

100% Noninductive Operation at High Beta Using Off-Axis ECCD

by
M. Murakami
in collaboration with

C.M. Greenfield,² M.R. Wade,¹ T.C. Luce,² J.R. Ferron,² H.E. St. John,² M.A. Makowski,³ M.E. Austin,⁴ S.L. Allen,³ D.P. Brennan,⁵ K.H. Burrell,² T.A. Casper,¹ J.C. DeBoo,² E.J. Doyle,⁶ A.M. Garofalo,⁷ P.Gohil,² I.A. Gorelov,² R.J. Groebner,² J. Hobirk,⁸ A.W. Hyatt,² R.J. Jayakumar,³ K. Kajiwara,⁵ C.E. Kessel,⁹ J.E. Kinsey,¹⁰ R.J. La Haye,² J.Y. Kim,² L.L. Lao,² J. Lohr,² J.E. Menard,⁹ C.C. Petty,² T.W. Petrie,² R.I. Pinsker,² P.A. Politzer,² R. Prater,² T.L. Rhodes,⁶ A.C.C. Sips,⁸ G.M. Staebler,² T.S. Taylor,² G. Wang,⁶ W.P. West,² L. Zeng,⁶ and the DIII-D Team

¹Oak Ridge National Laboratory, Oak Ridge, Tennessee, USA

²General Atomics, P.O. Box 85608, San Diego, California, USA

³Lawrence Livermore National Laboratory, Livermore, California, USA

⁴University of Texas at Austin, Austin, Texas, USA

⁵Oak Ridge Institute for Science Education, Oak Ridge, Tennessee, USA

⁶University of California at Los Angeles, Los Angeles, California, USA

⁷Columbia University, New York, New York, USA

⁸Max-Planck-Institut für Plasmaphysik, Garching, Germany

⁹Princeton Plasma Physics Laboratory, Princeton, New Jersey, USA

¹⁰Lehigh University, Bethlehem, Pennsylvania, USA

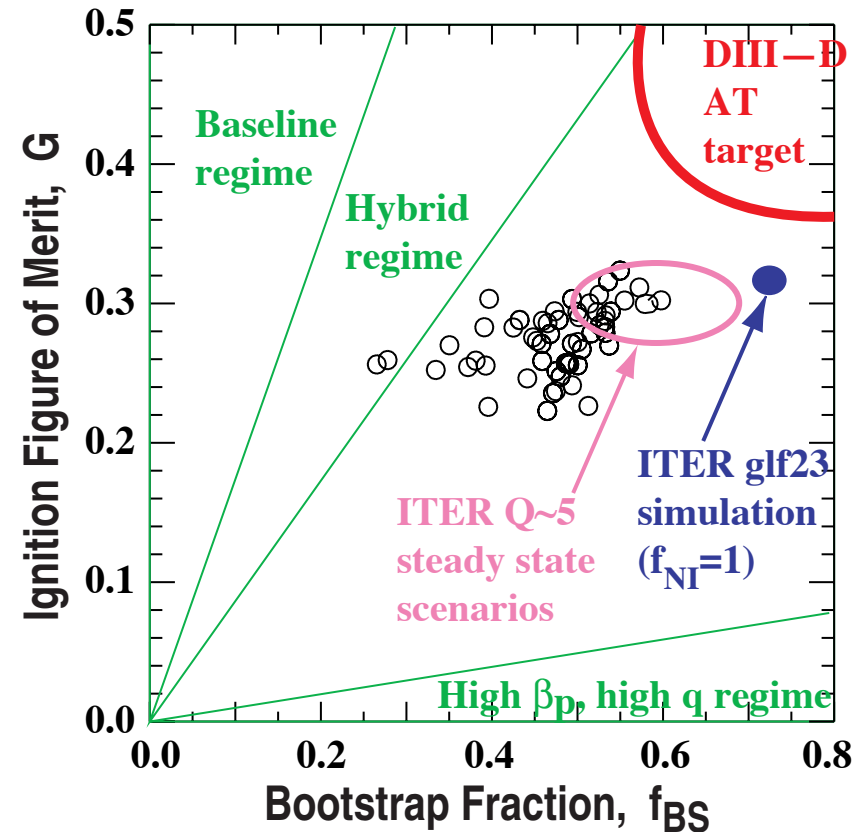
Presented at
20th IAEA Fusion Energy Conference
Vilamoura, Portugal

November 2, 2004

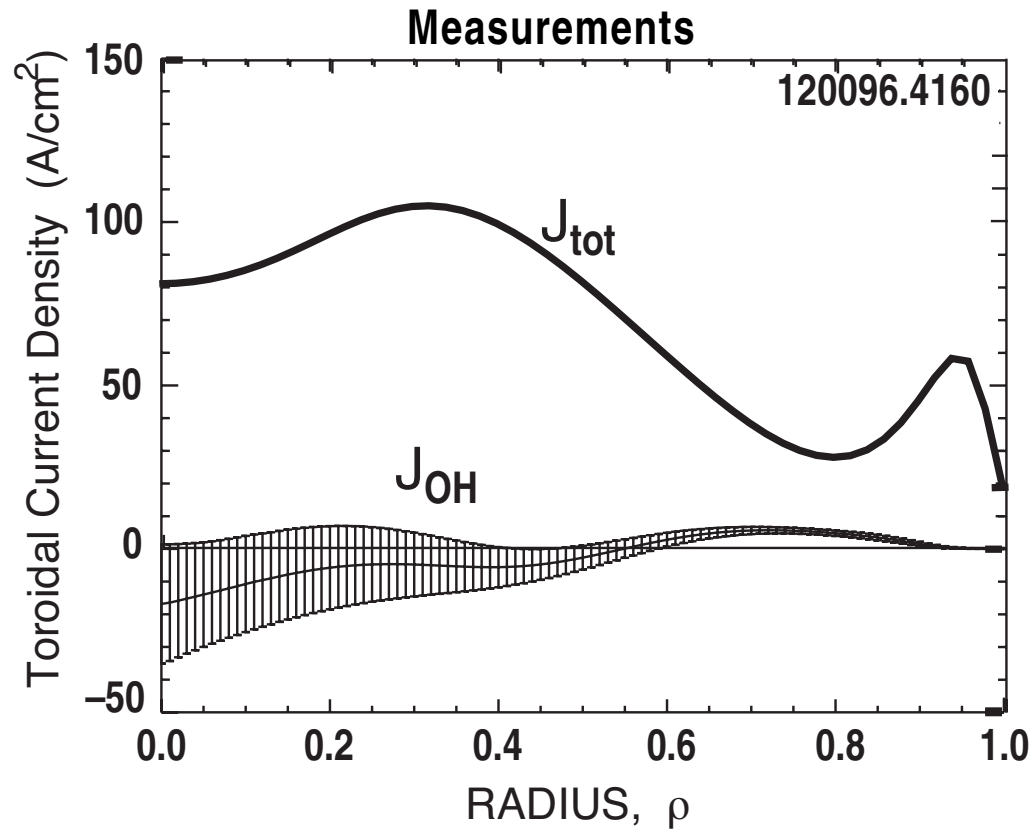


DIII-D AT PROGRAM GOAL: SCIENTIFIC BASIS FOR STEADY STATE, HIGH PERFORMANCE OPERATION

- **Steady-state**
 - 100% noninductive
 - High bootstrap current fraction $f_{BS} \propto \beta_p$
- **Maintaining sufficient fusion gain with reduced engineering parameters**
 - High β_T
 - High τ_E
 - ⇒ High Normalized fusion performance $G = \beta_N H/q^2$
- **DIII-D AT experiments have demonstrated performance required for ITER steady state scenario**



100% NONINDUCTIVELY DRIVEN PLASMAS WITH GOOD CURRENT DRIVE ALIGNMENT

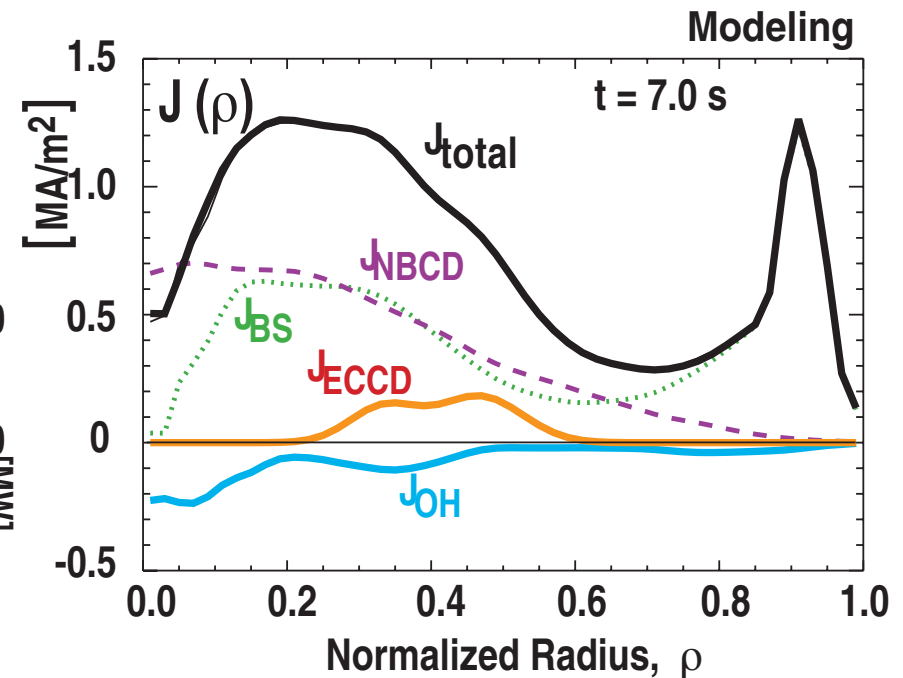
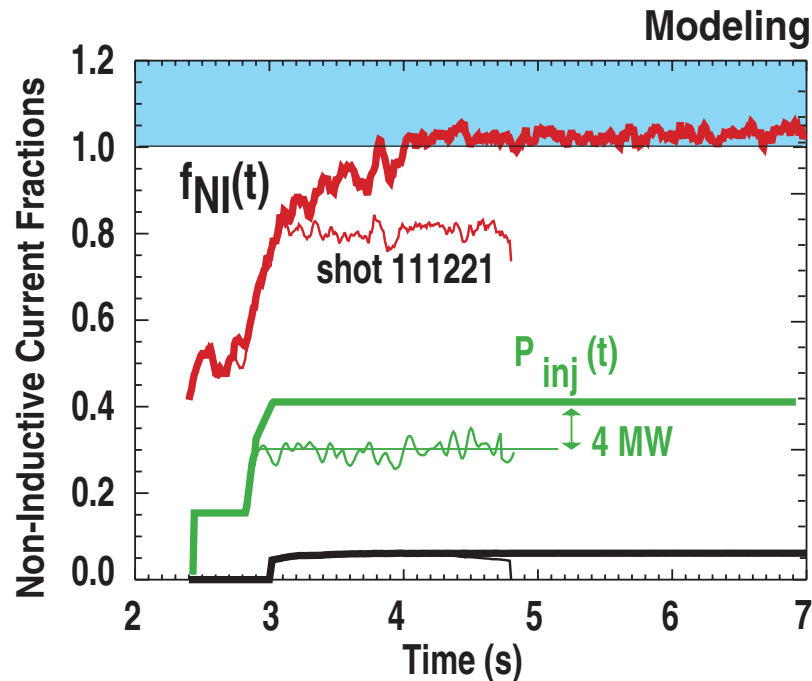


- $f_{NI} = 1 - f_{OH}$; $J_{OH} = \sigma_{neo} E_I \propto \sigma_{neo} \partial \Psi_{pol} / \partial t$
- $f_{OH} = 0.5\%$, $f_{NI} = 99.5\%$
- $\beta_T = 3.5\%$, $\beta_N = 3.6$, $q_{95} = 5.4$

CRITICAL ISSUES COVERED IN THIS TALK

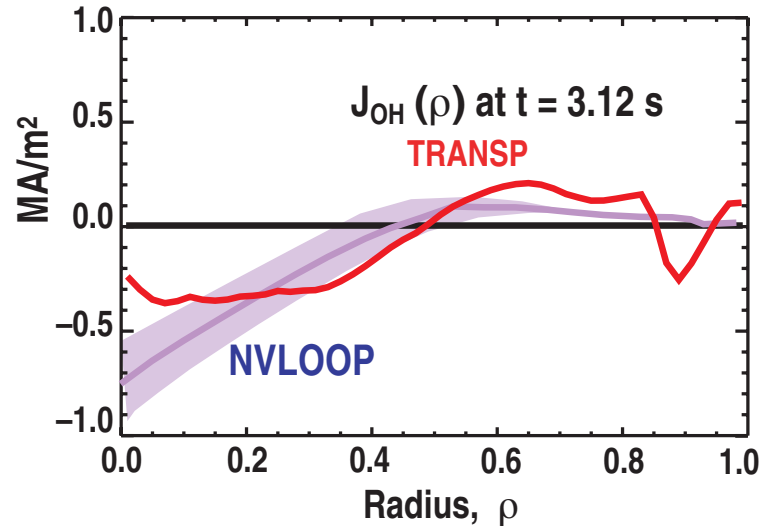
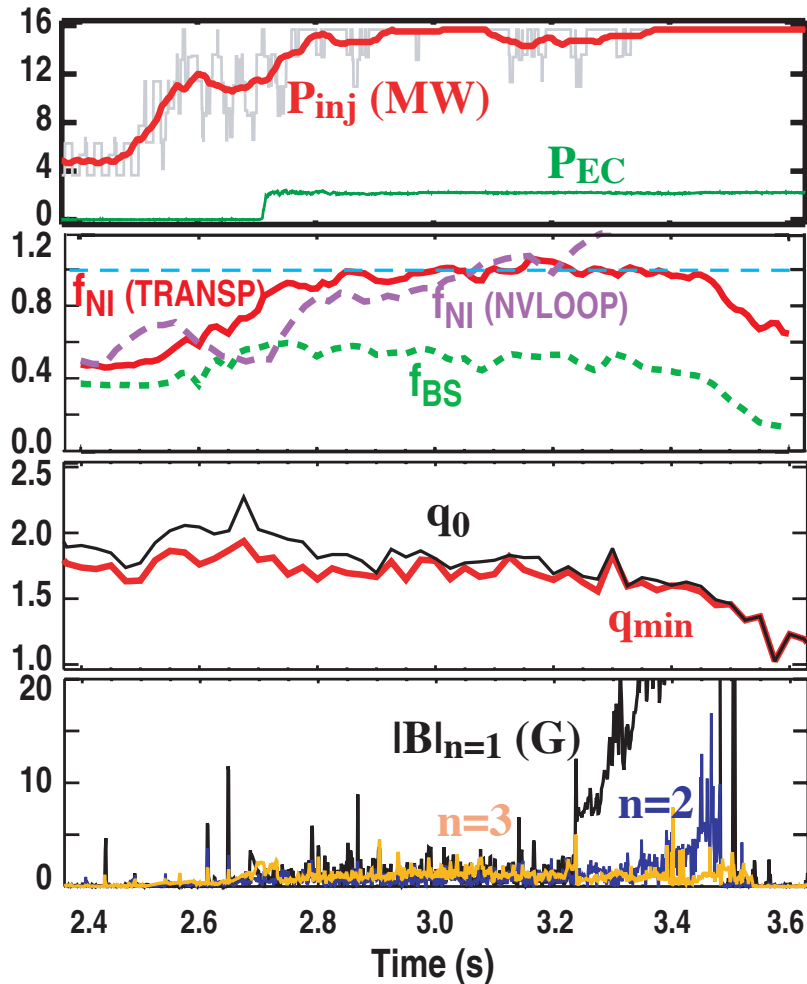
- **Self-consistent solutions for full noninductive, high performance operation requires:**
 1. $f_{NI} = 100\%$
 2. Good current drive alignment
 3. Pressure profile evolution stable for ideal MHD and NTMs
 4. Integration \Rightarrow Current profile stops evolving ($E_{||} \approx 0$ everywhere)
- **Modeling validated against experiments is applied to**
 - Projection for longer sustainment of $f_{NI}=100\%$ in DIII-D
 - ITER steady-state *AT* scenario

PREDICTIVE SIMULATIONS INDICATE PREVIOUS ECCD DISCHARGE COULD BE EXTENDED TO 100% NONINDUCTIVE WITH INCREASED NBI POWER



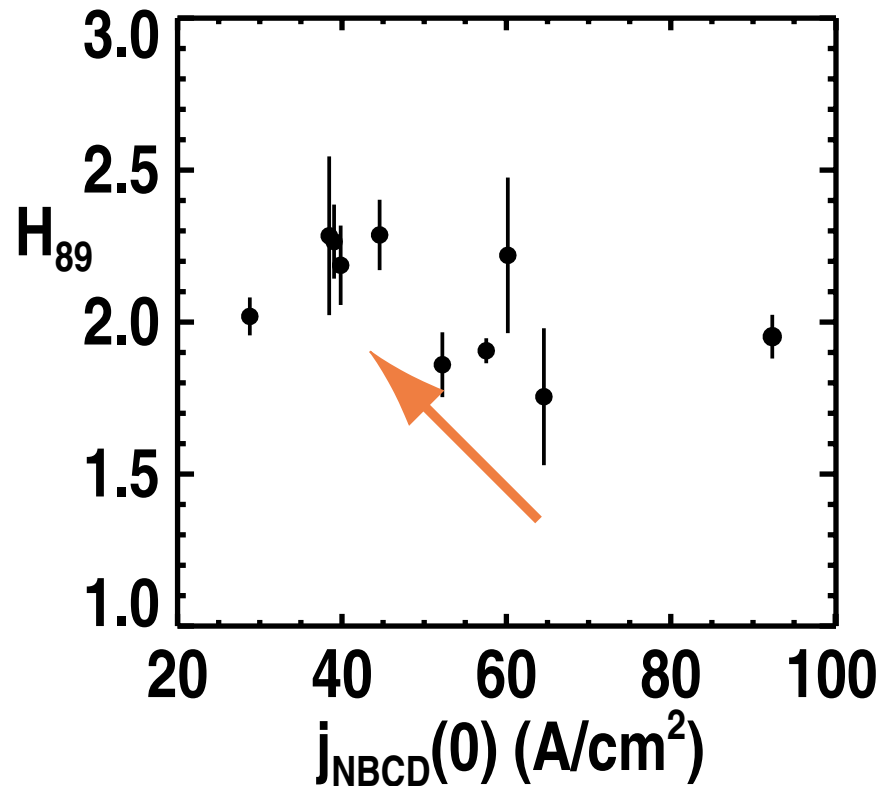
- Two transport models produce consistent results:
 - Scaled experimental transport coefficients
 - Recalibrated GLF23
- Modeling has become essential tool for the experimental program
 - Used both to plan and interpret experiments

NEUTRAL BEAM OVERDRIVE NEAR THE AXIS DECREASES q_0 , RESULTING IN *NTMs*



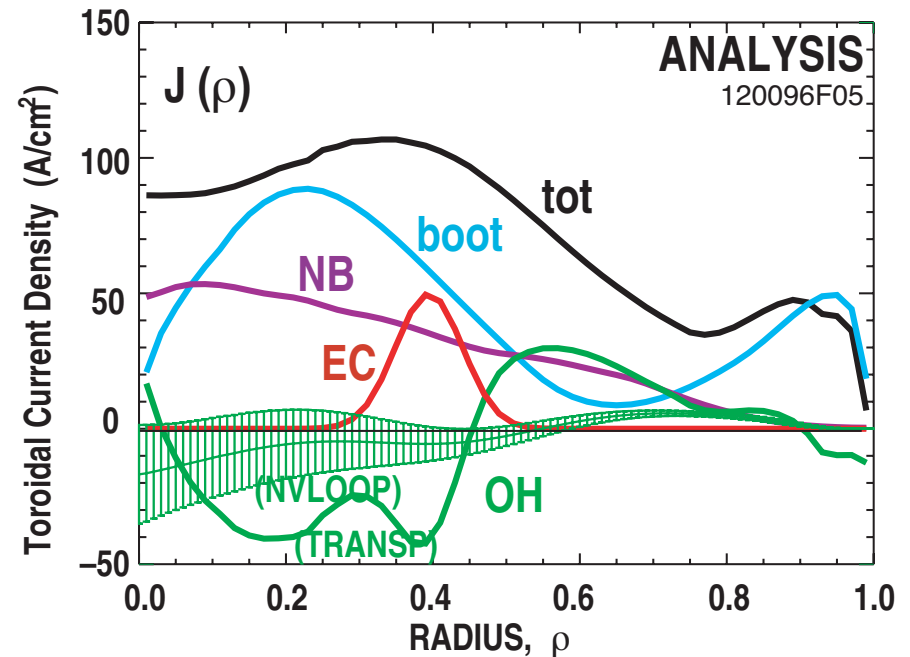
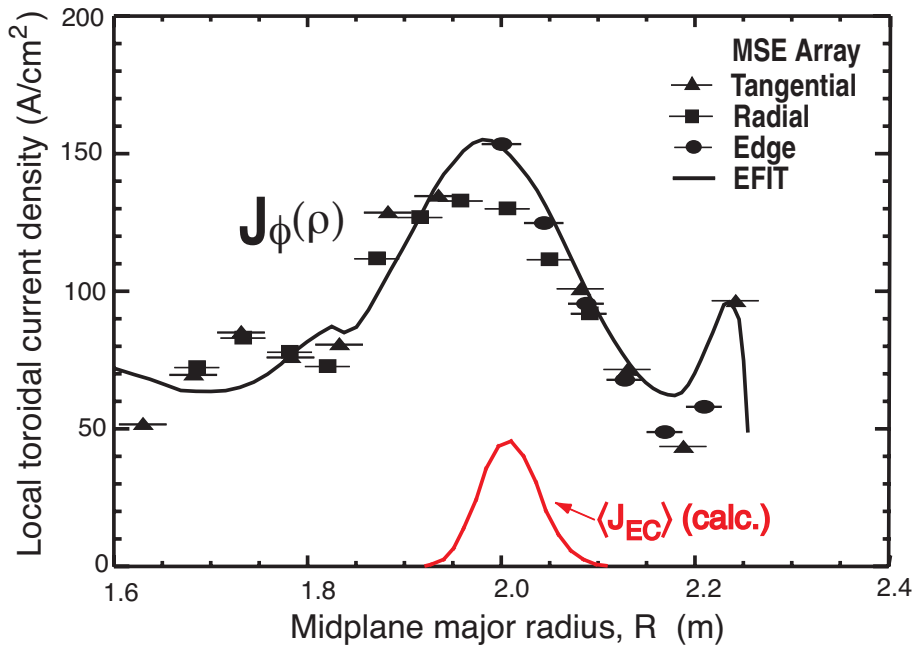
- Achieved net $f_{NI} \approx 100\%$ with $\beta_N \approx 3.5$, $\beta \approx 3.6\%$
- Net Ohmic current is zero, but local Ohmic current is NOT zero
- Confinement somewhat degraded (large P_{NB} demand) in these discharges
 - Rotation velocity often slower
 - Flatter q profiles ... often more monotonic

IMPROVED CONFINEMENT RESULTS IN REDUCED NEUTRAL BEAM CURRENT DRIVE NEAR THE AXIS



- Confinement improvement in recent experiments is attributed to:
 - Optimized non-axisymmetric field feedback
 - Slightly negative central shear

WITH IMPROVED CONFINEMENT, $f_{NI}=100\%$ ACHIEVED WITH GOOD CD ALIGNMENT

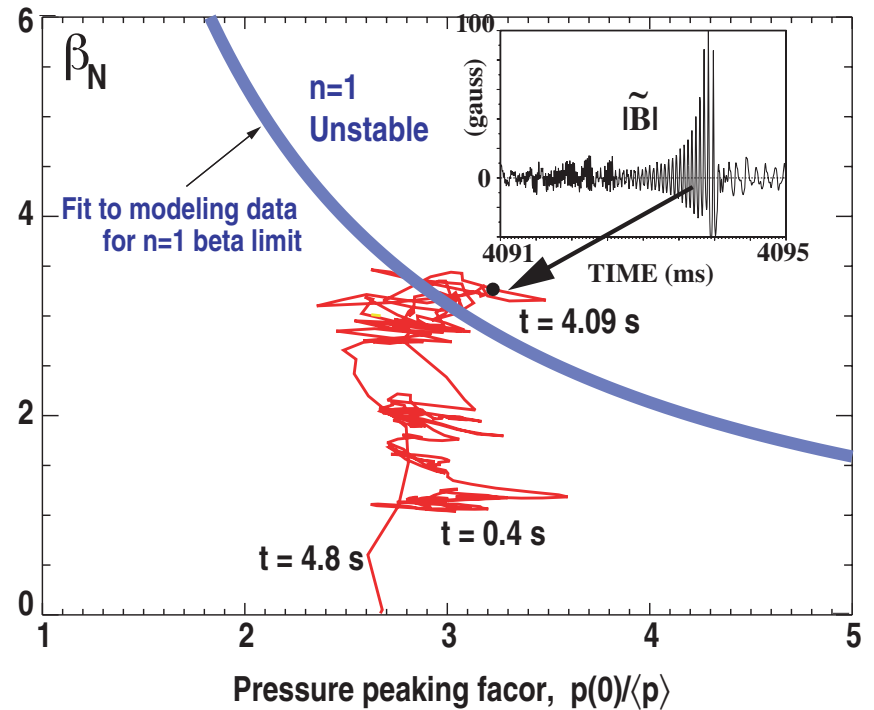
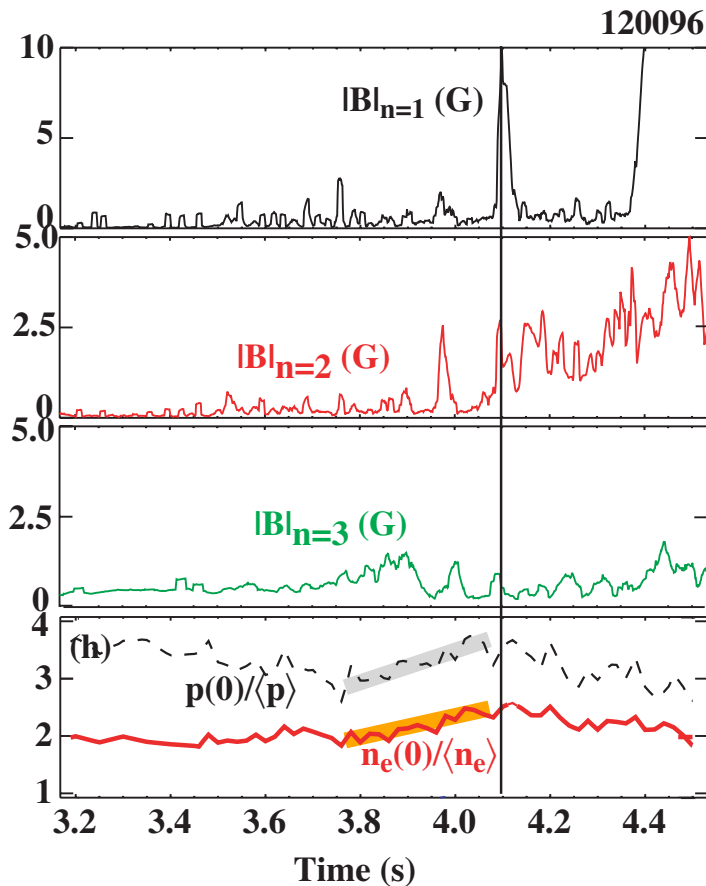


- Measure of CD alignment: $\xi_{tot} = 1 - \frac{\int (n_e/T_e) |J_{OH}| dA}{\int (n_e/T_e) |J_{tot}| dA}$

Case:	Present	NB-overdrive
ξ_{tot}	0.94	0.90

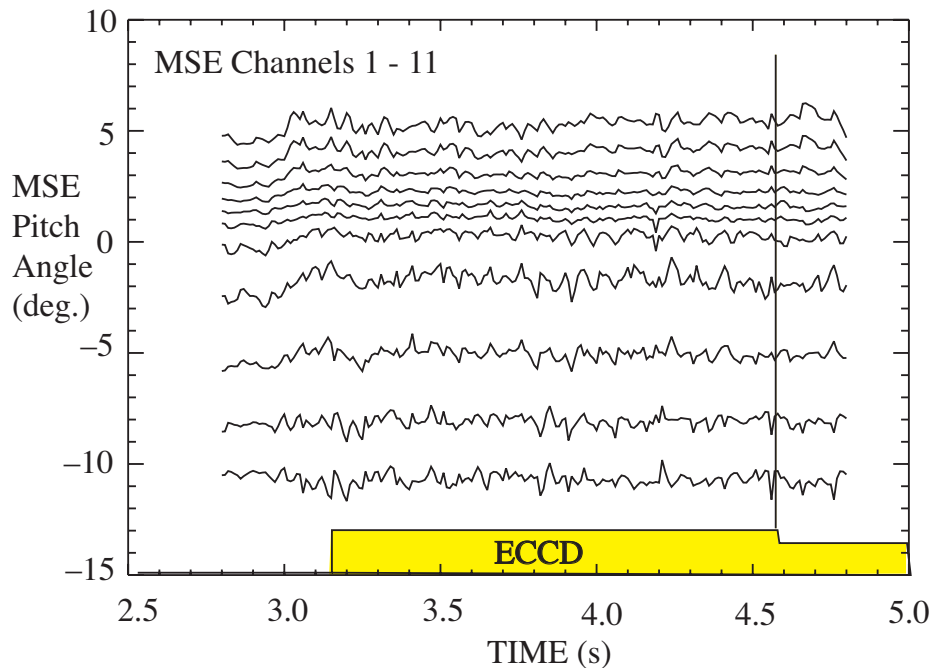
- $f_{BS}=59\%$ $f_{NB}=31\%$ $f_{EC}=8\%$ $f_{NI}=98\%$

PRESSURE EVOLUTION RESULTED IN $n=1$ FAST GROWING MODE WHICH TRIGGERED $n=1$ NTM

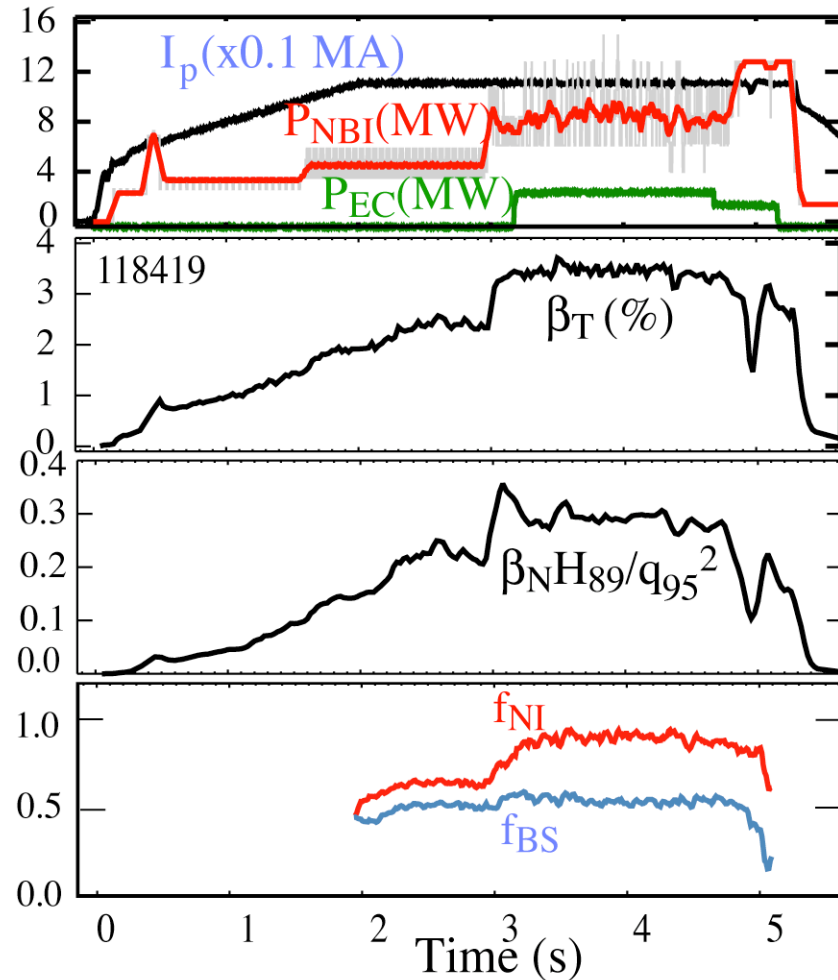


- $n=1$ ideal instability caused by pressure peaking primarily due to density peaking

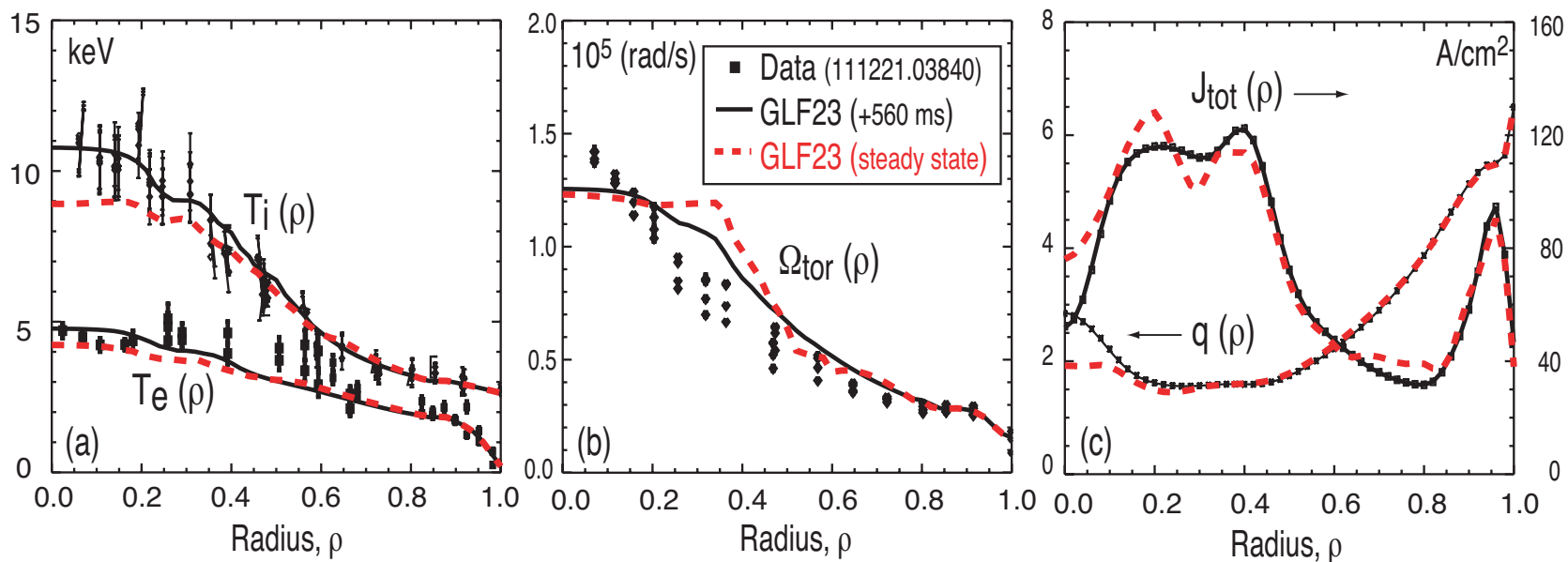
NEARLY FULL NONINDUCTIVE, STATIONARY DISCHARGE OBTAINED, LIMITED ONLY BY GYROTRON PULSE LENGTH



- MSE signals stationary
 $\Rightarrow J_{\phi}(\rho)$ stopped evolving
- $f_{NI} \sim 90\%$ for $1 \tau_R (=1.8s)$
- $\beta_T = 3.7\%$, $\beta_N = 3.5$, $q_{95} = 5.1$
- $G = \beta_N H / q^2 = 0.3$ with $f_{BS} = 63\%$

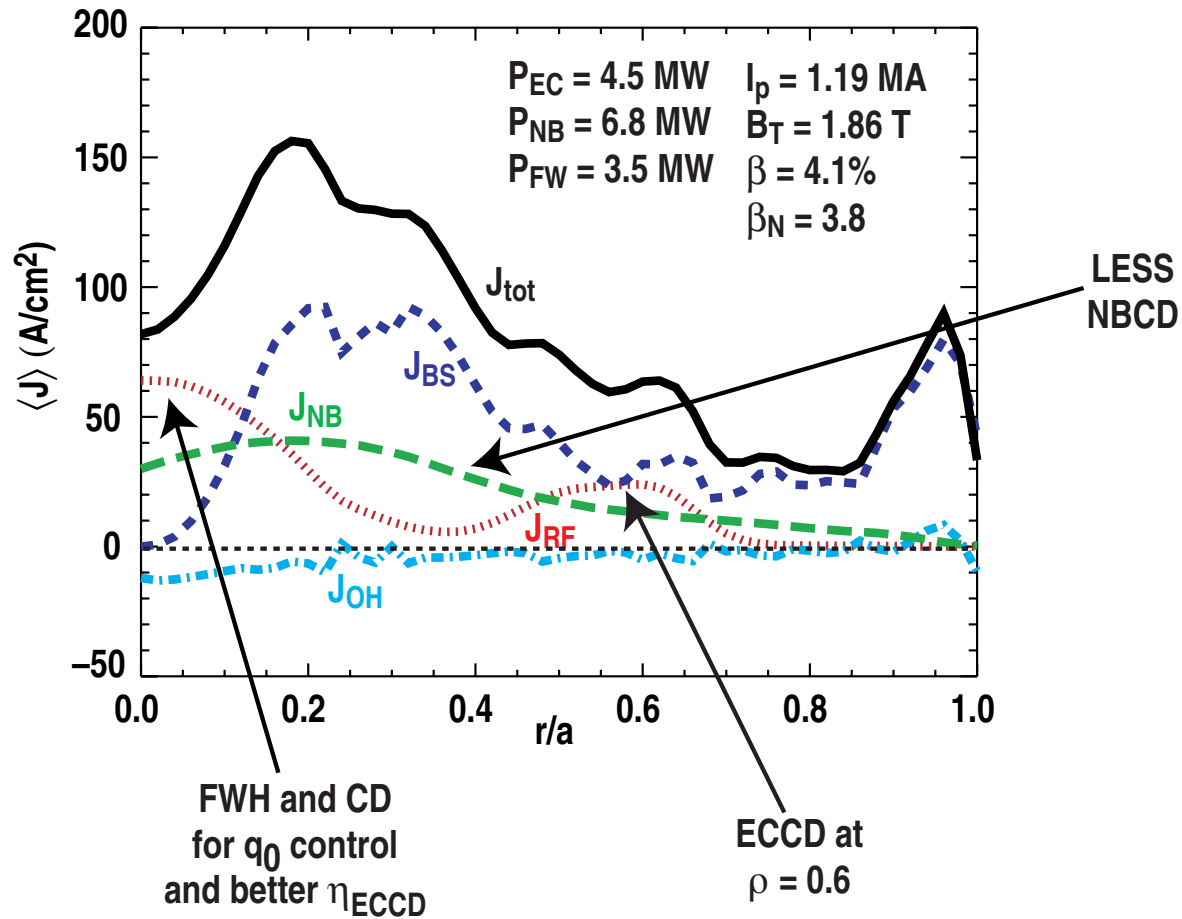


GLF23/ONETWO CAN REPRODUCE EXPERIMENTAL PROFILES REASONABLY WELL, AND ALSO CAN PREDICT STEADY STATE PERFORMANCE IN TOKAMAKS



- Good coupling between experiment and modeling
- Numerical advance (global convergence technique) incorporated into ONETWO allows prediction of steady state in one step (without time stepping calculation)

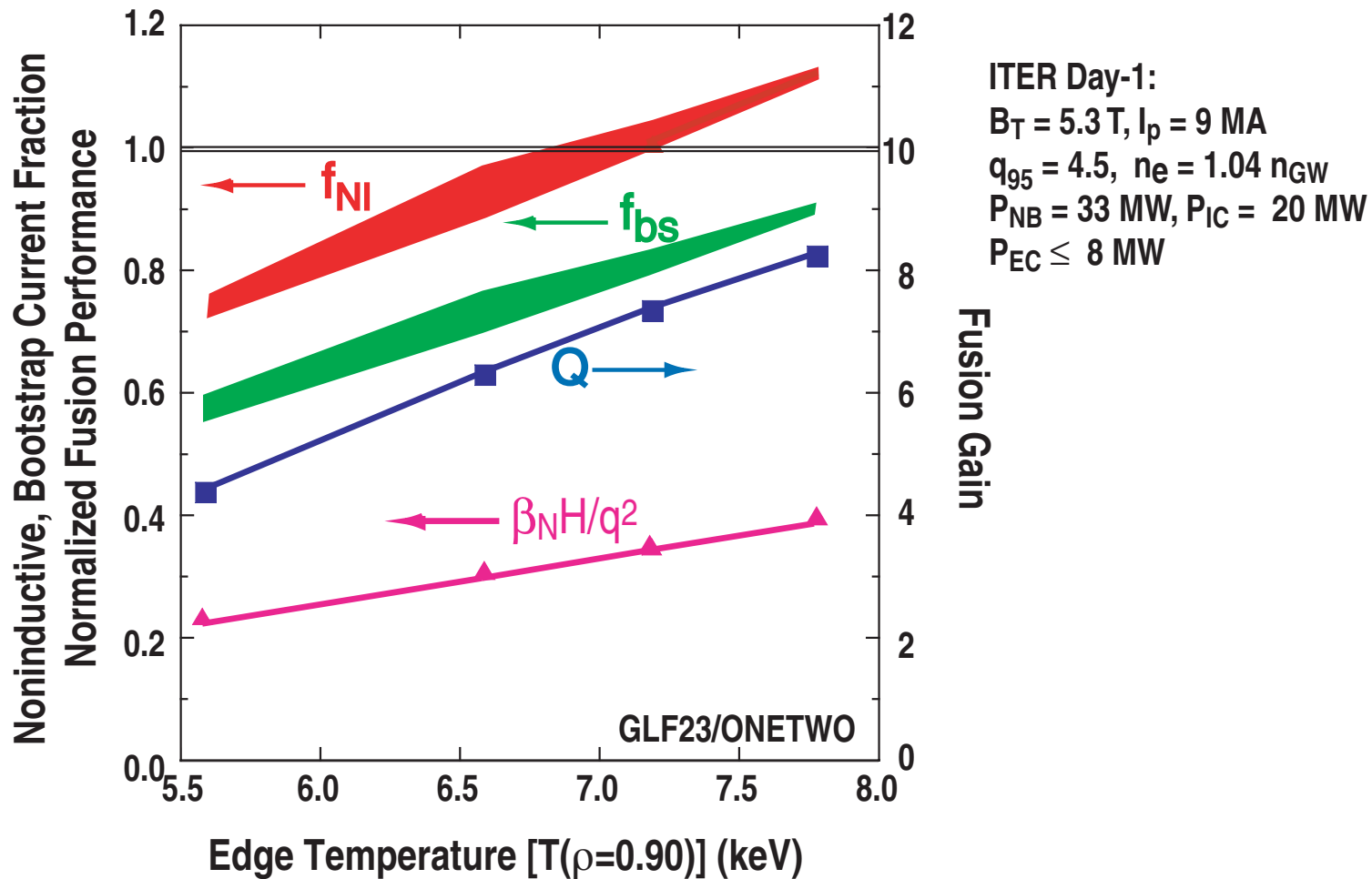
GLF23 MODELING INDICATES THAT STEADY STATE OPERATION IS POSSIBLE WITH β VALUES CONSISTENT WITH STABILITY LIMITS



- Modeling uses hardware improvements planned for DIII-D:
 - Better control of $J(\rho)$ and $p(\rho)$ at high beta with more EC and FW power with long duration
 - Advanced plasma control system

MODELING APPLIED TO ITER AT SCENARIO PREDICTS

$f_{NI} = 100\%$ FEASIBLE WITH $Q > 7$



- Stiff transport model \Rightarrow core performance related to edge \Rightarrow edge temperature scan
- Emphasizes importance of understanding the edge pedestal in AT plasmas
- More details will be discussed by W. Houlberg [IT/P3-33]

CONCLUSIONS

- 100% noninductively driven plasmas with good *CD* alignment at $\beta_T \leq 3.6\%$ and $\beta_N \leq 3.5$ for up to one current relaxation time
- With good coupling between experiment and modeling, progress has been made in several important areas:
 - Current drive alignment
 - Current profile stationary over one current relaxation time
 - Challenge: Control of pressure profile evolution to avoid ideal MHD and *NTM* instabilities to further extend high performance phase
- Future plans include:
 - Better control of $J(\rho)$ and $p(\rho)$ at high beta with more EC and FW power with long duration
 - Advanced plasma control system
- The scientific basis being developed on DIII-D is leading to increased confidence in establishing steady-state scenarios for ITER and beyond