

# Transport and Deposition of $^{13}\text{C}$ from Methane Injection Into L- and H-mode Plasmas in DIII-D

by  
**S.L. Allen\***

for

N.H. Brooks,<sup>†</sup> J.D. Elder,<sup>‡</sup> M.E. Fenstermacher,\*  
M. Groth,\* C.J. Lasnier,\* A.G. McLean,<sup>‡</sup> V. Phillips,<sup>¶</sup>  
G.D. Porter,\* D.L. Rudakov,<sup>§</sup> P.C. Stangeby,<sup>‡</sup> W.R. Wampler,#  
J.G. Watkins,# W.P. West,\* and D.G. Whyte,<sup>△</sup>

\*Lawrence Livermore National Laboratory, Livermore, California, USA

<sup>†</sup>General Atomics, San Diego, California, USA

<sup>‡</sup>University of Toronto Institute for Aerospace Studies, Toronto Canada.

<sup>¶</sup>FZJ Jülich GmbH/Euratom Institut für Plasmaphysik, Jülich, Germany.

<sup>§</sup>University of California, San Diego, California, USA

#Sandia National Laboratory, Albuquerque, New Mexico, USA

<sup>△</sup>Massachusetts Institute of Technology, Cambridge, Massachusetts, USA

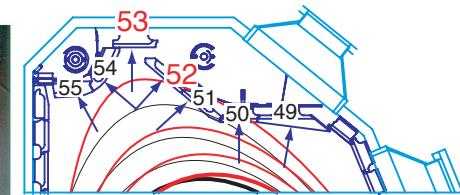
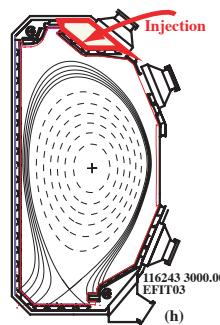
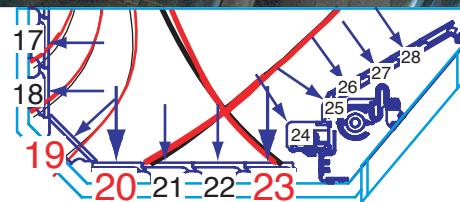
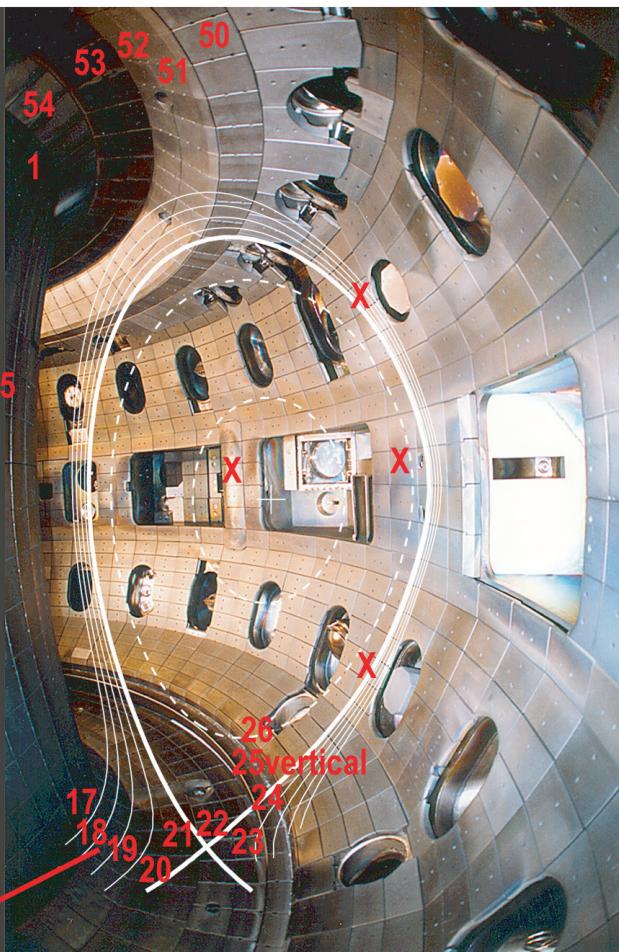
**Presented at the  
21st IAEA Fusion Energy Conference  
Chengdu, China**

**October 16–21, 2006**

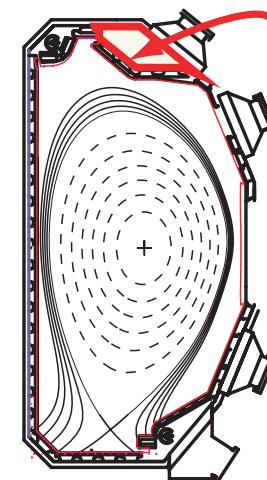
## Carbon deposition studies on the DIII-D Tokamak



Tritium on bottom of Tile : only tile with gap



Toroidally Symmetric  $^{13}\text{CH}_4$  injection



- Both L- and H-mode plasmas
- Tiles removed at end of run and analyzed with two different NRA analysis techniques

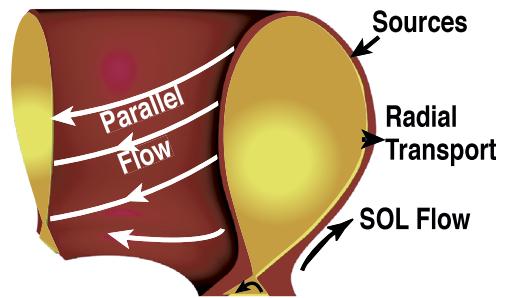
# Experimental Method

## Carbon Transport in All-Carbon DIII-D

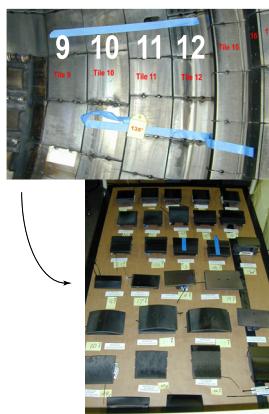
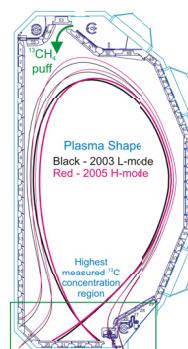
- Carbon Source (divertor or main walls)
- Physical or chemical sputtering
- SOL and divertor transport
- Where is the carbon re-deposited, and what is it's form?  
(C13 tracer experiments)

Relevant to ITER because the largest tritium inventory could be co- deposited with carbon layers that have large surface area.

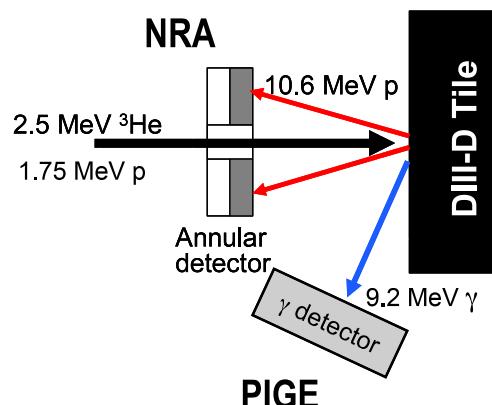
- $^{13}\text{CH}_4$  was injected from the upper divertor pump plenum into lower single null plasmas at a rate that did not significantly perturb plasma conditions.  
(toroidally symmetric injection & deposition).
- This plasma geometry, and location of injection far from the divertor, were chosen to simulate hydrocarbons originating from plasma interactions with the main chamber wall.
- Tiles were removed for nuclear reaction analysis of  $^{13}\text{C}$ .  
 $^{13}\text{C}(\text{He},\text{p})^{15}\text{N}$  NRA Sandia National Laboratories  
 $^{13}\text{C}(\text{p},\gamma)^{14}\text{N}$  PIGE University of Wisconsin



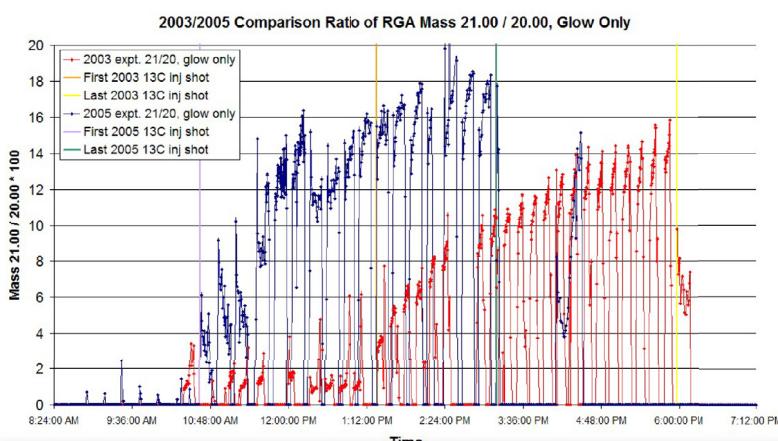
Divertor:  
Deposition & Erosion - Sources



- 3 rows of analysis on each tile at ~1 cm intervals (shown schematically)
- 1255 data points on 29 tiles



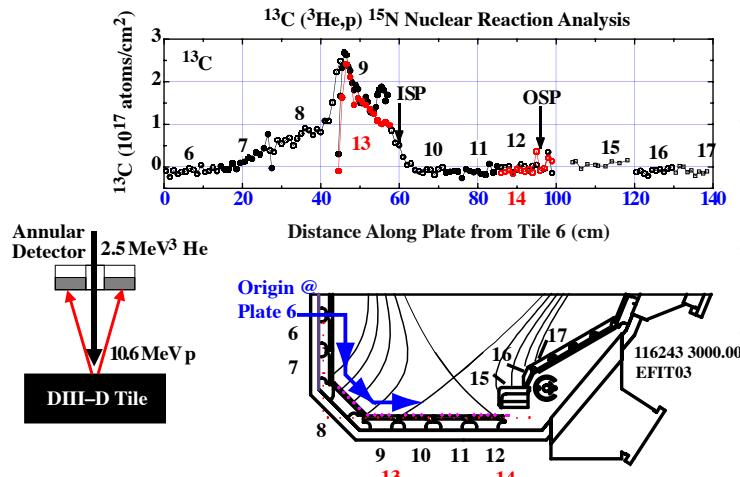
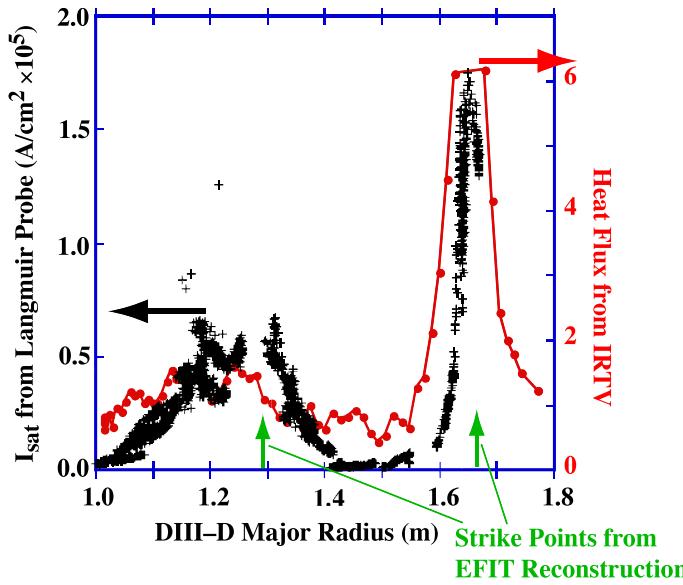
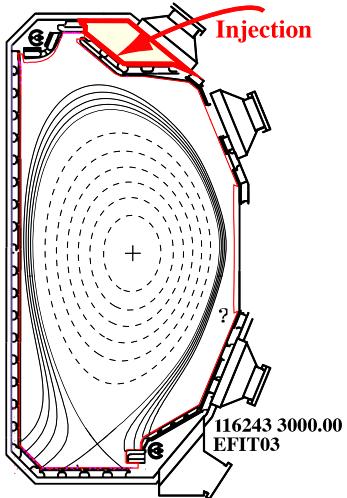
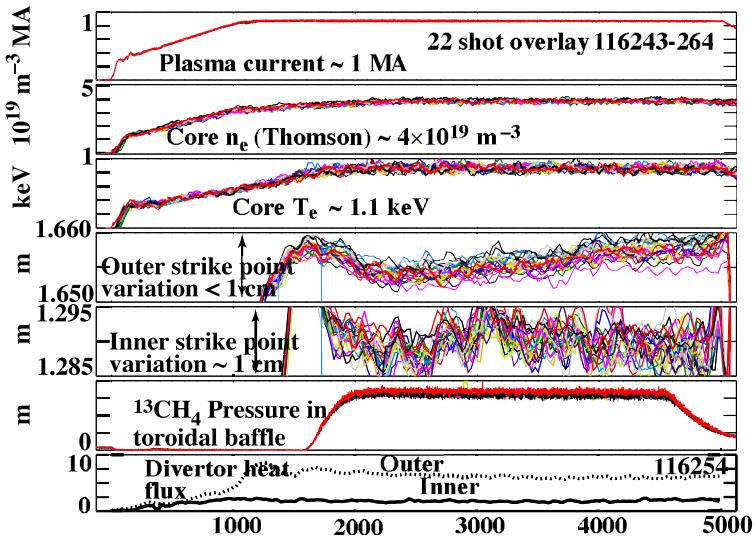
PIGE



He Glow Between Shots

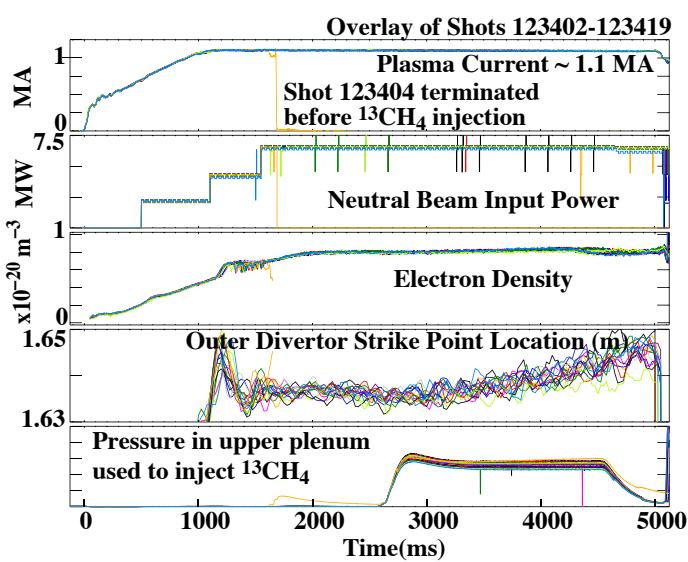
5 min glow between shots did not remove carbon deposits, as shown in RGA scans. Over 99% of injected methane remained.

# L-mode Results

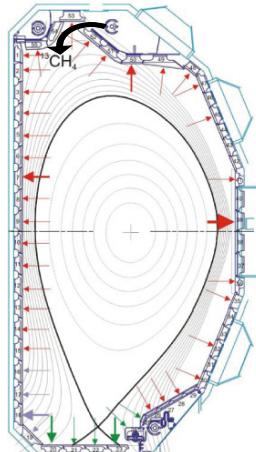


## Deposition Results

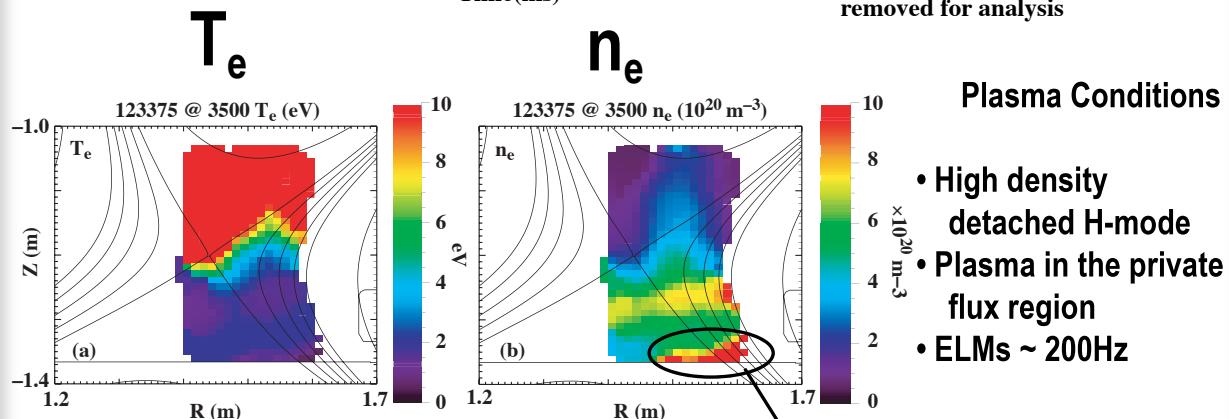
# H-mode Plasmas



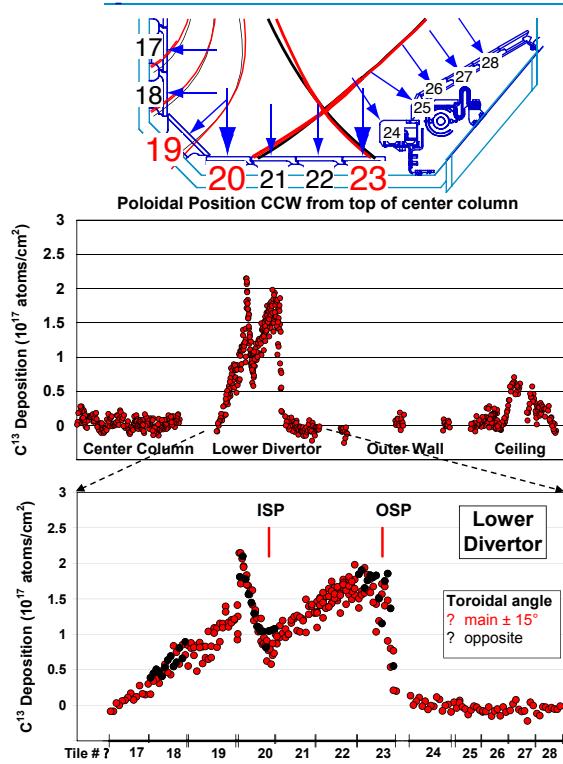
$^{13}\text{CH}_4$  injected into the top of LSN plasmas



Arrows indicate tiles removed for analysis



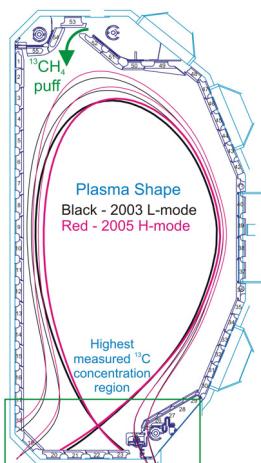
There may be instrumental problems with stray divertor light very close to the divertor plate



## Deposition Results

- Highest concentration of deposition near inner strike point
- Deposition in the private flux region
- Surprising amount of toroidal symmetry

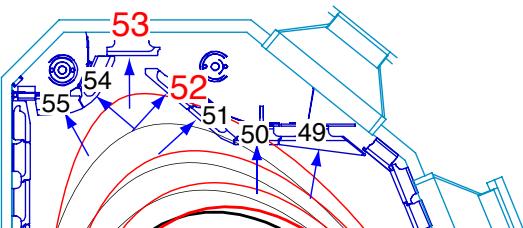
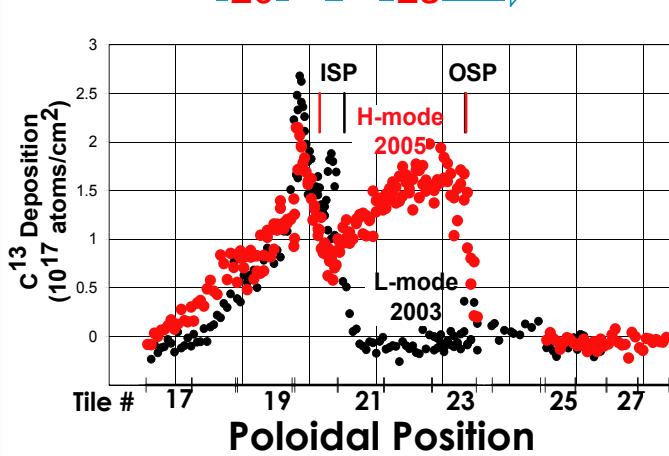
# Comparison of L- and H- Mode



Year	2003	2005
Plasma	SAPP L-Mode	ELMy H-mode
Inner divertor	detached	detached
Outer divertor	attached	detached
Repeat shots	22	17
Line average $n_e$ ( $10^{19} \text{ m}^{-3}$ )	3	8
NB Power (MW)	0.17	6.6
$^{13}\text{C}$ injection ( $10^{22} \text{ atoms}$ )	1.0	2.3
Tiles removed	29	64

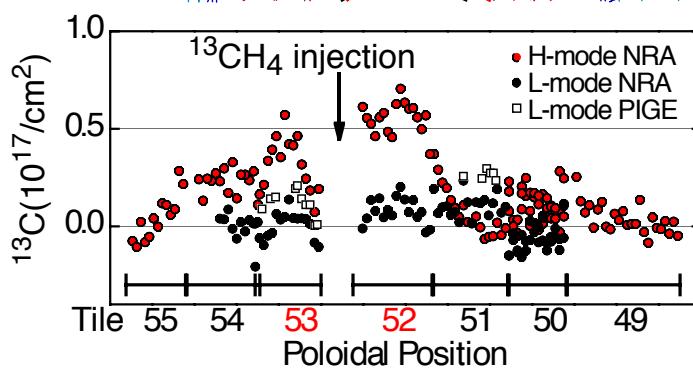
## Divertor Deposition Results

- Highest concentration of deposition near inner strike point
- Deposition in the private flux region for H-mode
- Both NRA and PIGE show low concentration at outer strike point
- About 35% deposited in divertor

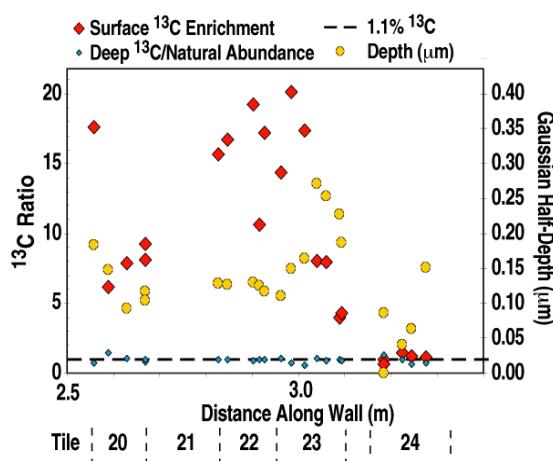


## Near Injection Zone

- About 10% deposited close to gas injection plenum
- Recall about twice as much methane injected in H-mode



# Carbon Surface Physics on DIII-D



## $^{13}\text{C}/^{12}\text{C}$ ratio

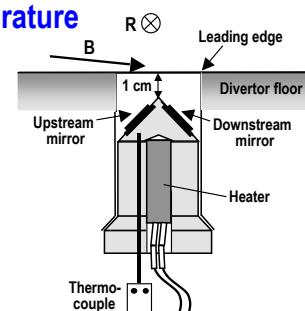
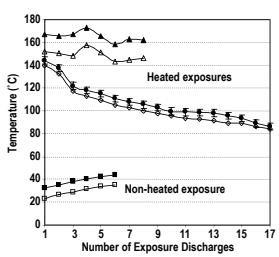
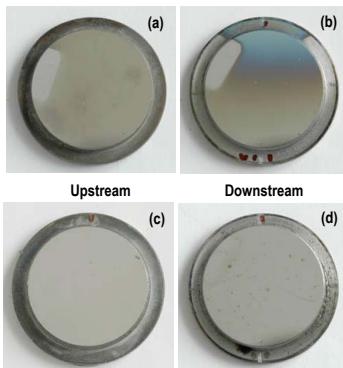
- Average ratio is 5 - 10
- Naturally occurring  $^{13}\text{C}$  is 1.1%
- $10 \times 1\% \sim 10\text{-}20\%$  of  $^{13}\text{C}$
- Still have 80%  $^{12}\text{C}$  - so even on the surface scale, this was a “trace” experiment

## Depth Scale of $^{13}\text{C}$

- Typical depth scale is 0.1-0.2 microns
- Deposition rate is 1-4nm/s (50-100s)
- ~200 grams of Carbon/hr of DIII-D

## DiMES Experiments at Elevated Temperature

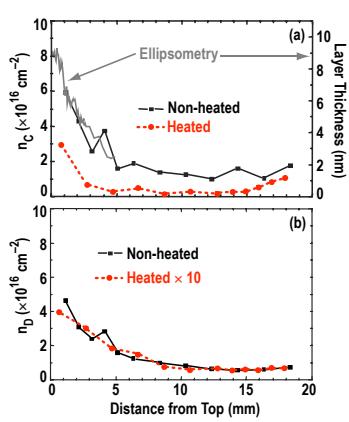
- Samples compared at two temperatures
  - a) & b)  $40^\circ$  (unheated)
  - c) & d)  $> 100^\circ$  (heated)
- No appreciable deposition at the elevated temperature



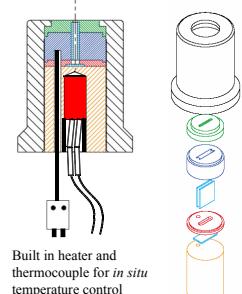
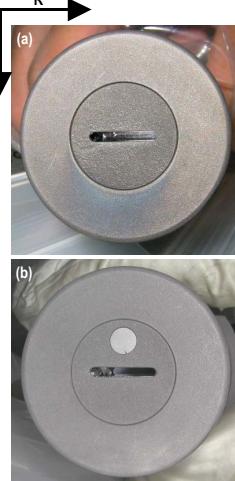
## WARM

## HOT

## Tile Gap Experiments

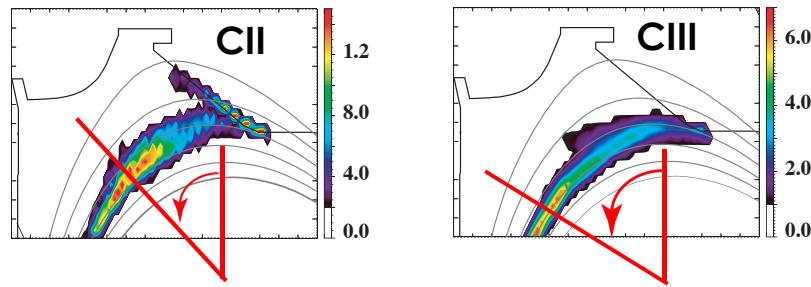


- Heated sample has 4x less carbon and 10x less deposition than non-heated sample.

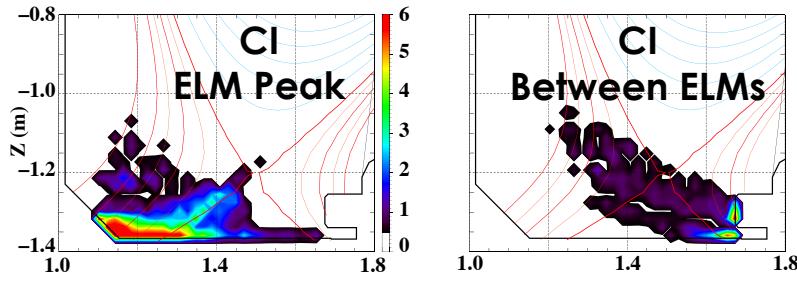


Built in heater and thermocouple for *in situ* temperature control

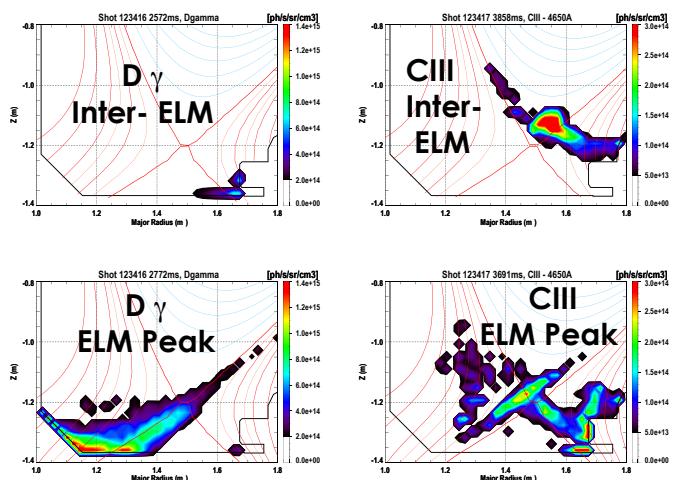
# Physics of Carbon Transport in DIII-D



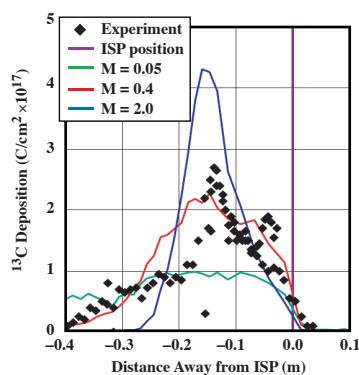
2-D tomographic reconstructions show SOL carbon flow of  $\sim 20$  km/s



Neutral carbon emission shows changes during the ELM cycle - private flux deposition?



- Inner plate emission can be ionization and/or recombination
- CIII emission moves from near x-point (detached) to the outer strike point during the ELM
- $J_{sat}$  increases near outer strike point at the ELM peak

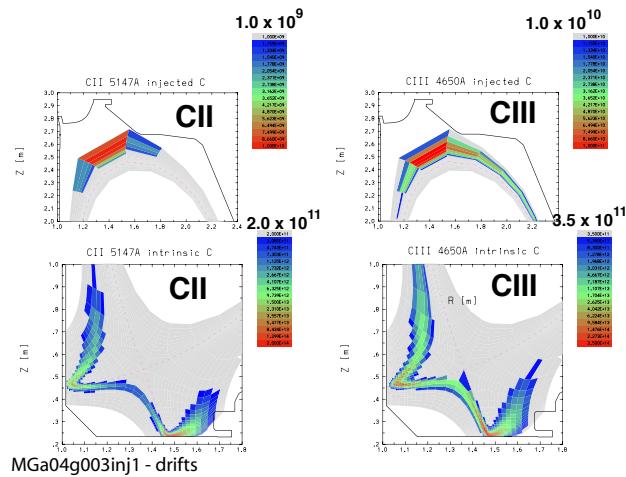


OEDGE Modeling follows carbon breakup:

- Methane breakup into fixed background plasma
- Inward radial shift required to match plate profile
- Uses ad-hoc carbon SOL flow towards inner divertor

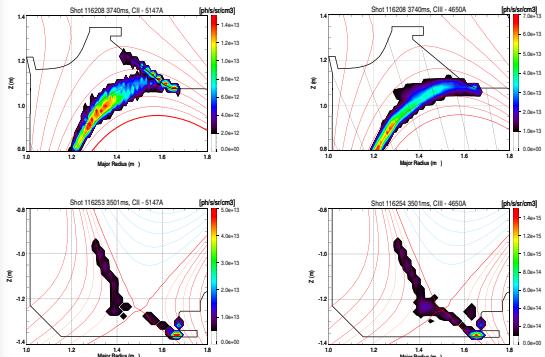
# UEDGE Modeling Compared to Data

## UEDGE with Drifts L-mode



## Drift Case

### DATA L-mode



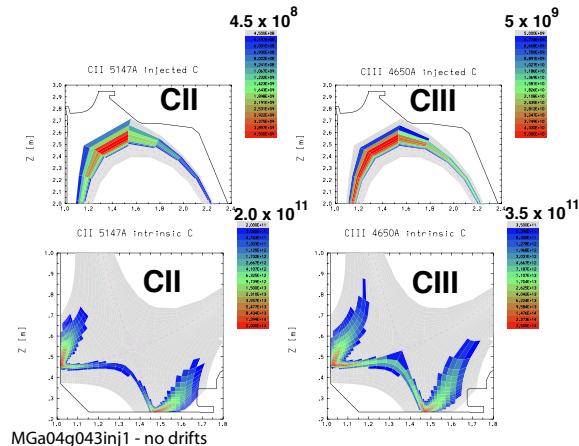
- Emission in crown is tilted towards the OUTER divertor in model
- Emission in divertor is more similar to data

UEDGE shows carbon transport follows deuterium flow.

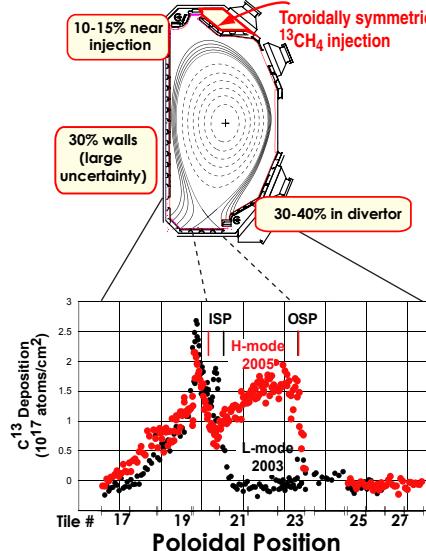
## No-Drift Case

- Emission in crown is tilted towards the inner divertor in both model and data
- No drift case - ISP attached in model, detached in data

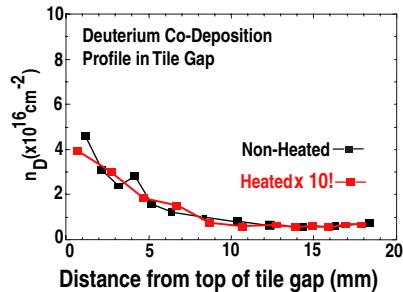
## UEDGE No Drifts L-mode



## Recent Experiments Suggests Tritium Uptake in Carbon Facing Surfaces May be Controllable



- DiMES experiments show large reduction in C and D deposition on heated materials



## Conclusions

- These experiments simulate transport of carbon entering the SOL in the main chamber
- Highest concentration of carbon deposition is in the divertor, localized near the inner strike point
- Carbon transport from upper crown to inner divertor by SOL flows of 20 km/s
- We can account for ~30-40% of the injected carbon in the divertor, about 10% at the injection region, and measurements of a few tiles in the main chamber suggest the remaining may be in a low-level deposit.
- Hydrocarbon breakup does not result in sufficient radial penetration to explain the profile at the plate - radial shift needed. Ad hoc flow of  $M \sim 0.4$  imposed in OEDGE modeling.
- Removal: Oxygen bake experiments are in progress, if promising, these could be carried out in DIII-D at the end of the next campaign.