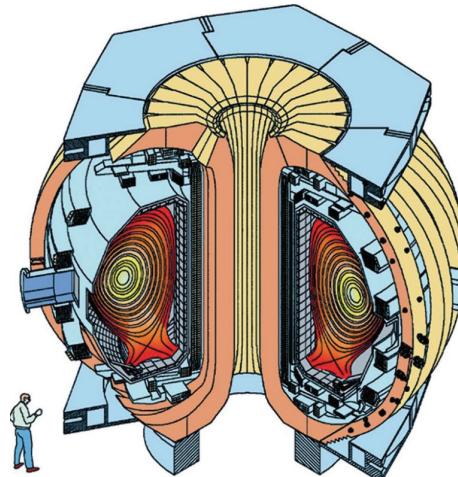


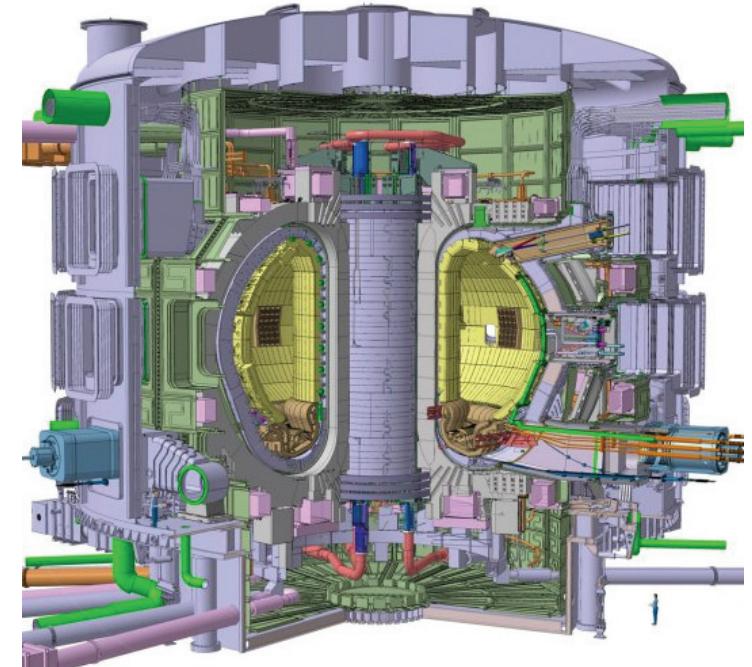
# ITER Test Blanket Module Error Field Simulation Experiments at DIII-D

by  
M.J. Schaffer



Presented at  
the Twenty-Third IAEA  
Fusion Energy Conference  
Daejeon, Republic of Korea

October 11-16, 2010



# A Large International Team Participated

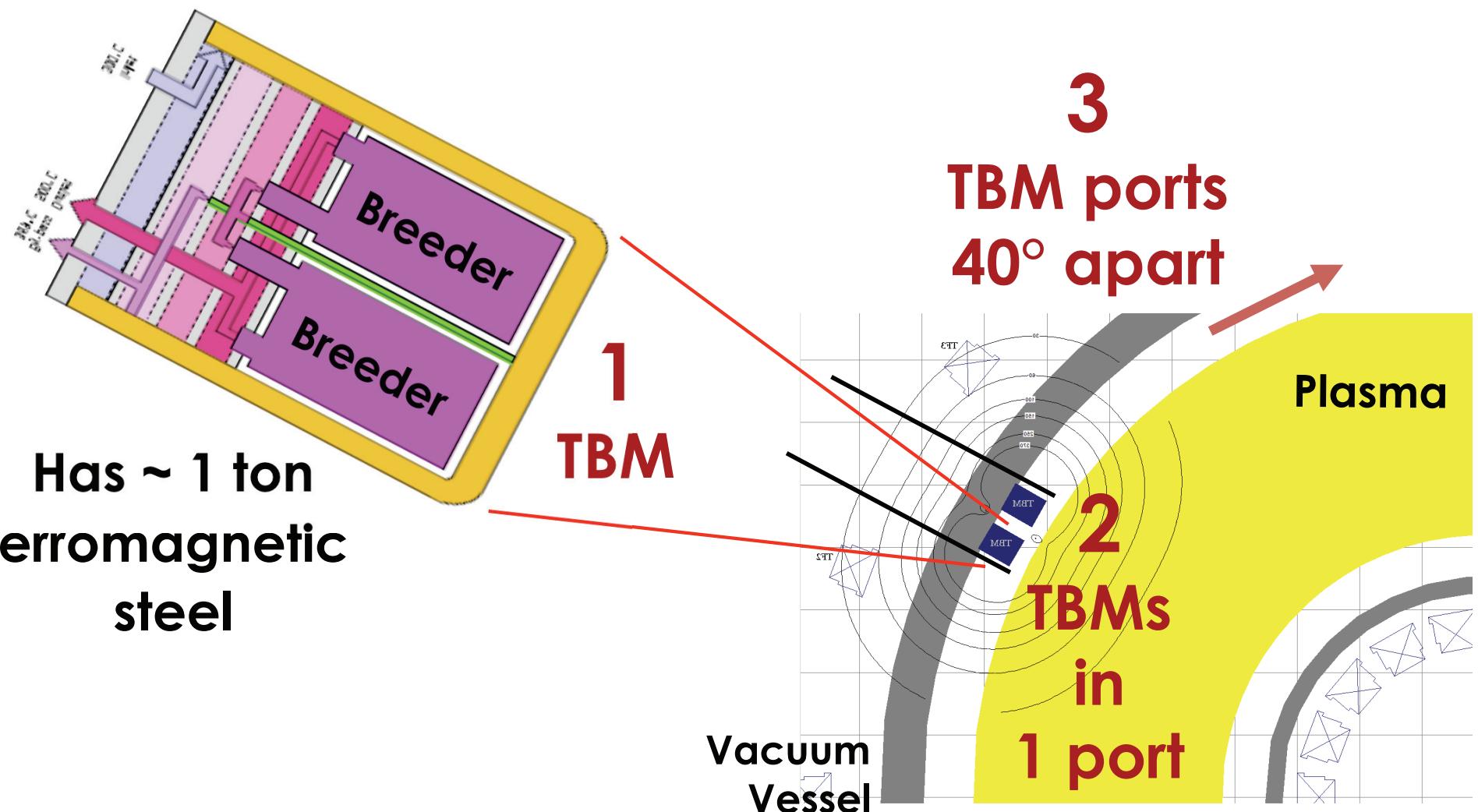
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R. Srinivasan<sup>10</sup>, T.S. Taylor<sup>1</sup>, M.R. Wade<sup>1</sup>, K.-I. You<sup>19</sup>, L. Zeng<sup>17</sup>, and the DIII-D Team<sup>1</sup>

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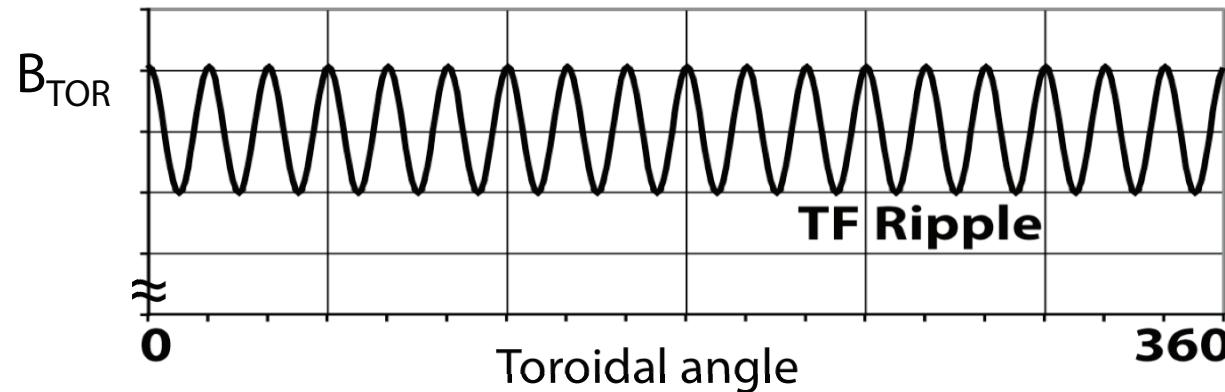
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## Concern Raised About the Impact of Ferromagnetic Test Blanket Modules (TBMs) on ITER Performance



# It Is Well-Known That Toroidal Field Ripple Has Undesirable Effects on Plasmas

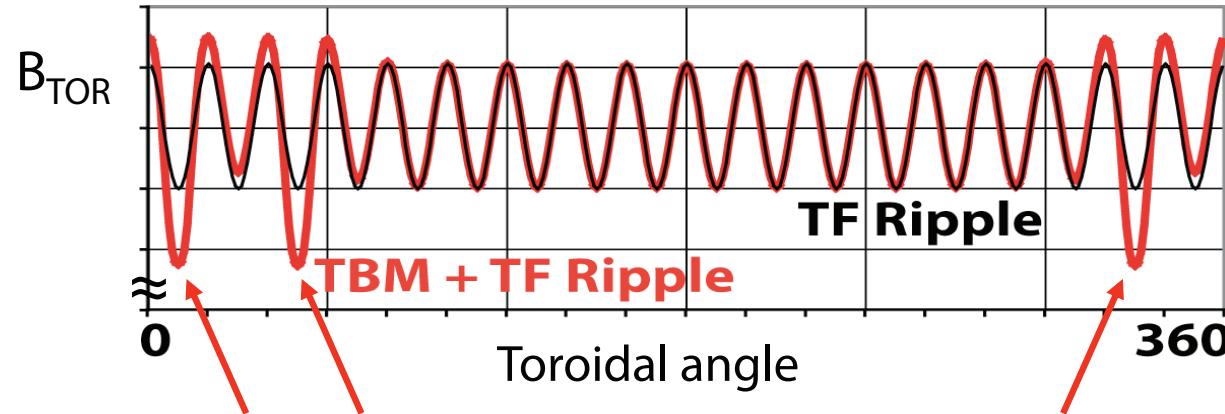
- Common magnetic field ripples made by toroidal field coils are uniformly periodic



- DIII-D matched ITER TF coil ripples for most of the TBM experiments
  - 0.35 ~ 0.4% ripple for both

# TBMs Complicate Ripple Geometry

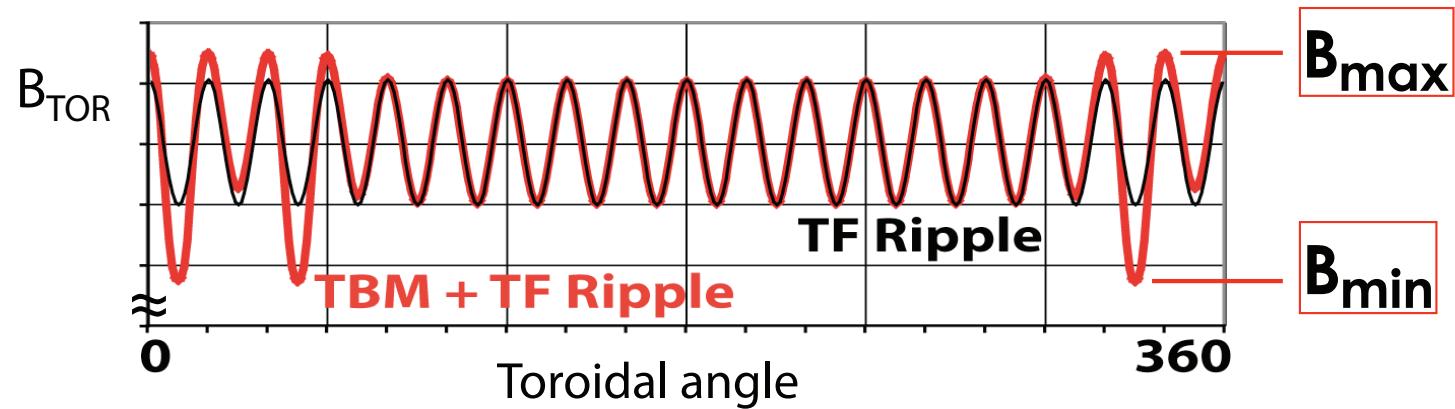
- Larger perturbations
- Broken periodicity



- Magnetized TBM steel makes 3 local “dips”
- Cannot predict consequences, NEED EXPERIMENTS

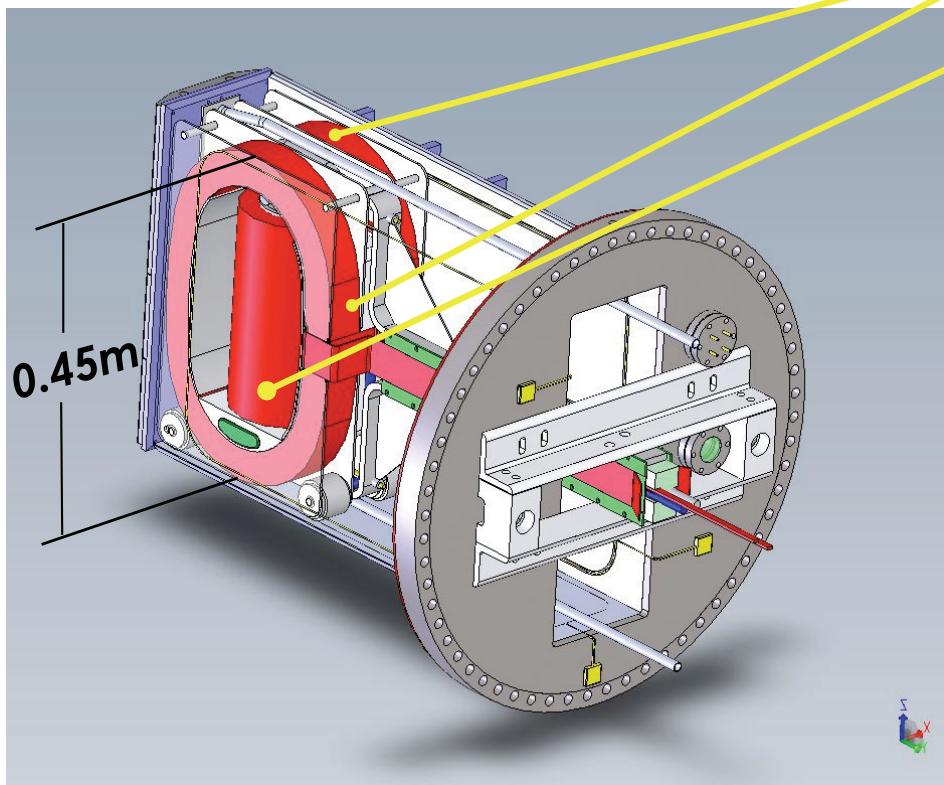
# Use the Combined “Local Ripple” $\delta$

$$\delta = \frac{B_{\max} - B_{\min}}{B_{\max} + B_{\min}}$$



# We Designed a Mock-up of the TBM Field to Measure Its Effects on DIII-D Plasmas for ITER

## Mock-up Approximates the Magnetization at One ITER Port



Racetrack coils ~  $M_{TOR}$  TBM

Vertical solenoid ~  $M_{POL}$  TBM

- Matches ITER TBM field well
- Moveable,  $\Delta R = 0.28$  m
  - 1.0 m in ITER
- Up to ~3 x ITER local ripple
  - To match surface-average amplitude of 3 ITER TBM ports

# Experimental Results Showed that Most TBM Effects Were Small, and the Remainder May Be Correctable

Nil or Small Effects on

Plasma Startup  
L-mode Confinement

L-to-H Transition

ELM Characteristics

ELM Control by RMP

Global Fast Ion Loss

Divertor Power

**Of Concern\* if  
Scaling is Unfavorable**

Braking of plasma rotation

Reduction of H-mode confinement

(both at high  $\beta$ )

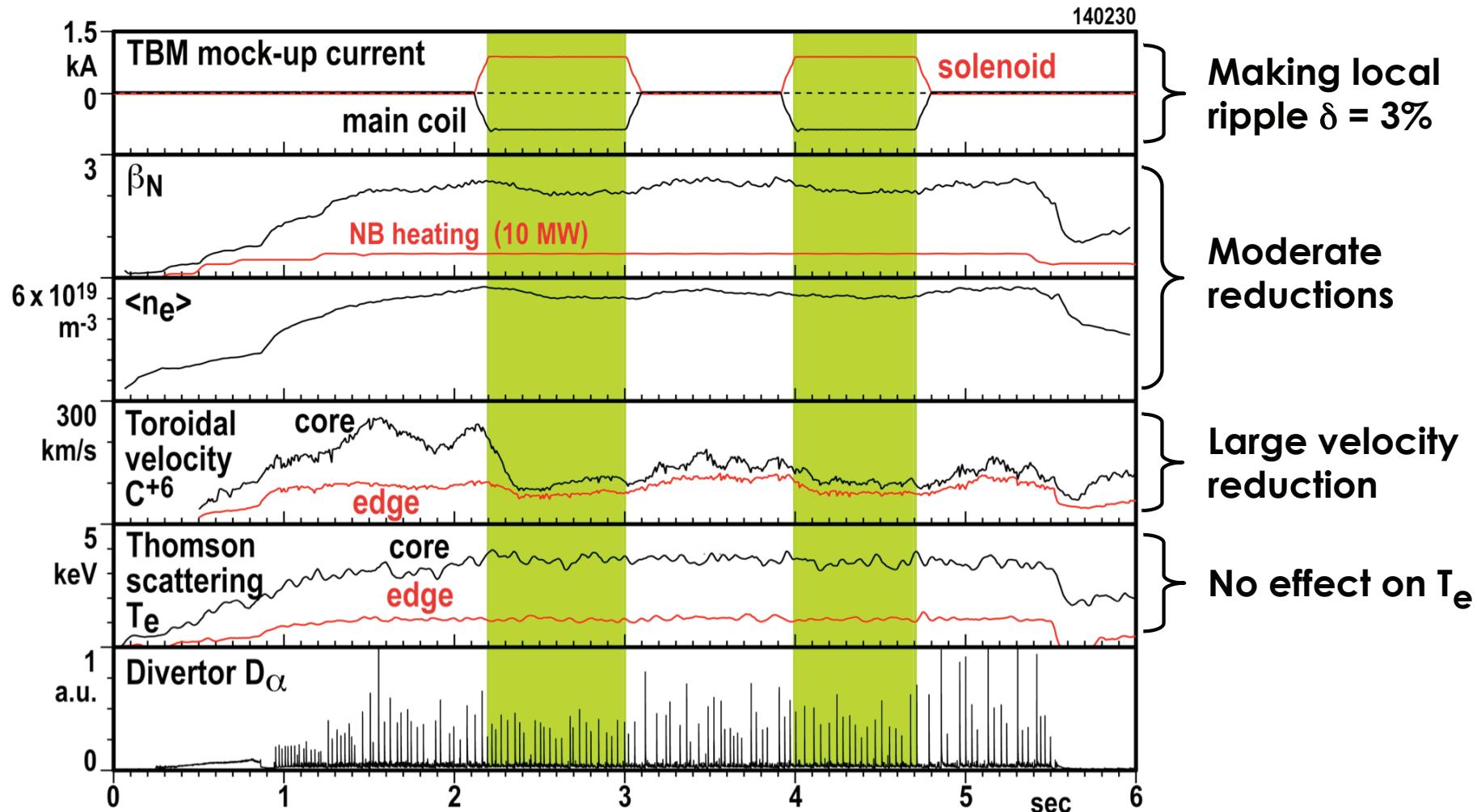
\* to ITER High-Gain Mission

**May Be  
Correctable**

n=1 error compensation eliminated TBM contribution to locking in L-mode experiments

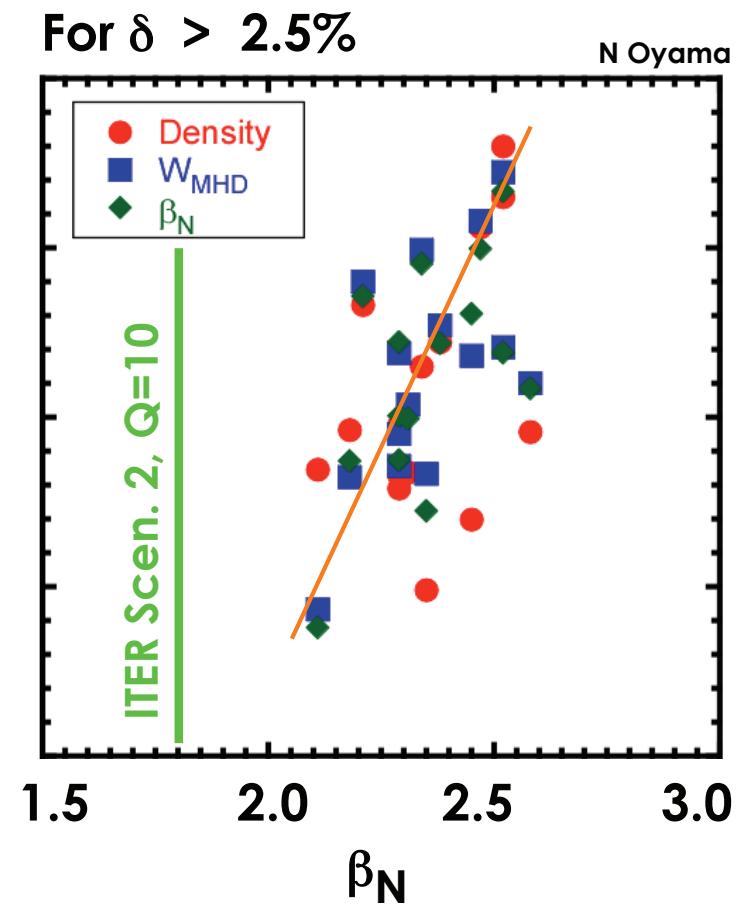
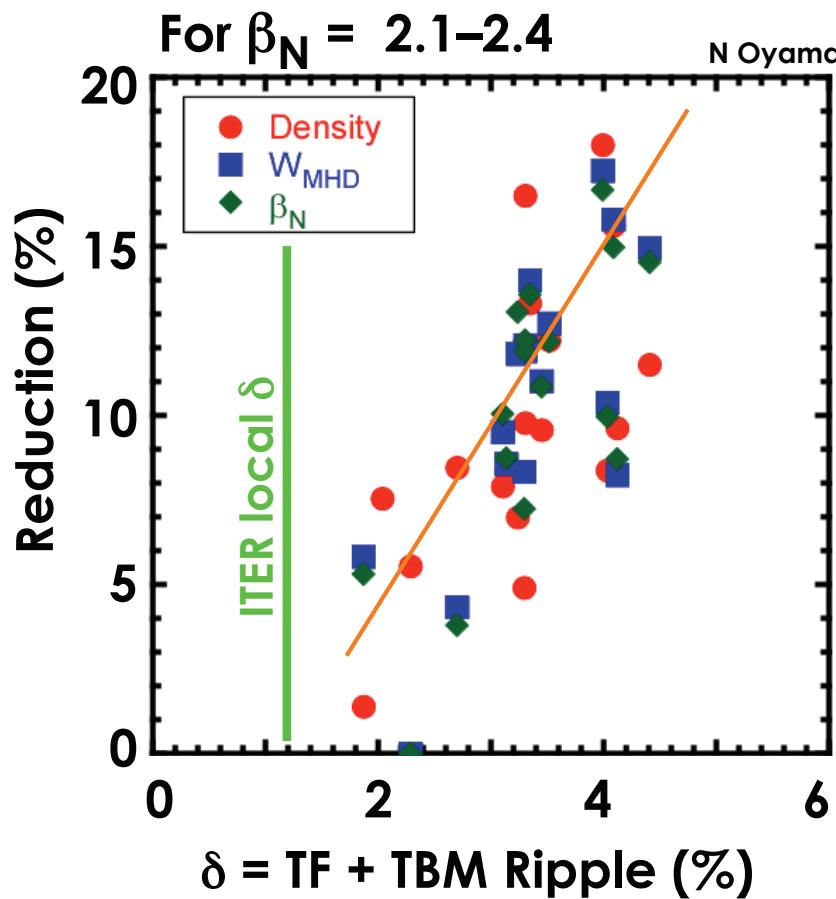
Numerical model predicts n=1 compensation will reduce braking in H-mode plasma

# Greatest TBM Effect Was Toroidal Velocity Reduction – $\beta$ , $n_e$ , $T_e$ Were Less Affected

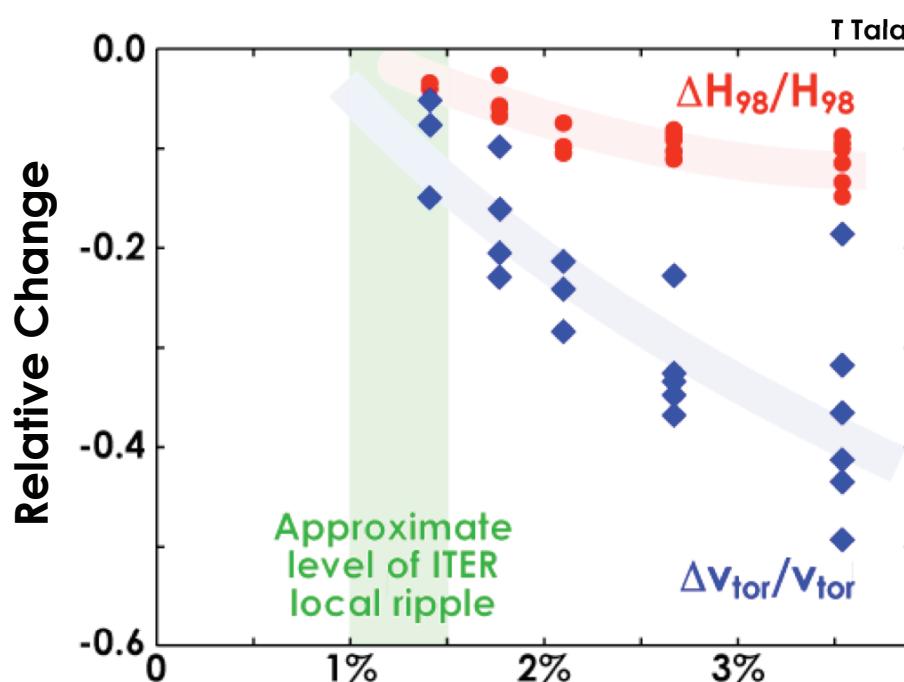


# TBM Effects Were Small at ITER Baseline Parameters. – We Did Systematic Scans at Higher Ripple and $\beta$ .

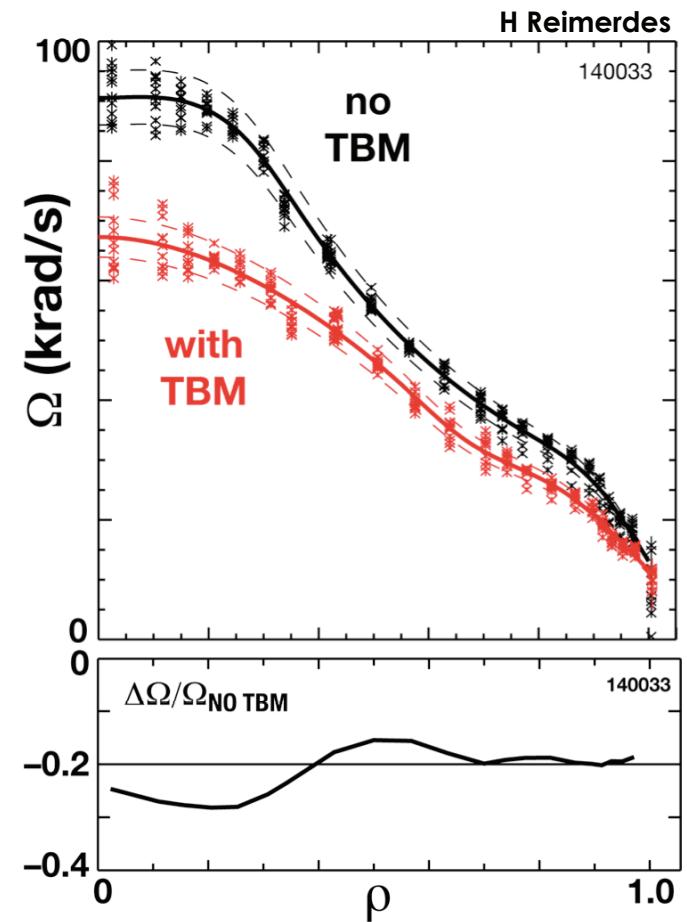
$$R_{\text{midout}} = 2.30 \text{ m} \quad q_{95} = 3.5 \quad B_T = 1.7 \text{ T} \quad I_p = 1.4 \text{ MA}$$



# The TBM Field Reduced Toroidal Velocity Much More Than H-mode Confinement

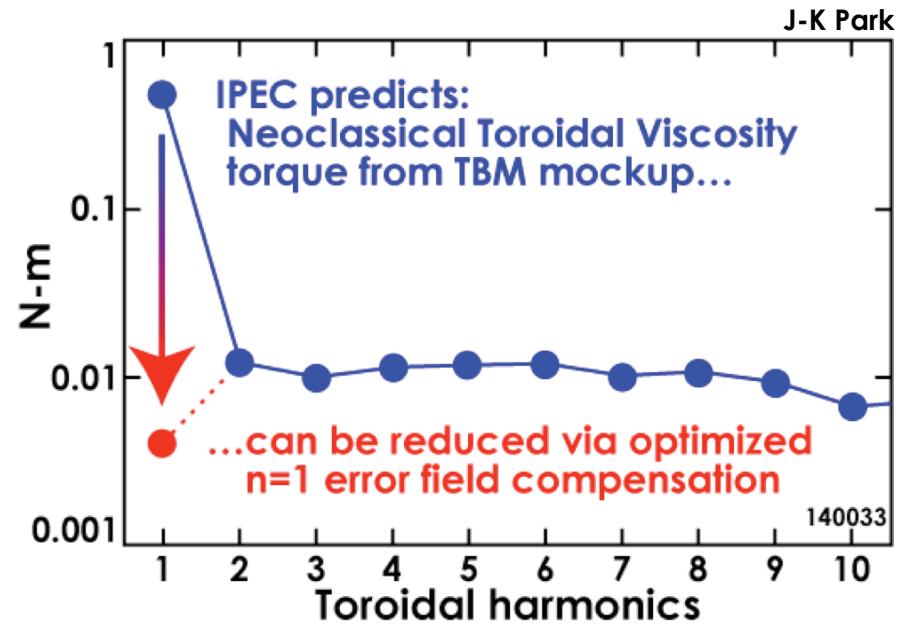
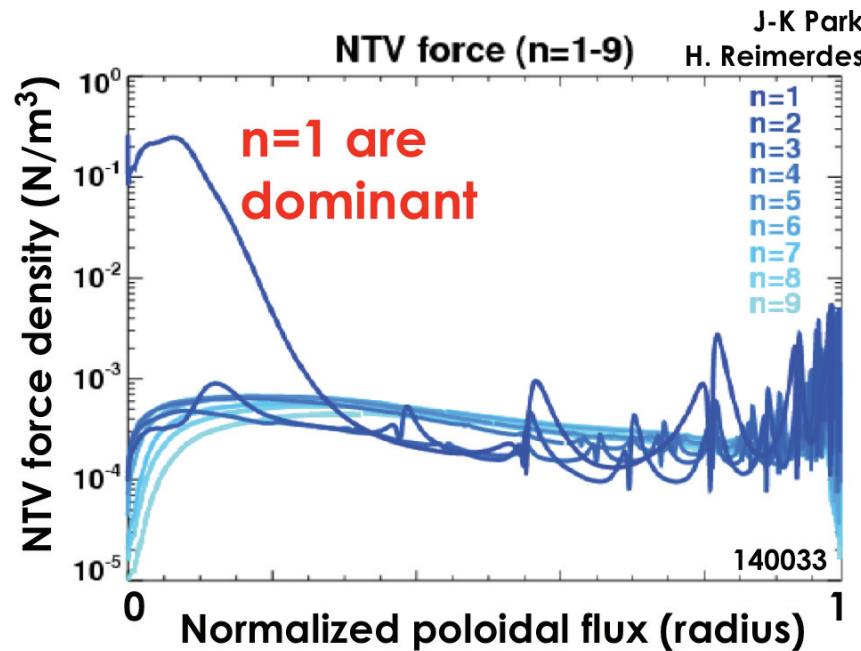


**Relative  
velocity change  
was ~ 3 times the  
confinement change**



$\Delta\Omega/\Omega \approx \text{constant}$   
across plasma radius

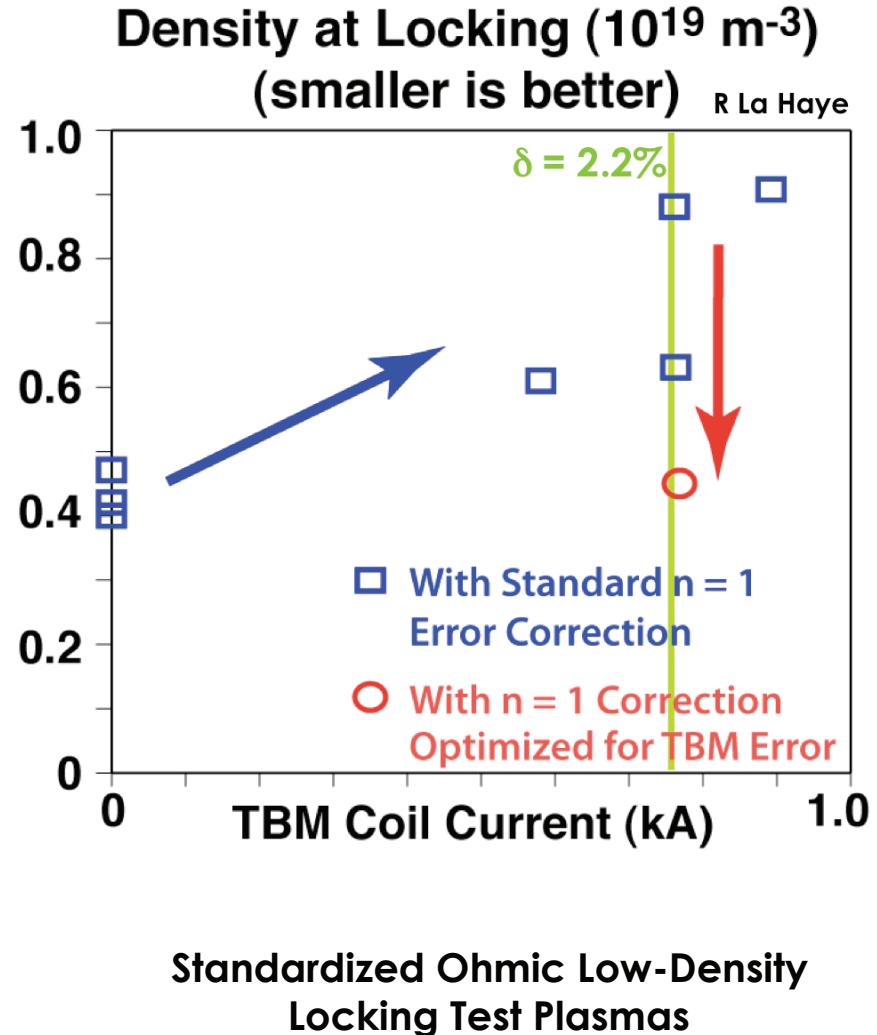
# The Ideal Perturbed Equilibrium Code (IPEC) Implicates n=1 Harmonics in Toroidal Rotation Braking



- Mock-up Neoclassical Toroidal Viscous braking is mostly from n=1 harmonics in central core
  - n=1 is **amplified** by H-mode

- A great opportunity...
- One expects higher plasma speed and restored confinement

# $n=1$ Error Compensation Was Optimized for TBM, and It Restored Locked Mode Tolerance at low $\beta$



- TBM field raised the low-density threshold for locked mode avoidance
- $n=1$  compensation restored locked mode tolerance in low- $\beta$  test
- IPEC analysis and experiment ≈ agree
- Will  $n=1$  compensation also restore H-mode confinement at high- $\beta$ ?

# How to Scale from 1 Port in DIII-D to 3 Ports in ITER?

- If TBM effects vary linearly with ripple mirror ratios, and they also sum linearly, then equivalence is  $\delta_{\text{DIII-D}} = 3 \delta_{\text{ITER}}$ 
  - for case of 1 DIII-D vs. 3 ITER ports
- If TBM effects  $\propto \delta^\alpha$  and  $\alpha > 1$ , then  $(\delta_{\text{DIII-D}})^\alpha = (3 \delta_{\text{ITER}})^\alpha$  makes TBM effects 9  $(\delta_{\text{ITER}})^\alpha$  in DIII-D vs. 3  $(\delta_{\text{ITER}})^\alpha$  in ITER
  - $\alpha \approx 2$  for DIII-D density reduction data (V Chuyanov)
  - DIII-D TBM experiments may be more pessimistic than ITER reality
- A different view: ITER rotation may be too small for the TBM error to be important

# Summary

- DIII-D experiments did not reveal any reason why a TBM-like error field would seriously limit ITER plasma performance objectives
- IPEC numerical analysis indicates that NTV braking is dominated by plasma-amplified  $n=1$  TBM error field
  - IPEC predicts rotation recovery by  $n=1$  compensation
- A DIII-D experiment demonstrated  $n=1$  compensation of the TBM contribution to locking in OH plasmas
- DIII-D plans experiments in 2011 to test if  $n=1$  re-compensation can obviate rotation and confinement degradations in high- $\beta$  H-mode plasmas



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# Backup Slides



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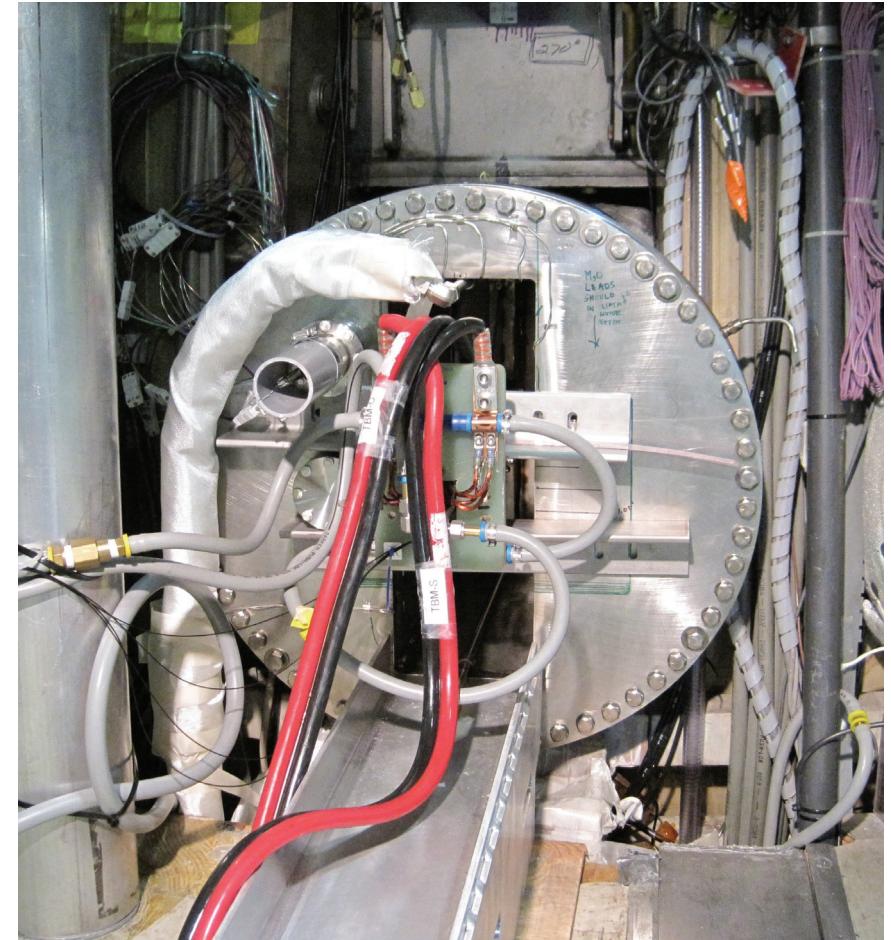
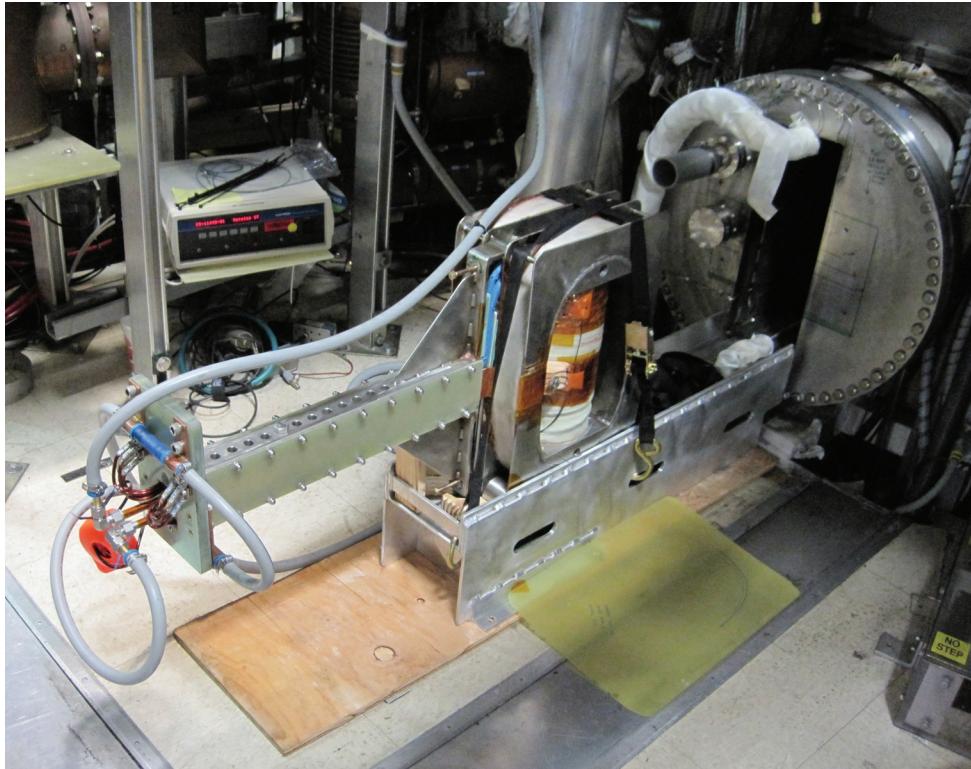
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# TBM Mock-up at its DIII-D Port

**Mock-up secured in its channel  
with cooling water attached**



**Mock-up rolled into port**

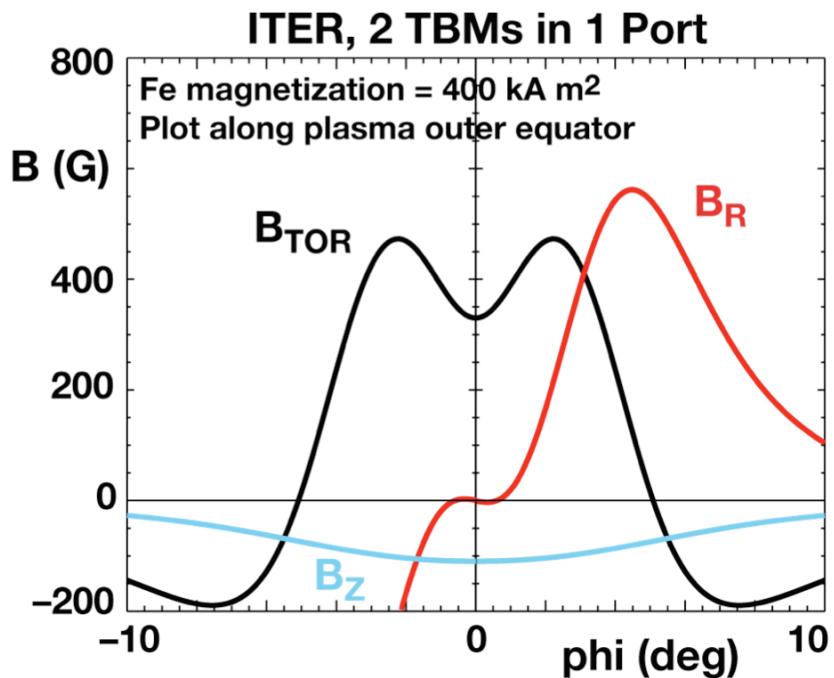
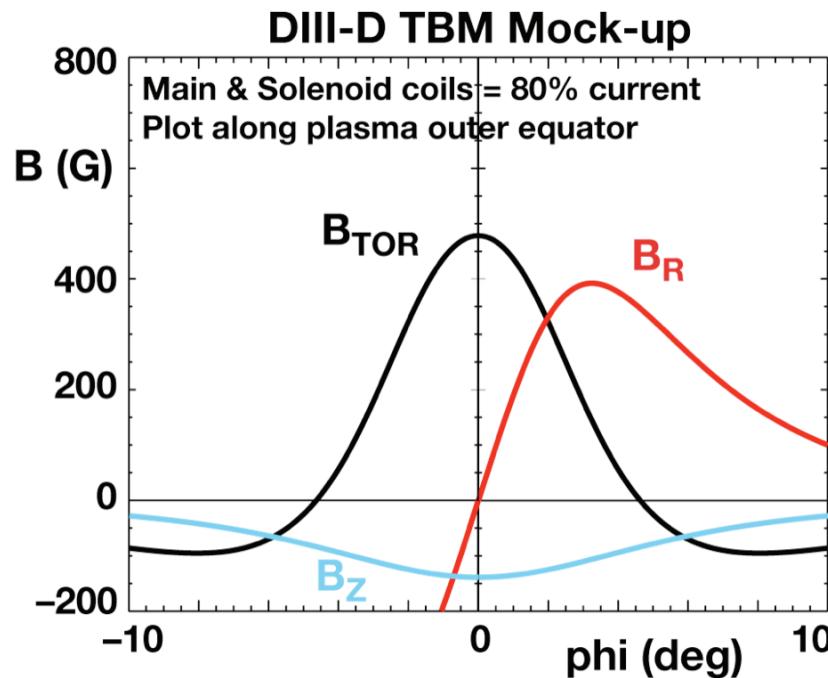
# Two Main Differences Between DIII-D and ITER:

- 1 DIII-D TBM port vs. 3 ITER TBM ports

No validated theory for extrapolation

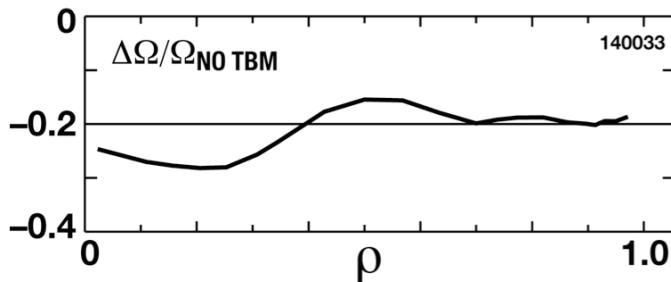
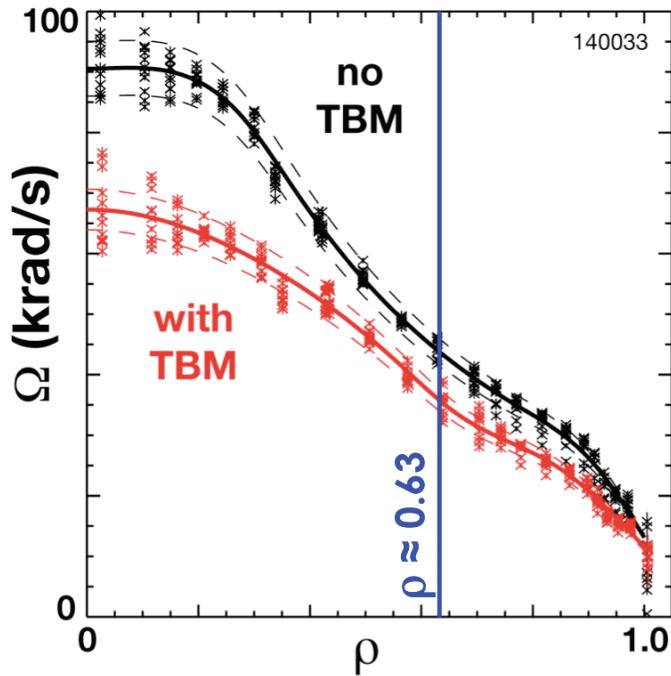
- Near-field details

Limited by DIII-D port width

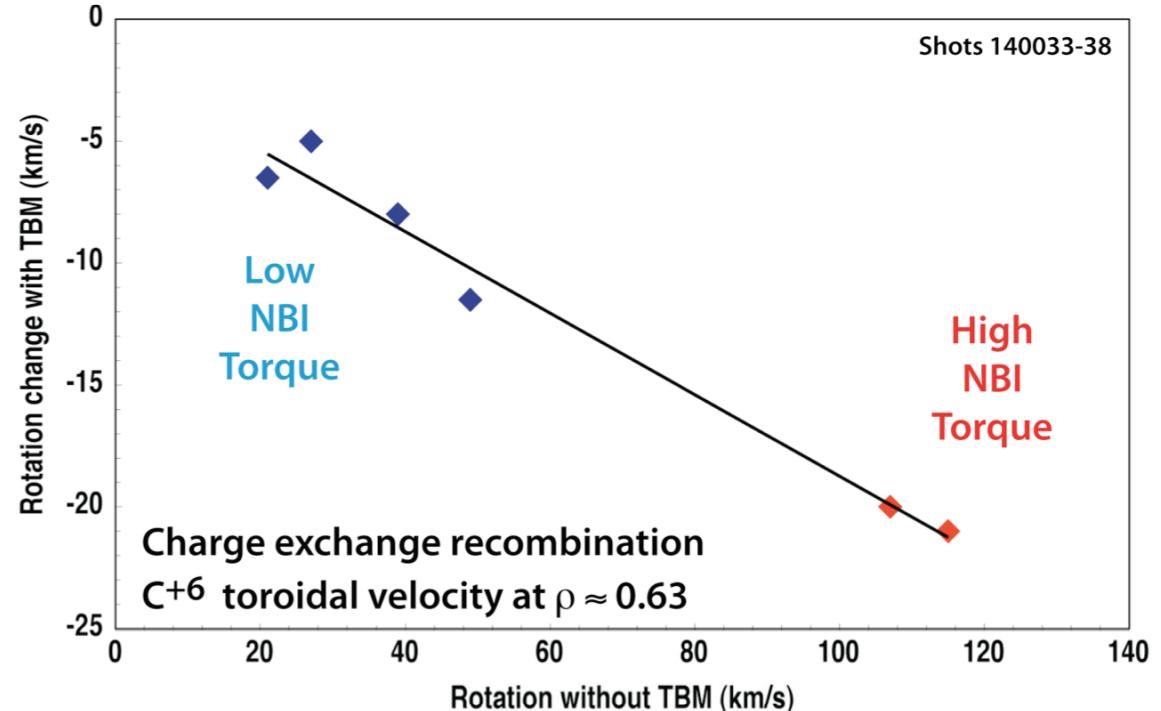


# Mock-up Field Exerts Non-Resonant Braking on Plasma Toroidal Rotation, $v_T = \Omega R$

H Reimerdes



H Reimerdes



- $\Delta\Omega/\Omega \approx \text{constant}$ , either way