L-H Transition Studies on DIII-D to Determine H-mode Access for Operational Scenarios in ITER

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Background/Motivation

- Can H-mode be achieved in the first (non-nuclear) phase of ITER operations with He (and/or H) plasmas ?
- Need to access H-mode in non-nuclear phase of ITER operations
 - Assess machine hardware and systems in higher performance H-mode plasmas e.g. heat loads, fueling, heating, etc.
 - Test ELM mitigation techniques and hardware in ITER environment
- Reduce the large scatter in H-mode power threshold database and large error in scaling predictions
 - Examine physical trends not included in P_{TH} scaling
 - Obtain physics basis for the scaling laws
- Determine methods to reduce the H-mode power threshold and extrapolate to ITER
- Knowledge beyond the L-H transition is important; quality of H-mode performance dependent on input power above threshold power
 - Affects pedestal behavior, ELM characteristics, etc.



H-mode Power Threhold Increases Smoothly from Near Pure D Plasmas to Near Pure He Plasmas

- No sudden change in P_{TH} observed
 - He concentrations exceeding 40% exhibit discernible increase in PTH
- D plasmas: $I_p = 1.0 \text{ MA}$, $B_T = 1.65 \text{ T}$, $n_e = 2.8-3.0 \times 10^{19} \text{ m}^{-3}$ with ECH





The Plasma Geometry in Vicinity of Divertor has a Strong Effect on PTH

- Decreasing the height of the X-point above the divertor surface significantly decreases P_{TH} (> factor of 2)
- ECH into He plasmas





The X-point height has a Strong Effect on the H-mode Power Threshold for H, D and He

- Effect previously observed on DIII-D and other devices
- First systematic study of effect for H, D and He



• Preliminary analysis indicates edge neutrals may be affecting the power threshold



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- H-mode power threshold scaling for D plasmas
 - P_{TH} , SCAL08 (D) = 0.049 $n_e^{0.72} B_T^{0.80} S^{0.94}$ (units: 10²⁰ m⁻³, T, m²)
- X-point dependence is not included in the power threshold scalings
 - Results in factor of 2 difference between
 P_{TH} at low X-point and the scaling prediction

• Preliminary analysis indicates edge neutrals may be affecting the power threshold



Edge E_r Shear and Edge Magnetic Shear Show No Significant Change with X-point Height







Difference in the H-mode Power Threshold Between He and D Plasmas Decreases at Higher Densities

- He and D plasmas ($I_p = 1.0 \text{ MA}, B_T = 1.65 \text{ T}$)
 - Balanced NBI (i.e. zero torque) at same ion species as plasma species
 (D NBI → D; He NBI → He)
 - ECH
 - High X-point location
- At low densities (<3x10¹⁹ m⁻³)
 P_{TH} (He) ~1.5-2 P_{TH} (D)
- At high densities (>3x10¹⁹ m⁻³)
 P_{TH} (He) ~1-1.5 P_{TH} (D)





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 P_{TH} (He) ~1-1.5 P_{TH} (D)
- Lowering the X-point will move all curves significantly downwards with respect to the scaling





Application of Strong Resonant n=3 RMP Fields Increase PTH in Helium Plasmas

- n=3 resonant magnetic perturbations (RMPs) applied by in vessel coils (I-coils)
- Stronger resonant components lead to higher P_{TH}
- Similar effect observed with ECH





Resonant Magnetic Perturbations Increase the H-mode Power Threshold (Helium Plasmas)

- n=3 resonant magnetic perturbations (RMPs) applied by in-vessel coils (I-coils) to be resonant at specific values of q₉₅ (=3.4)
- 3 He plasma discharges with - No I-coil at q₉₅ = 3.4 - I-coil current = 2 kA at q₉₅ = 3.4 - I-coil current = 2 kA at $q_{9.5}$ = 4.0 Application of I-coil current = 2 kA - Resonant field at $q_{95} = 3.4$ remained in L-mode at up to $P_{ECH} = 3.5 \text{ MW}$ - H-mode achieved again with non-resonant RMP fields ($q_{95} = 4.0$) 2 Careful timing of I-coil activation **q**₉₅ required after L-H transition, but





before first type I ELM

Application of Strong Resonant n=3 RMP Fields Increase P_{TH} for Both ECH and Balanced H-NBI Heating (Helium Plasmas)

• n=3 resonant magnetic perturbations (RMPs) applied by in vessel coils (I-coils)





For D Plasmas, there is a Minimum Required RMP Field Before P_{TH} Increases

- Effect on P_{TH} observed for $\delta B/B_T > \sim 3x10^{-4}$
- Determined for both ECH and balanced D-NBI (plasma shape different to He plasma study)





The H-mode Power Threshold is Unaffected by the TBM

- Test Blanket Module (TBM) magnetic ripple replicated using mock up coils on DIII-D
 - Results are for TF + TBM local ripple ~3.1% (expected TF + TBM local ripple in ITER ~1.3%)
- Determined for ECH, balanced
 D-NBI and co D-NBI in D plasmas





The TBM has no Significant Effect on the H-mode Power Threshold

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Summary

- Strong dependence of P_{TH} on the X-point height at the divertor for H, D and He plasmas (not included in P_{TH} scaling)
- The difference between the H-mode threshold power (P_{TH}) for He and D plasmas decreases at higher densities
- Resonant magnetic perturbations (n=3) increase P_{TH} in He and D
 - Threshold in RMP field for effect on P_{TH} in D
 - Requires appropriate timing of RMP coil activation after L-H transition
- Local magnetic ripple from test blanket module mockup coils have no significant effect on $P_{\rm TH}\, in\, D$ plasmas
 - TBMs not expected to significantly affect P_{TH} in ITER
- Need to include certain dependences (e.g. X-point) and determine underlying physics of all known effects for reliable predictions by H-mode power threshold scalings



