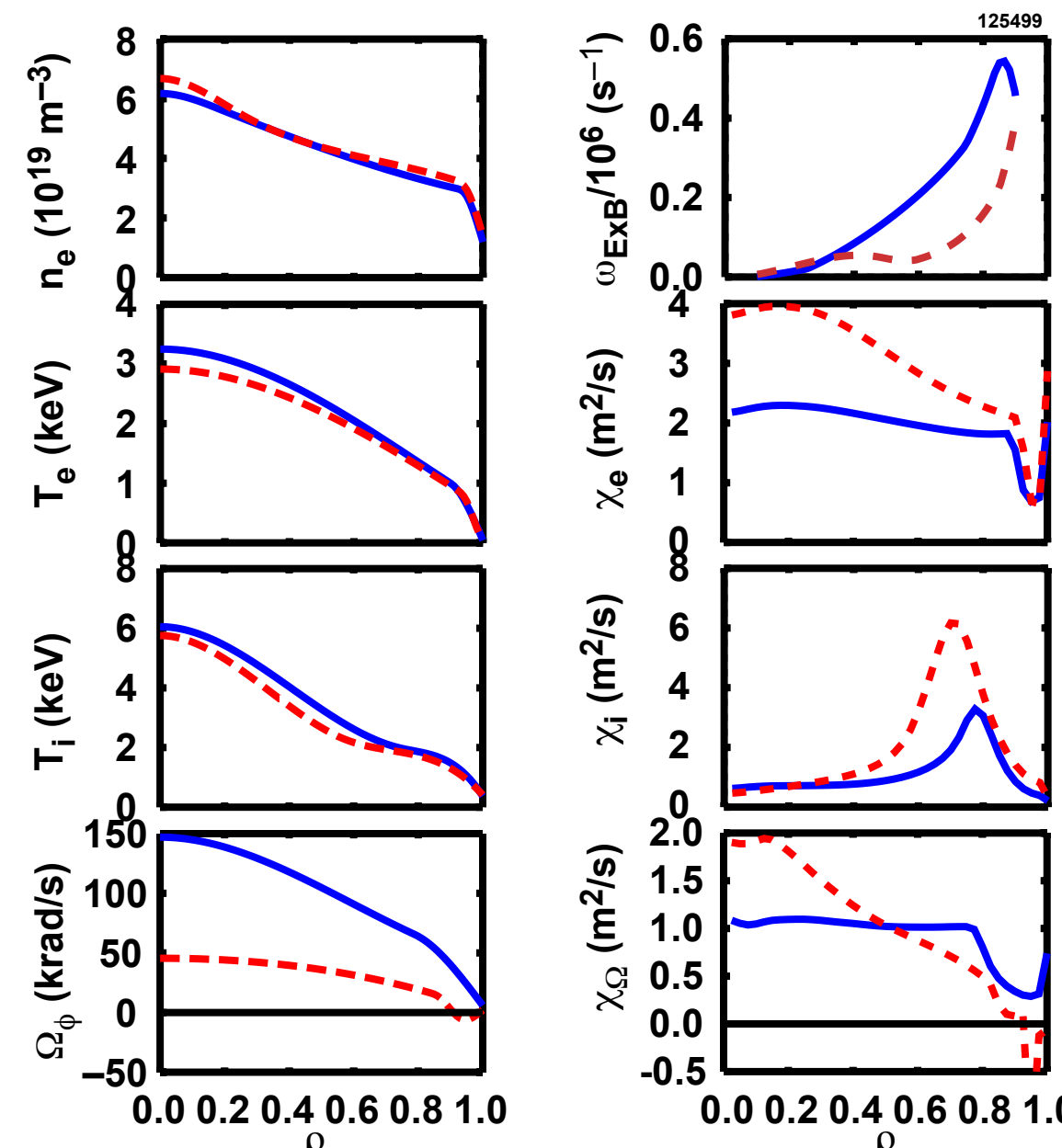
- compare lowest ρ^* in JET with highest in DIII-D – select pairs with $\rho^*_{DIII-D}/\rho^*_{JET} \geq 2.3$ @ $\rho = 0.65$
- Measured profiles match well when scaled by a and B
- Dimensionless parameters (except ρ^*) match without scaling, – ρ^* profiles match when scaled.
- For $q = -n\chi VT$ and $\chi = \chi_B \rho^{\alpha}$, $q \sim a^{-2} B^{5/3} \rho^{\alpha}$
- α can be determined from the ratio q_{JET}/q_{DIII-D} (scaled).



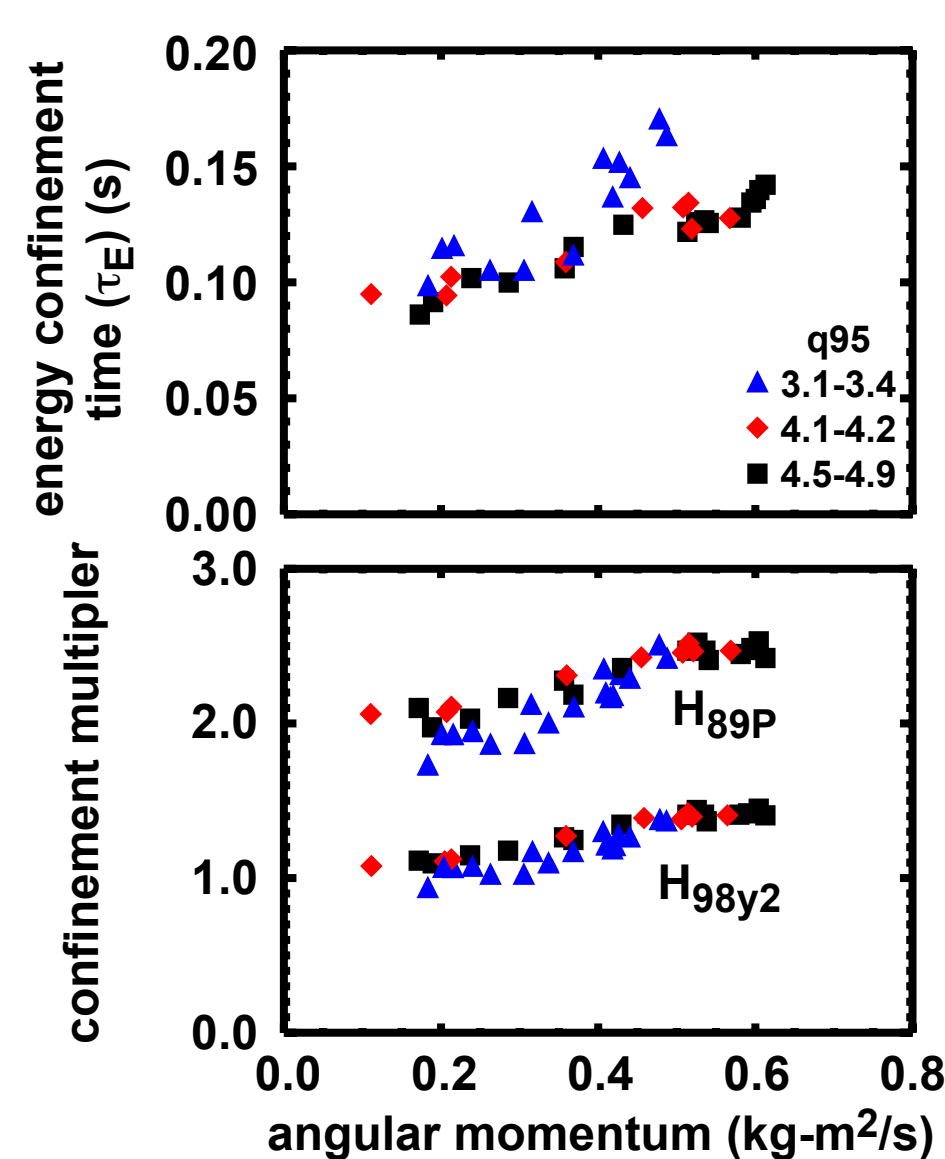
- Mean and std. dev. for best 25 pairs.
- Result is consistent with global confinement scaling.
- $\alpha_e = \alpha_i$.
- Strong dependence on radius – α more negative as radius increases.
- Indicates that scaling to ITER will be Bohm-like.

In DIII-D AI plasmas, global confinement improves as rotation increases; effect of NTM island is also reduced

- Compare high and low input torque at constant β , – vary the mix of co- and counter-NBI.
- In spite of large difference (factor of 3) in rotation, density and temperature profiles are little changed.
- Large decrease in ExB flow shear.
- Large (factor of 2) increases in all transport coefficients.

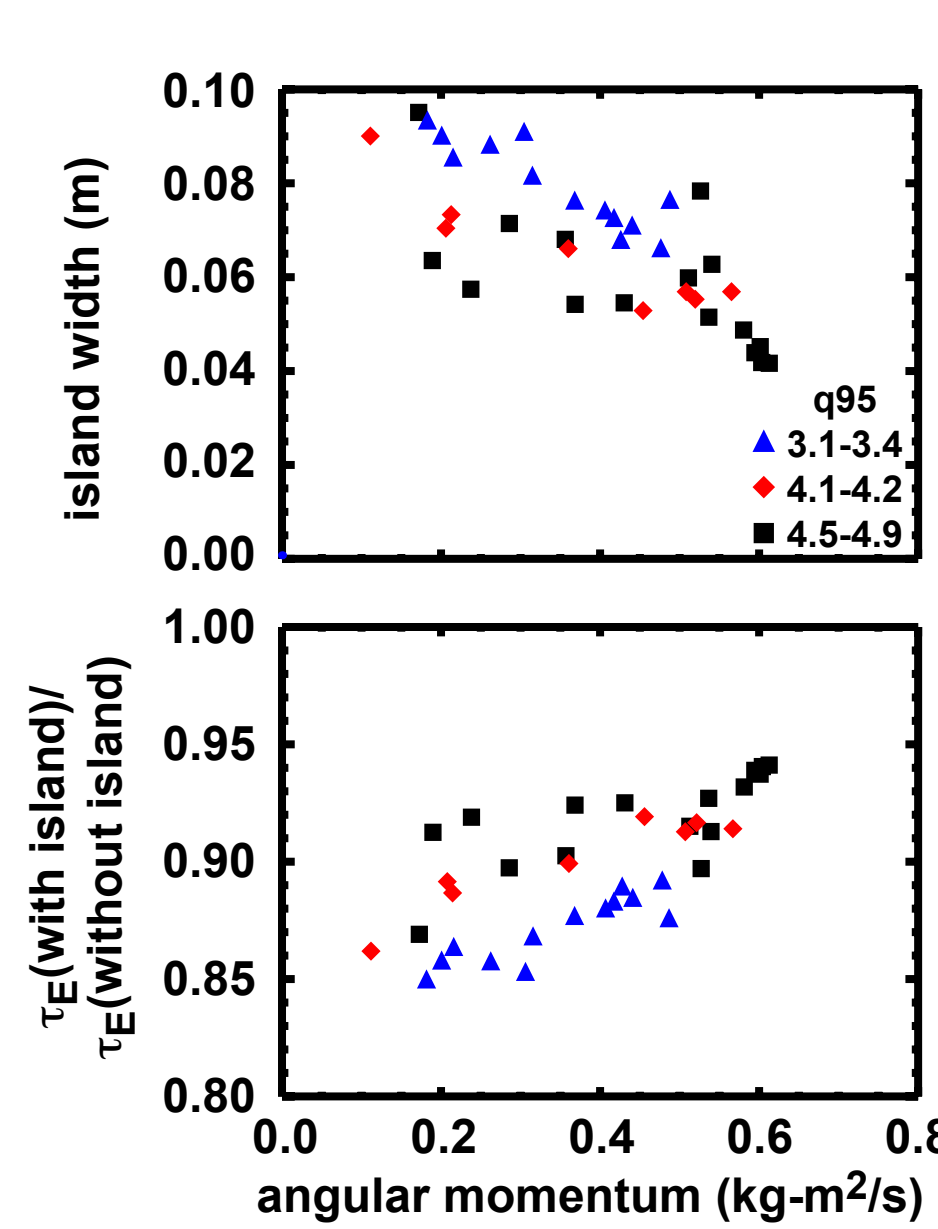


- From lowest to highest accessible rotation (factor of ~ 4.5), both H_{89P} and H_{98y2} increase by $\sim 25\%$.
- Dominant effect is increase in ExB flow shear – supported by transport modeling.
- Energy confinement time is more sensitive to change in rotation at lower q_{95} .



– Central Mach number is proportional to angular momentum: $M(0) = 0.83 L$

- Island width of $m/n = 3/2$ NTM decreases as rotation increases. – probably due to increased flow shear.
- Effect on confinement is stronger at low q_{95} – island width is larger and $q = 3/2$ surface is at larger radius.
- Difference in confinement effect of NTM at low vs high rotation is 4-6% – effect of NTM is smaller than that of ExB flow shear.



Density decrease with ECH appears to be coupled to rotation change

- DIII-D experiments have addressed the dependence of confinement on T_e/T_i .
- For a matched pair of discharges, with the same values of β and rotation – one with 3.5 MW of ECH and the other using counter-NBI to match the rotation, – the same density decrease occurs in both.

- This appears to indicate that the density 'pump-out' often seen with ECH is an indirect consequence of electron heating – coupled to the reduction in rotation
- Indicates an avenue for further research.

