

# Minimal Plasma Response Models for Design of Tokamak Equilibrium Controllers with High Dynamic Accuracy

D.A. Humphreys, J.A. Leuer, M.L. Walker  
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**Minimal Plasma Response Models for Design of Tokamak Equilibrium Controllers with High Dynamic Accuracy<sup>1</sup>**

D.A. HUMPHREYS, M.L. WALKER, J.A. LEUER, General Atomics  
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<sup>1</sup>Supported by U.S. DOE Contract DE-AC03-99ER54463.

Prefer Oral Session  
 Prefer Poster Session

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Special instructions: DIII-D Poster Session 2, immediately following JA Leuer

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## Abstract

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We describe a model of linearized plasma shape and position response which is based on low poloidal mode number ( $m \leq 2$ , approximately vertical and major radial) displacements of the plasma current distribution. The model introduces minimal plasma degrees of freedom while providing sufficient accuracy for high performance controller design. The effects of significant variation in plasma poloidal beta, internal inductance, and separatrix configuration are taken into account. Models which can predict plasma shape and position variation with reasonable accuracy are particularly important for design of dynamic controllers in devices with significant variation in auxiliary heating input power and plasma shape - conditions common in the DIII-D tokamak. Model predictions are validated using experimental response data from DIII-D. Application of the plasma response model to design of multivariable dynamic plasma controllers recently implemented on DIII-D is described.



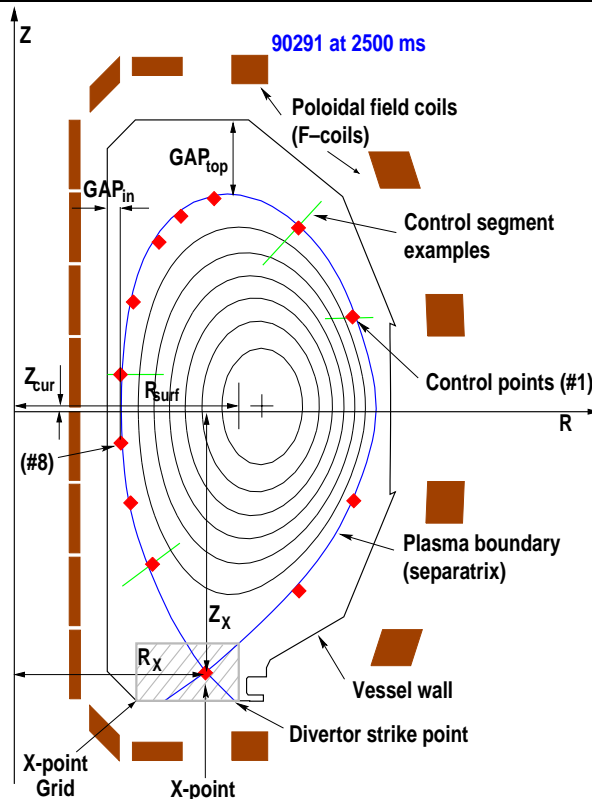
## OVERVIEW

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- Accurate models of dynamic plasma response are needed for design of multivariable model-based controllers.
- Validated, analytically-based plasma models:
  - Reveal important/unimportant physics effects;
  - Improved physics understanding enables model extension to:
    - ⇒ New operating regimes in existing devices;
    - ⇒ Next-generation tokamak experiments.
- Minimal complexity, minimal order dynamic plasma response models:
  - Have been developed and extensively validated against DIII-D experiments with varying  $\beta_p$  and  $I_i$ ;
  - Enable rapid calculation and controller design.
- Validated minimal plasma response models have been used to design successful dynamic shape controllers for DIII-D. [See poster JP1.33, this session]



# Isoflux Shape Control System on DIII-D Regulates Flux at Specified Control Points on Boundary



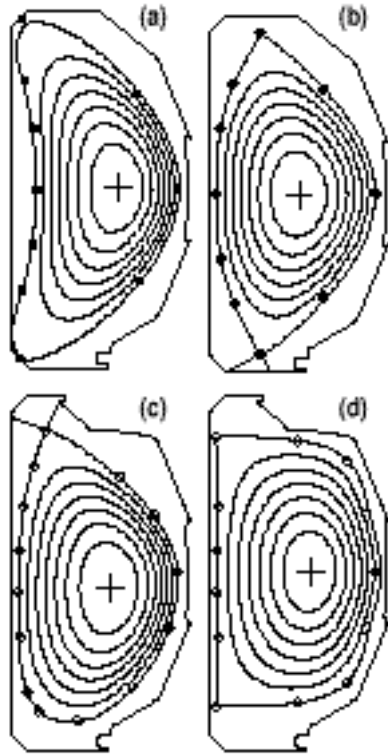
- **Real time EFIT** (Equilibrium **FIT**ing code) calculates flux and field very accurately near plasma boundary.
- **Controlled parameters** are:
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## Realtime Equilibrium Reconstruction Provides High Accuracy Required by Isoflux Control Scheme

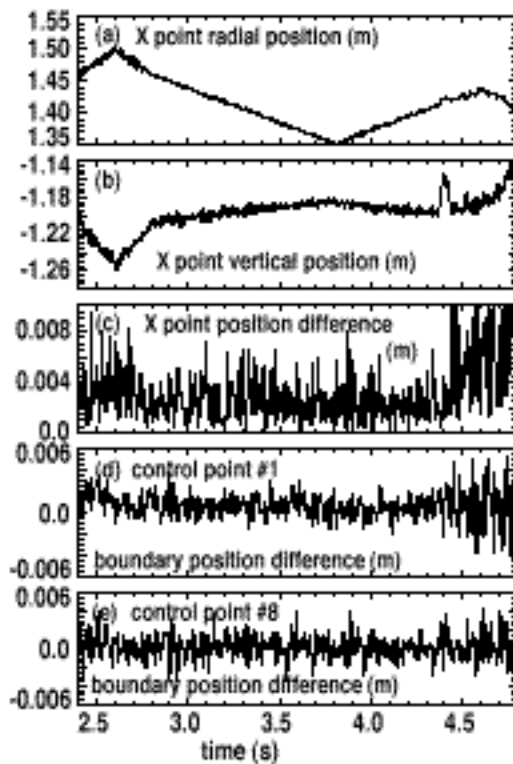
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- **Realtime version of EFIT** algorithm provides solution to Grad-Shafranov tokamak equilibrium equation consistent with magnetic measurements:
  - ⇒ Best fit to magnetic measurements from 39 flux loops, 31  $B_p$  probes, Rogowski loop measurement of  $I_p$ , 16 Motional Stark Effect measurements of plasma current density, 18 shaping coil currents.
- Realtime algorithm performs **iterations on 2 timescales**:
  - “**Inner loop**” iteration every **1-1.5 ms** (find  $J_t$  based on measurements and *old* normalized flux  $\bar{\psi}$  ) ;
  - “**Outer loop**” iteration (calculate fit weights, update  $\bar{\psi}$  ) performed every **20-30 ms**:
- Using old  $\bar{\psi}$  until updated value available still produces sufficiently accurate reconstruction:
  - ⇒ **Boundary determined within ~ few mm**;
  - ⇒ **Flux, field on grid determined within ~ 0.1%**;

# Isoflux Control Scheme Used With Realtime EFIT Provides High Shaping Flexibility, High Shape Control Accuracy



⇐ Discharges produced include:  
a) Crescent/bean-shaped;  
b) Low triangularity DN;  
c) Upper SN;  
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## **Motivation: New DIII-D Control System Will Use Model-Based Approach to Controller Design**

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- **Design philosophy:**
  - **Develop and validate linearized and nonlinear models** of plasma/conductor/power supply responses;
  - **Design controllers** based on minimal linear models;
  - **Test and optimize** controller performance with linear and nonlinear model simulations;
- **Advantages:**
  - **Controllers allow significant variation** in plasma equilibrium without switching to new controller;
  - **New equilibrium/shape development** without consuming valuable experimental time on tokamak;
  - **Improved control performance:** accuracy/precision, dynamic response, disturbance/noise rejection.
  - **Ability to balance tokamak constraints** with desired control.
- **Disadvantages:**
  - Requires reasonably **accurate system response models**;
  - Possible increase in difficulty of tuning control algorithms “by hand” between shots (but with theoretical reduction in need for such tuning);





## Plasma Response Model Based on Rigid Plasma Displacements, Simple Force Balance Relations

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- Goal: Use “**minimally complex**” models which provide sufficiently accurate representation of experiment.
- Simple **radial force balance**:

$$B_{zm}^{appl} = B_z^{Shafranov} = -\frac{\mu_0 I_p}{4\pi R_m} \Gamma(R_m, \beta_p, l_i)$$

$$\equiv -\frac{\mu_0 I_p}{4\pi R_m} \left( \ln \left| \frac{8R_m}{a\sqrt{\kappa}} \right| + \frac{2\kappa}{\kappa^2 + 1} \beta_p + \frac{l_i}{2} - 1.5 \right)$$

- Simple **vertical force balance**:

$$\mathbf{0} = -2\pi R_m I_{p0} \frac{\partial B_R}{\partial z} \delta z + I_{p0} \frac{\partial M_{pc}}{\partial z} \delta I_c + \frac{\partial F_z}{\partial \beta_p} \delta \beta_p + \frac{\partial F_z}{\partial l_i} \delta l_i$$

- **Plasma response objects** from perturbed force balance:

$$R_m \text{ Current-Resp } \frac{\partial R_m}{\partial I_c} \equiv \left[ \frac{B_{zm0}}{R_0} n + \frac{\mu_0 I_{p0}}{4\pi R_0^2} (\Gamma_0 - 1) \right]^{-1} \frac{\partial B_{zm}}{\partial I_c} + \frac{\partial R_m}{\partial z} \frac{\partial z}{\partial I_c}$$

$$R_m \text{ } I_p \text{-Response } \frac{\partial R_m}{\partial I_p} \equiv \left[ \frac{B_{zm0}}{R_0} n + \frac{\mu_0 I_{p0}}{4\pi R_0^2} (\Gamma_0 - 1) \right]^{-1} \frac{\mu_0 \Gamma_0}{4\pi R_0}$$

$$\frac{\partial R_m}{\partial \beta_p} \equiv \left[ \frac{B_{zm0}}{R_0} n + \frac{\mu_0 I_{p0}}{4\pi R_0^2} (\Gamma_0 - 1) \right]^{-1} \frac{\mu_0 I_{p0}}{4\pi R_0} \left( \frac{2\kappa}{\kappa^2 + 1} \right) + \frac{\partial R_m}{\partial z} \frac{\partial z}{\partial \beta_p}$$

$$\frac{\partial R_m}{\partial l_i} \equiv \left[ \frac{B_{zm0}}{R_0} n + \frac{\mu_0 I_{p0}}{4\pi R_0^2} (\Gamma_0 - 1) \right]^{-1} \frac{\mu_0 I_{p0}}{8\pi R_0}$$

$$Z_c \text{ Curr-Resp } \frac{\partial z}{\partial I_c} \equiv \frac{(\partial M_{pc} / \partial z)}{-2\pi B_{zJ0} n_J}, \quad Z_c \beta_p \text{-Resp } \frac{\partial z}{\partial \beta_p} \approx \frac{\int R B_{R0} J_\phi dA_p}{\beta_{p0} B_{zJ0} n_J}$$

## Rigid Radial and Vertical Plasma Displacement Assumption Closes Plasma Response Equations

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- Plasma current distribution assumed to shift rigidly in both radial and vertical directions:

$$\delta J_\varphi = \left. \frac{\partial J_\varphi}{\partial R_m} \right|_{Rigid} \delta R_m + \left. \frac{\partial J_\varphi}{\partial z} \right|_{Rigid} \delta z$$

- Variations in flux at conductors, isoflux segment control points derived from simple Green functions and rigid shift:

$$\delta \psi_c = \frac{\partial \psi_c}{\partial J_\varphi} \left( \left. \frac{\partial J_\varphi}{\partial R_m} \right|_{Rigid} \delta R_m + \left. \frac{\partial J_\varphi}{\partial z} \right|_{Rigid} \delta z \right)$$

$$\delta \psi_{iso} = \frac{\partial \psi_{iso}}{\partial J_\varphi} \left( \left. \frac{\partial J_\varphi}{\partial R_m} \right|_{Rigid} \delta R_m + \left. \frac{\partial J_\varphi}{\partial z} \right|_{Rigid} \delta z \right)$$

- Variations in B-field (Z,R) components at X-point grid locations similarly derived from simple Green functions and rigid shift:

$$\delta B_{z,R}^{grid} = \frac{\partial B_{z,R}^{grid}}{\partial J_\varphi} \left( \left. \frac{\partial J_\varphi}{\partial R_m} \right|_{Rigid} \delta R_m + \left. \frac{\partial J_\varphi}{\partial z} \right|_{Rigid} \delta z \right)$$

## Certain Effects Included in the Minimal Model are Key to Producing Accurate Plasma Response

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- Explicit effects of  $\kappa$  significantly greater than 1:

$$B_z^{Shafranov} \equiv -\frac{\mu_0 I_p}{4\pi R_m} \left( \ln \left| \frac{8R_m}{a\sqrt{\kappa}} \right| + \frac{2\kappa}{\kappa^2 + 1} \beta_p + \frac{l_i}{2} - 1.5 \right)$$

- Radial response to variation in  $\beta_p$  and  $l_i$ :

$$\frac{\partial R_m}{\partial \beta_p} \equiv \left[ \frac{B_{zm0}}{R_0} n + \frac{\mu_0 I_{p0}}{4\pi R_0^2} (\Gamma_0 - 1) \right]^{-1} \frac{\mu_0 I_{p0}}{4\pi R_0} \left( \frac{2\kappa}{\kappa^2 + 1} \right) + \frac{\partial R_m}{\partial z} \frac{\partial z}{\partial \beta_p}$$

$$\frac{\partial R_m}{\partial l_i} \equiv \left[ \frac{B_{zm0}}{R_0} n + \frac{\mu_0 I_{p0}}{4\pi R_0^2} (\Gamma_0 - 1) \right]^{-1} \frac{\mu_0 I_{p0}}{8\pi R_0}$$

- Response of *vertical* position to variation in  $\beta_p$ :

$$\frac{\partial z}{\partial \beta_p} \approx \frac{\int R B_{R0} J_\phi dA_p}{\beta_{p0} B_{zJ0} n_J}$$

- Cross-coupling of radial and vertical responses:

$$\frac{\partial R_m}{\partial I_c} \equiv \left[ \frac{B_{zm0}}{R_0} n + \frac{\mu_0 I_{p0}}{4\pi R_0^2} (\Gamma_0 - 1) \right]^{-1} \frac{\partial B_{zm}}{\partial I_c} + \frac{\partial R_m}{\partial z} \frac{\partial z}{\partial I_c}$$

- Plasma (resistive) current response:

$$L_p \frac{dI_p}{dt} + R_p I_p + \frac{\partial \psi_p}{\partial R_m} \frac{dR_m}{dt} + \frac{\partial \psi_p}{\partial z} \frac{dz}{dt} + \frac{\partial \psi_p}{\partial I_c} \frac{dI_c}{dt} + I_{p0} \frac{\partial L_p}{\partial l_i} \frac{dl_i}{dt} = V_L$$

## Plasma Perturbation Experiments Were Performed on DIII-D to Validate Plasma Response Model

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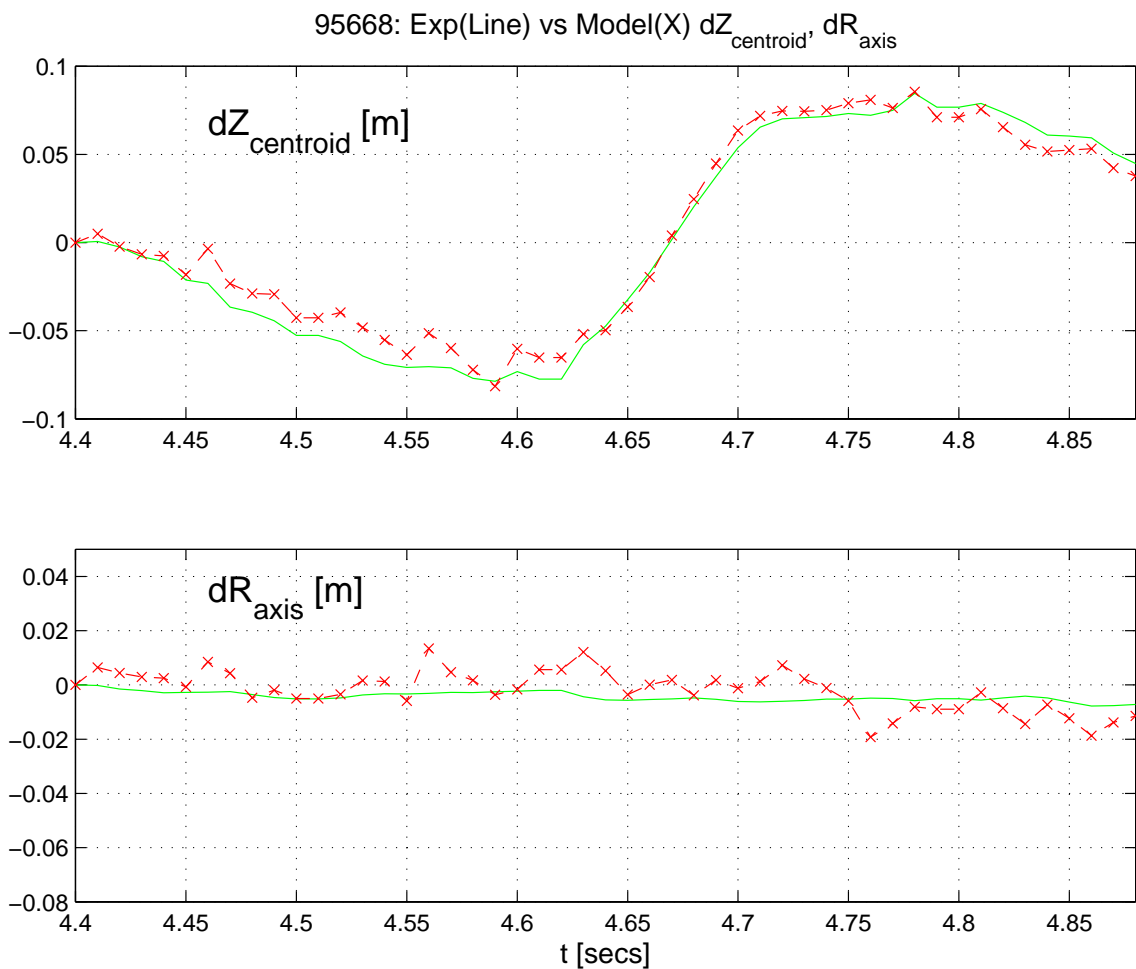
- **Wide range of equilibria perturbed in dedicated or piggyback experiments:**
  - ⇒ Lower/upper single null, double null;
  - ⇒ Ohmic → High  $\beta_p$ ; Low → High  $I_i$ .
- **Many plasma position/shape parameters varied:**
  - ⇒ Major radius, vertical position (rigid shifts);
  - ⇒ Inner/outer gaps (independently);
  - ⇒ Top gap (for lower single null);
  - ⇒ X-point radial/vertical position;
  - ⇒ Elongation (plasma height/width separately);
- **Several different waveforms** programmed to vary plasma position/shape parameters dynamically:
  - ⇒ Triangle, step, sinusoidal waveforms;
  - ⇒ Periods ranging from 20 → 400 msec.
- **Actual coil current,  $\beta_p$  history from experiment used to calculate model-predicted plasma/diagnostic response:**

$$\begin{bmatrix} \delta R_m \\ \delta B_{probe} \\ \delta \psi_{isoflux} \\ \mathbb{M} \end{bmatrix} = [C] \begin{bmatrix} \delta I_c \\ \delta \beta_p \\ \delta I_i \\ \mathbb{M} \end{bmatrix}$$

# Simple Model Reproduces Experimental Ohmic Plasma Response with High Accuracy:

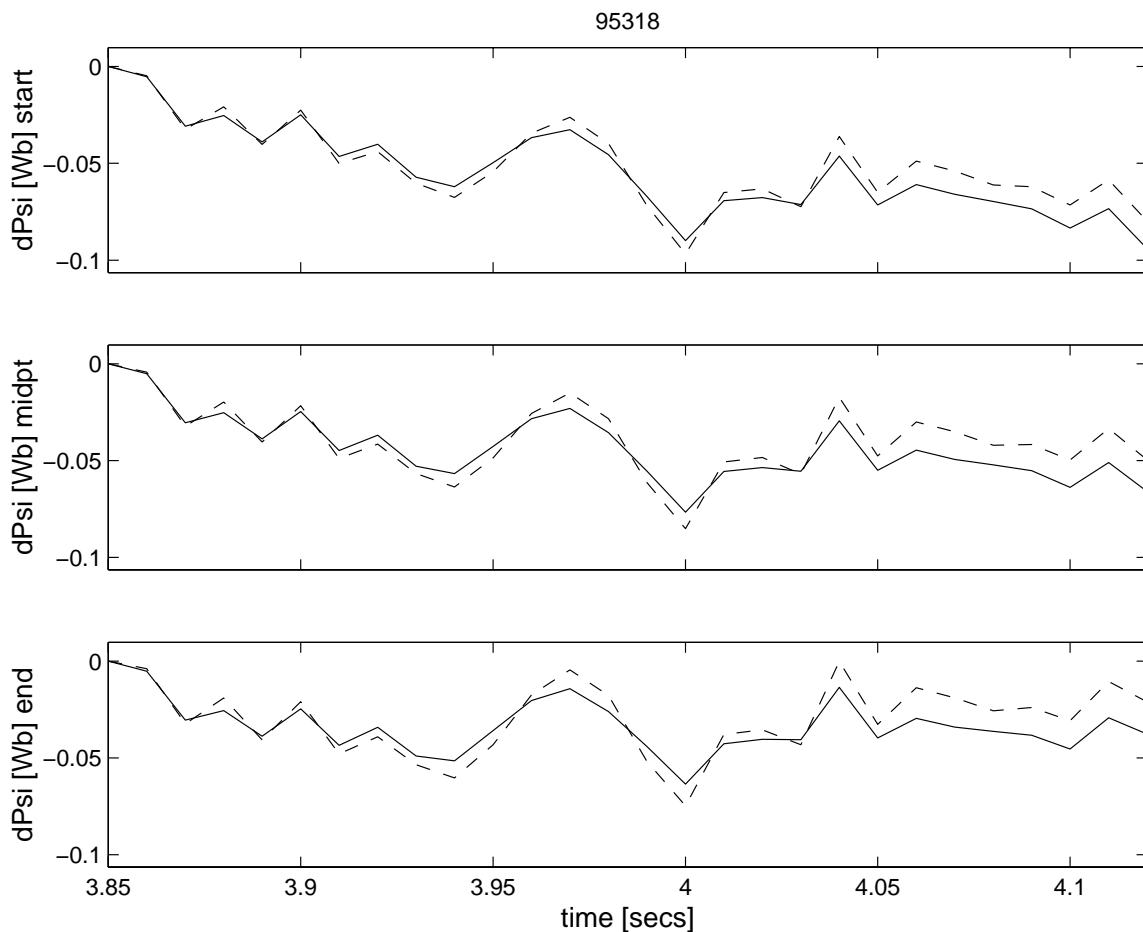
$$\Delta z, \Delta R_m$$

- **Experiment** (equilib. reconstruction)=**line**, **Model**=**X**;
- $dZ_{\text{centroid}}, dR_{\text{axis}}$  = centroid-vertical, axis-radial positions;
- **Vertical position intentionally perturbed with pre-programmed waveform to obtain model test data:**



# Simple Model Reproduces Experimental Ohmic Plasma Response with High Accuracy: $\Delta\psi$ (inboard-midplane isoflux segment)

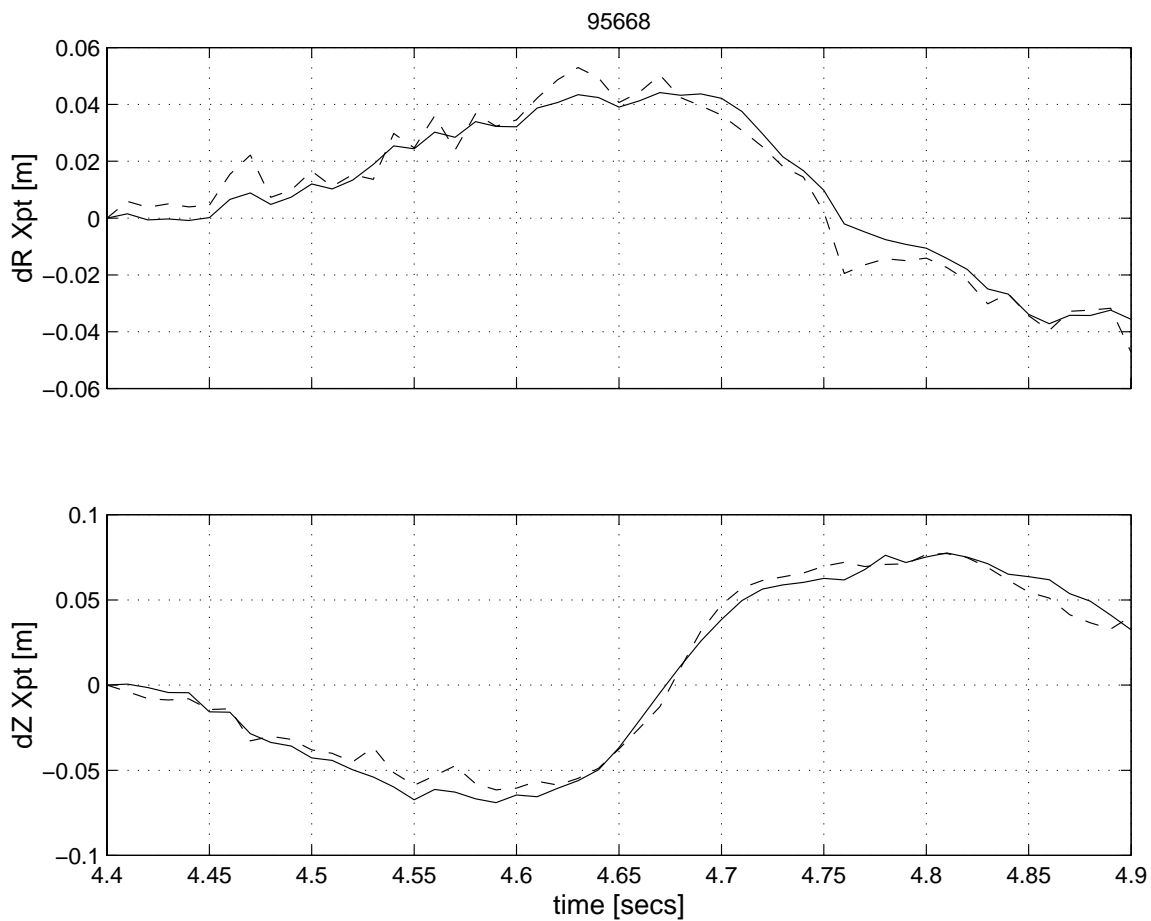
- Model accurately reproduces  $\Delta\psi$  inside/outside plasma, near plasma boundary, for ramping major radius;
- Experiment=solid, Model=dashed;



# Simple Model Reproduces Experimental Ohmic Plasma Response with High Accuracy:

## X-Point $\Delta z_x, \Delta R_x$

- Experiment (equilib. reconstruction)=solid, Model=X;
- $dZ_x, dR_x = X$ -point position;
- Vertical position intentionally perturbed with pre-programmed waveform to obtain model test data:

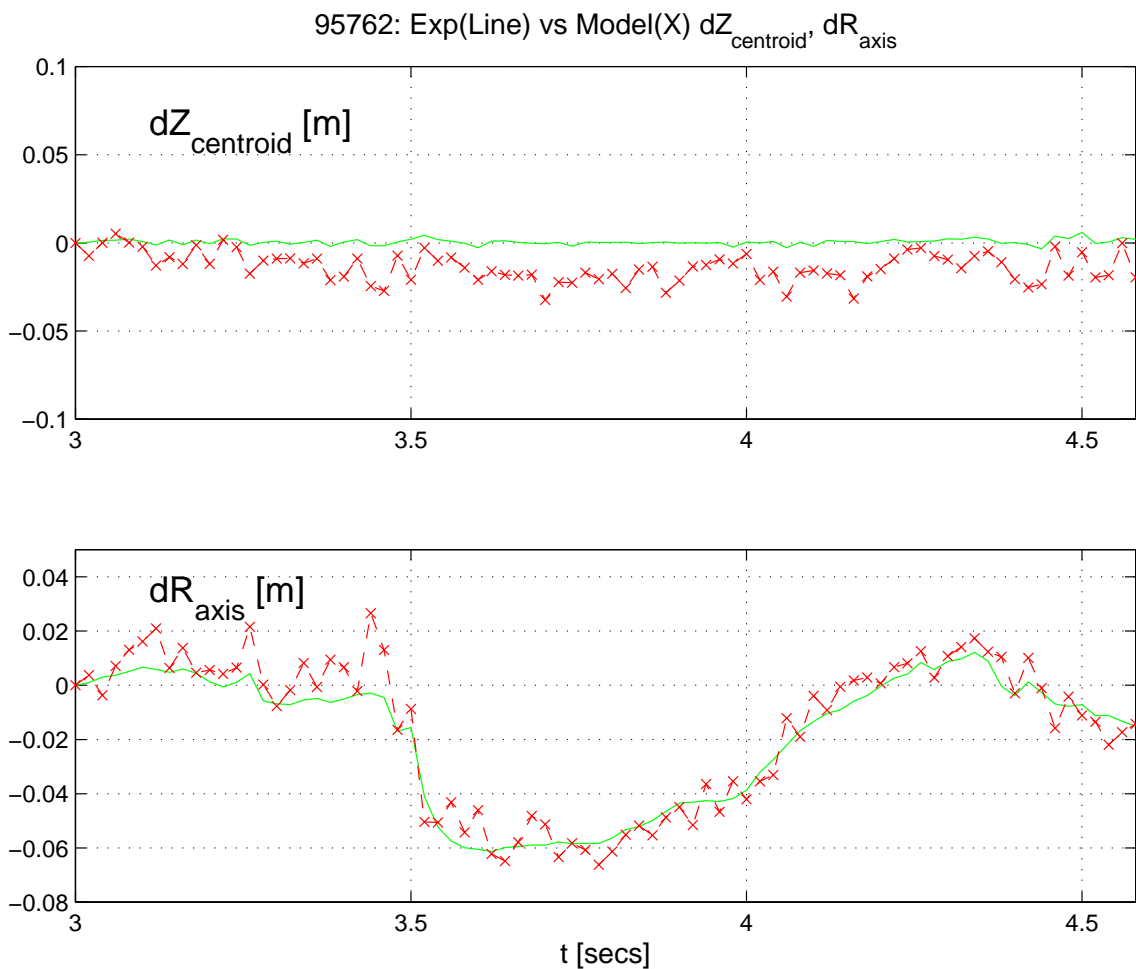




# Simple Model Reproduces Experimental **High- $\beta_p$** Plasma Response with High Accuracy:

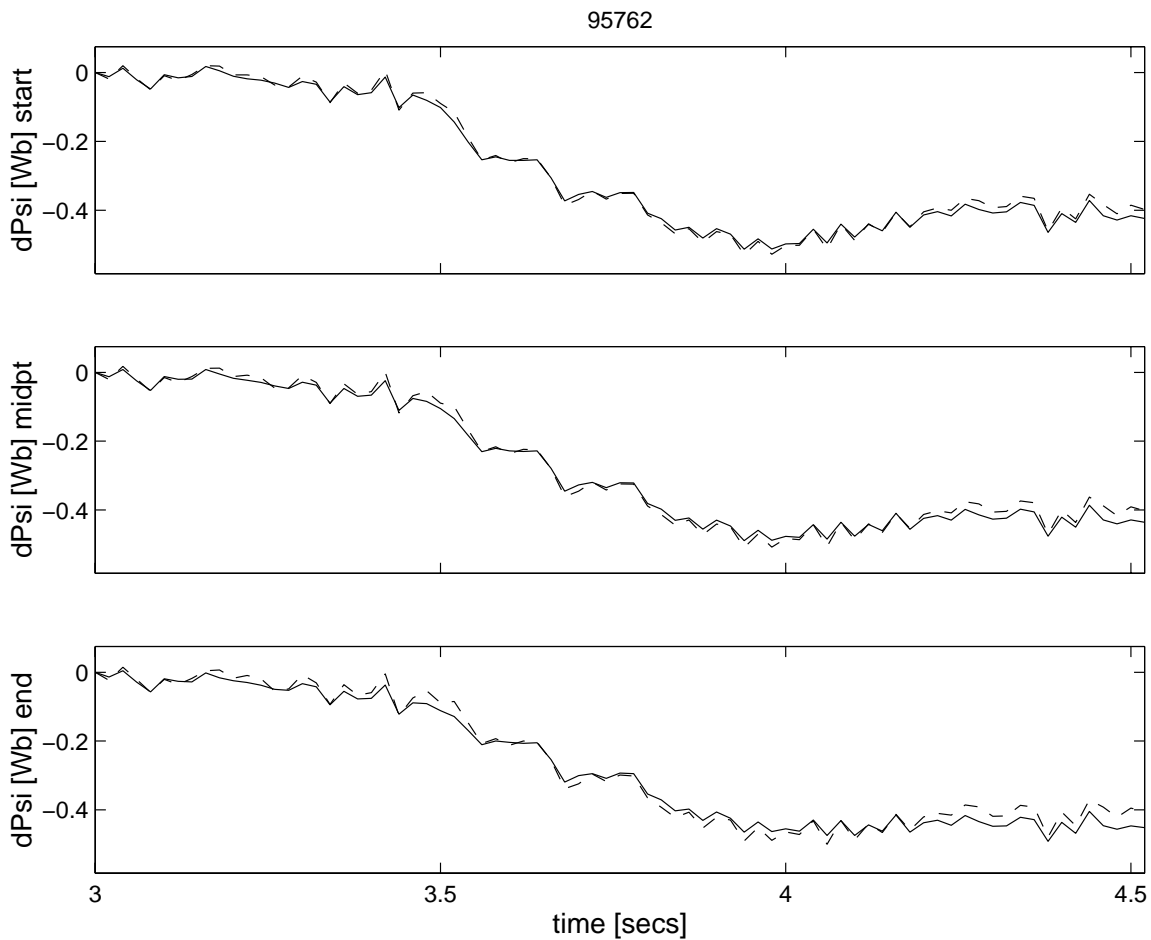
**$\Delta z, \Delta R_m$**

- **Experiment** (equilib. reconstruction)=**line**, **Model**=**X**;
- $dZ_{\text{centroid}}, dR_{\text{axis}}$  = centroid-vertical, axis-radial positions;
- $\beta_p$  dropped from  $\sim 1.1$  to  $\sim 0.5$  at  $t \sim 3.5$  s due to Ar gas puff  
 $\Rightarrow$  plasma moved radially inboard:



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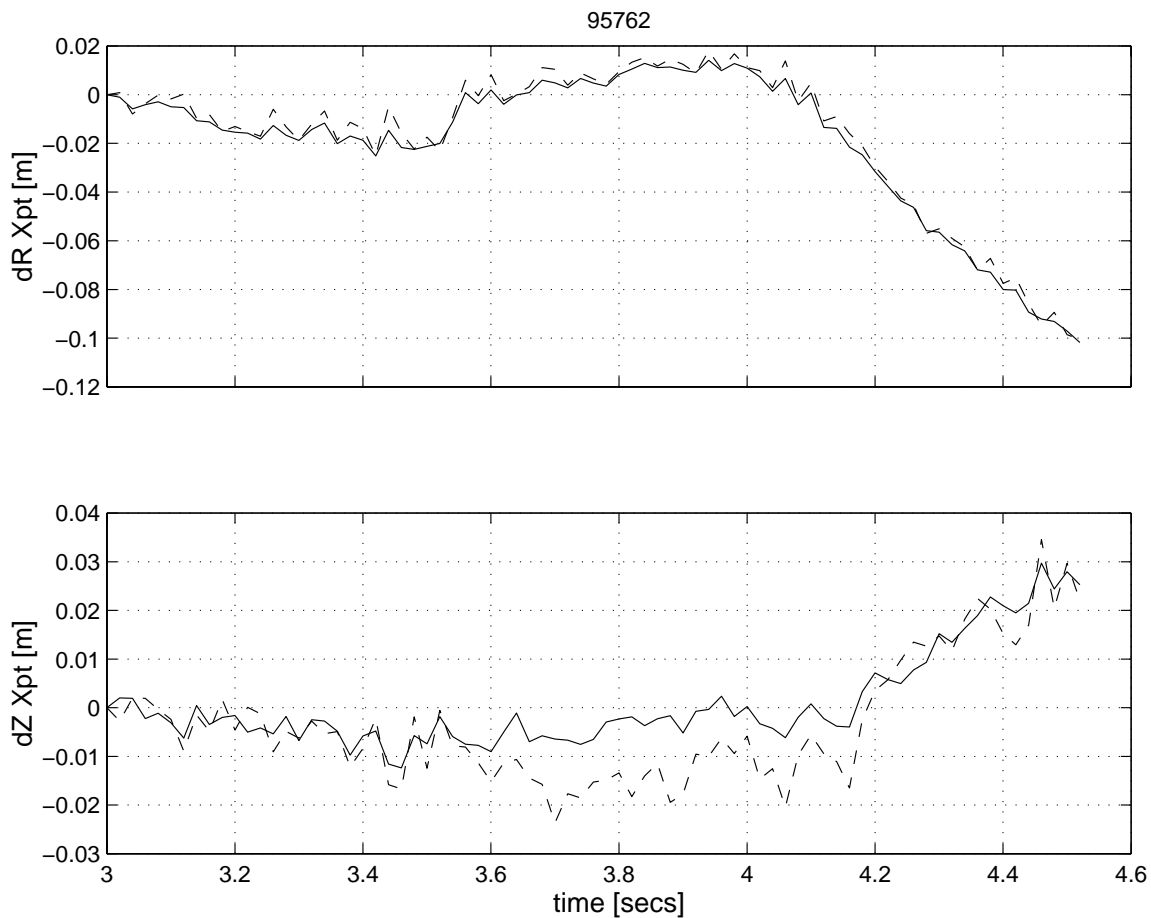
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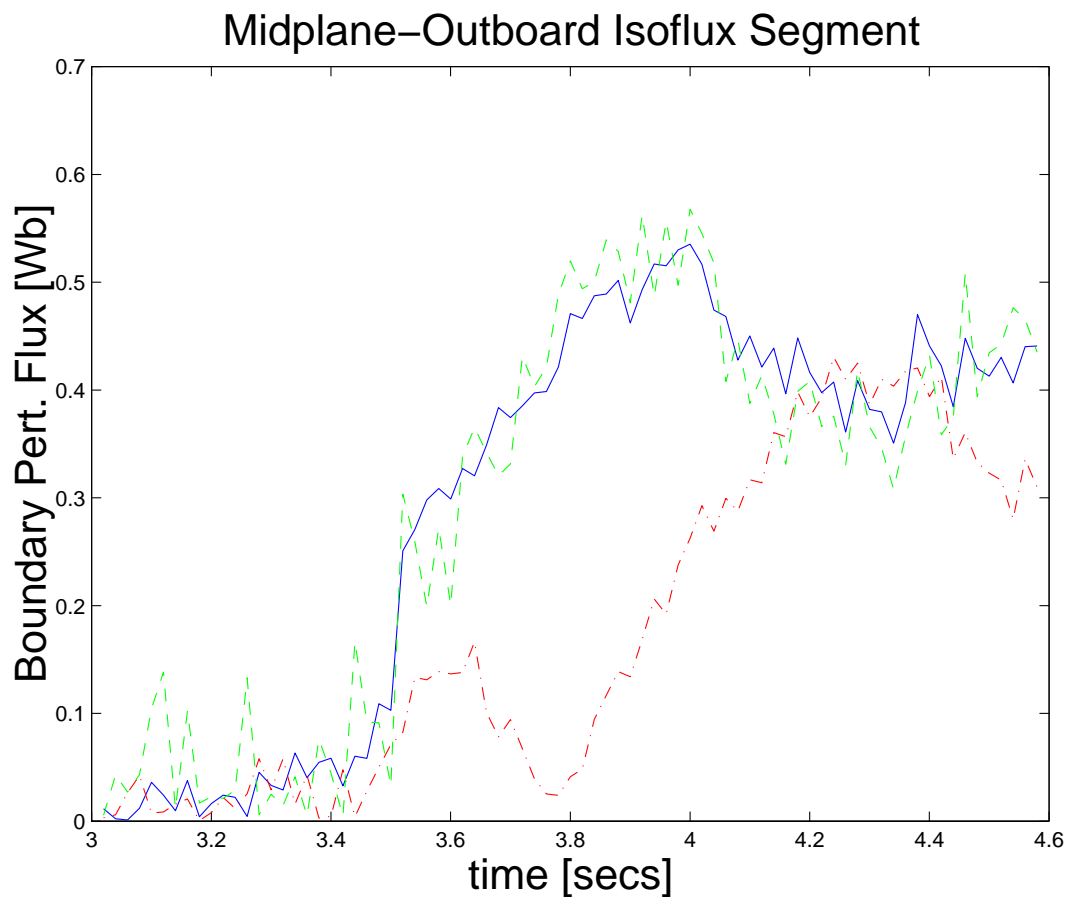
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 $\Rightarrow$  plasma moved radially inboard:



## A “Response Model” Based on Vacuum Response (No Plasma Contribution) Can Be VERY Inaccurate

- Example: Radial shift of high- $\beta_p$  single null plasma;
- Experiment (solid blue), plasma model (dashed green) differ significantly from vacuum model (dash-dot red) over interval  $3.5 < t < 4.2$  s:



Shot 9576201–Sep–1998

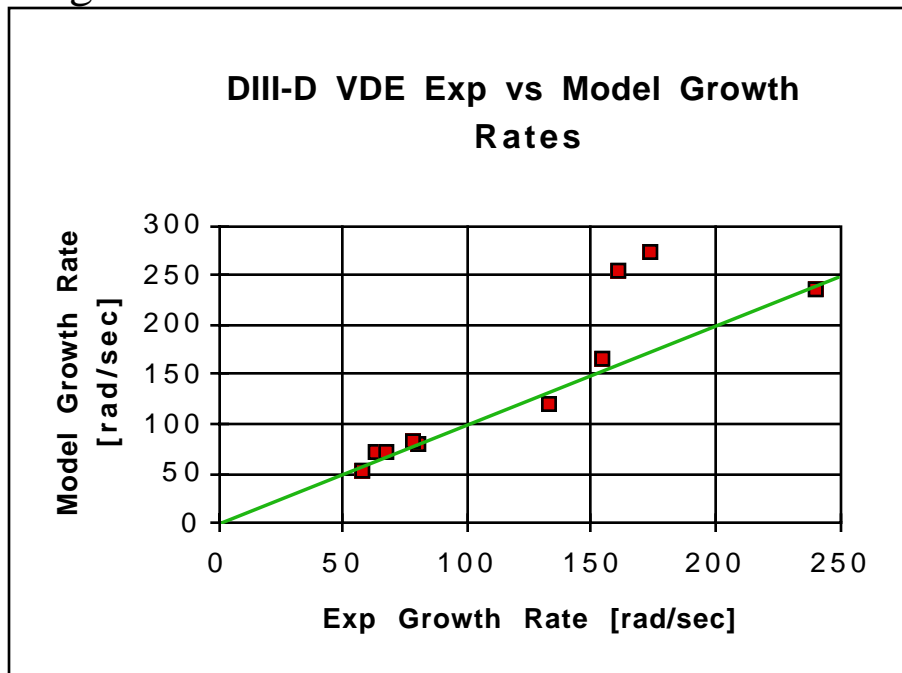
## Accurate Model of Vertical Plasma Response is Important for Designing Good Dynamic Controller

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- **Present DIII-D plasma control** has dynamically “separated” vertical position and stability control:
  - **Stability control uses velocity** ( $\propto dZ/dt$ ) feedback provided by fast ( $\sim 60 \mu\text{sec}$ ) separate cpu;
  - **Position control provided as part of shape control** with longer cycle time ( $\sim 1 \text{ msec}$ ) cpu;
- **MIMO controllers on DIII-D must** be designed to provide good shape control and **minimize interaction with (fast) stability control loop**;
- **Good dynamic control therefore requires accurate model of vertical plasma response**:
  - Open loop vertical growth rate;
  - Effect of stabilization (by fast velocity feedback) on vertical plasma displacement mode.
- Effect of **vertical motion is also important in** order to produce **correct radial plasma response**;

## Rigid Current-Conserving Model Vertical Growth Rate Matches DIII-D VDE Experiments

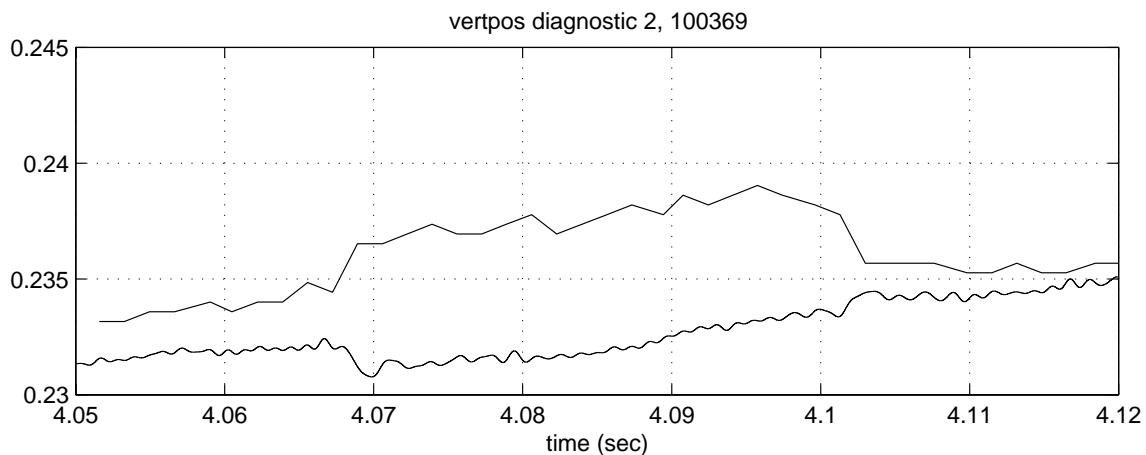
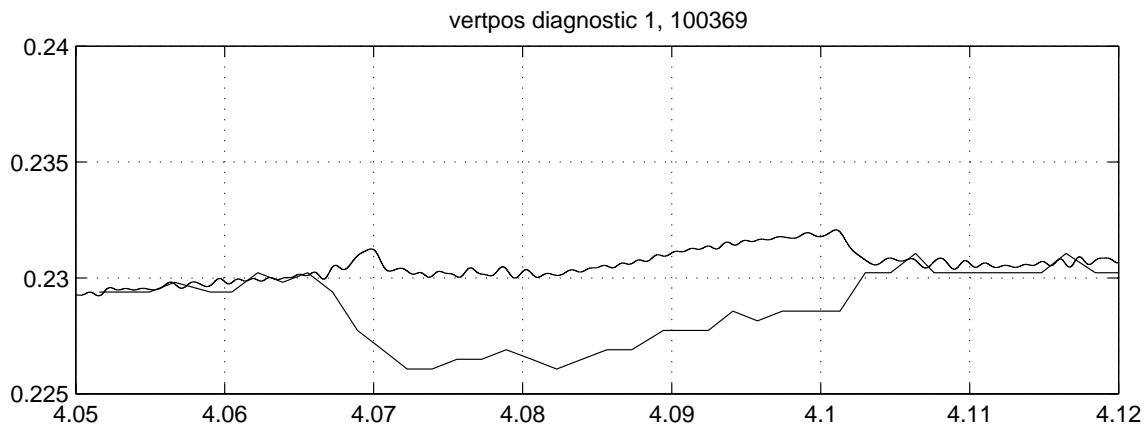
- Growth rates from current-conserving ( $\delta I_p=0$ ) rigid vertical model compared with experimentally measured VDE growth rates:



- Most VDE's yield vertical growth rate within ~10% of predicted value;
- Small subset of discharges produce VDE's with vertical growth rates differing by as much as 50% from predicted values:
  - Source of difference not yet well understood;
  - ~ 50% error in growth rate is tolerable for dynamic controller design in such cases.

## Incorrect Model of Vertical Instability Response Produces **Incorrect** B-Field Measurements

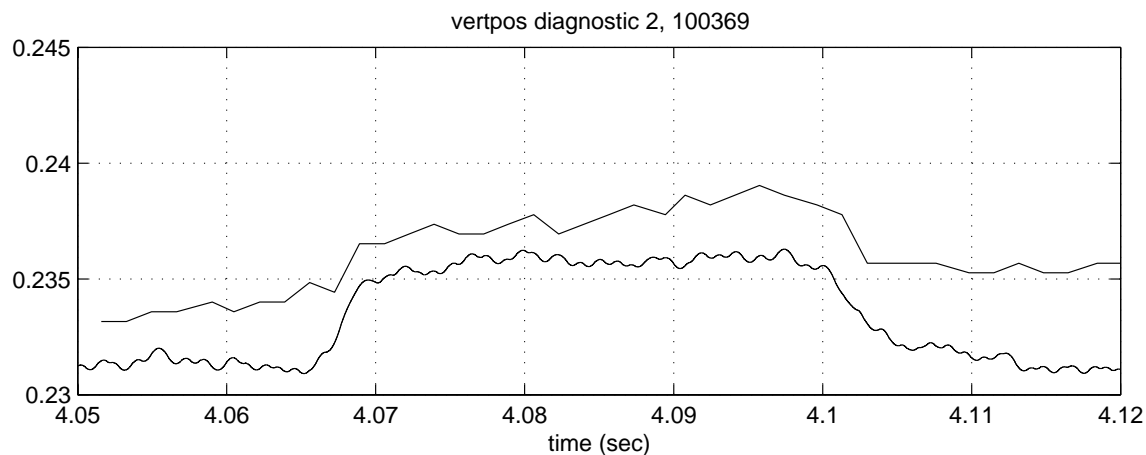
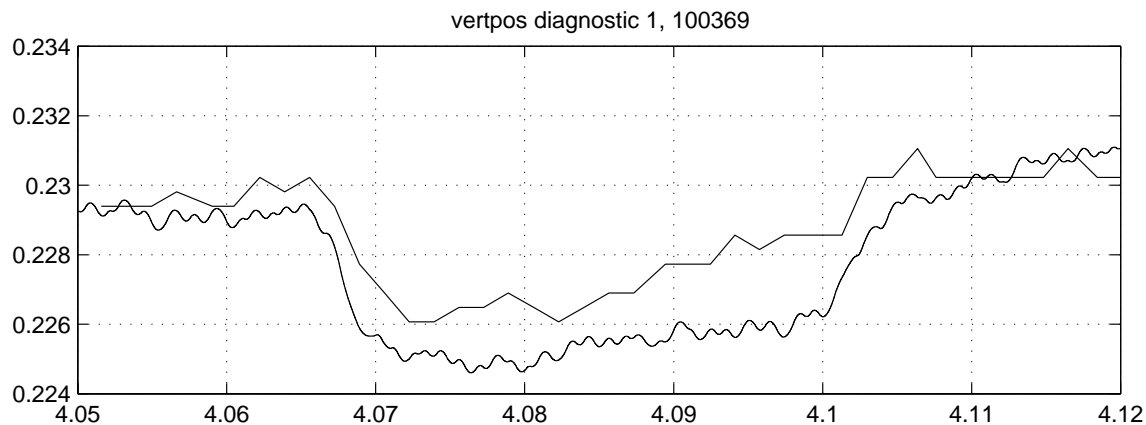
- Open loop simulation with **vertical instability removed** (to roughly model effect of fast vertical velocity control loop) **shows incorrect response of outboard poloidal field probes:**



- Vertical model affects both vertical and *radial* response: cross-coupling terms are significant in up-down asymmetric plasmas (this case = Upper Single Null).

## Inclusion of Accurate Vertical Instability Mode in Plasma Model Produces **Correct** Probe Measurements

- Closed loop simulation with **vertical instability included** (and effect of fast velocity control loop correctly included in model) **shows correct response of outboard poloidal field probes:**

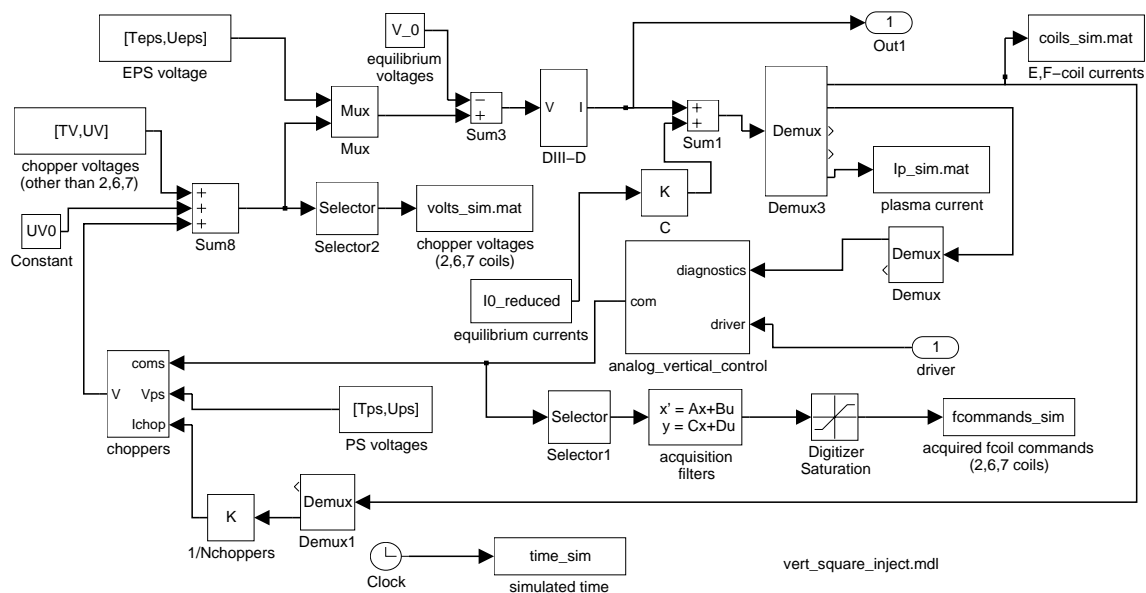


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# Accurate Representation and Simulation of Vertical Response in Tokamaks is Complex Problem

- **Simulation/validation of intrinsically unstable open-loop plasma model is difficult:** little more than growth rate can be validated;



- **Simulation/validation of closed-loop response in DIII-D requires** accurate modeling of highly nonlinear **chopper power supplies** and **digital controller** action (as well as unstable plasma).

## CONCLUSIONS

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- **High dynamic accuracy** of performance in DIII-D isoflux boundary/X-point control **requires accurate plasma response models** for controller design.
- **“Minimal” analytically-based plasma models:**
  - **Are sufficient** to accurately describe plasma response for control;
  - **Have been developed and extensively validated** against DIII-D experiments.
  - **Have been used to design successful dynamic shape controllers for DIII-D.** [See poster JP1.33, this session]
- **Physics terms** important to plasma response have been identified:
  - Effect of plasma elongation;
  - Effects of variation in  $\beta_p$  and  $l_j$ ;
  - Cross-coupling of vertical and radial displacement;
  - Validated physics understanding allows **models to be extended to other operating regimes and devices** more reliably.

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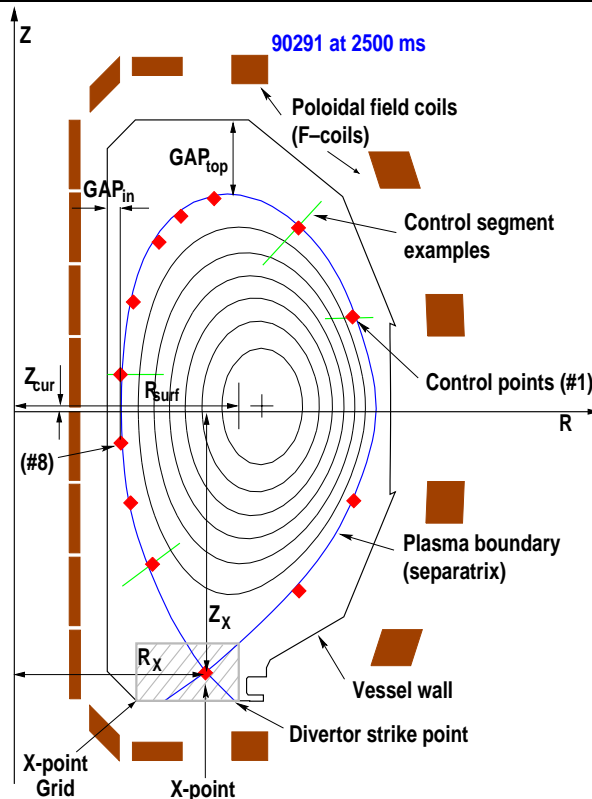
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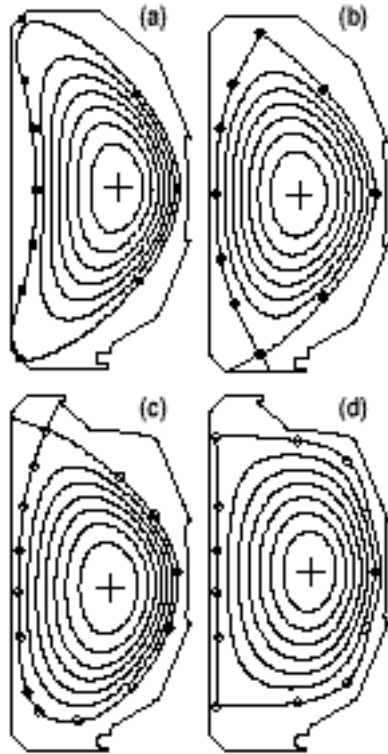
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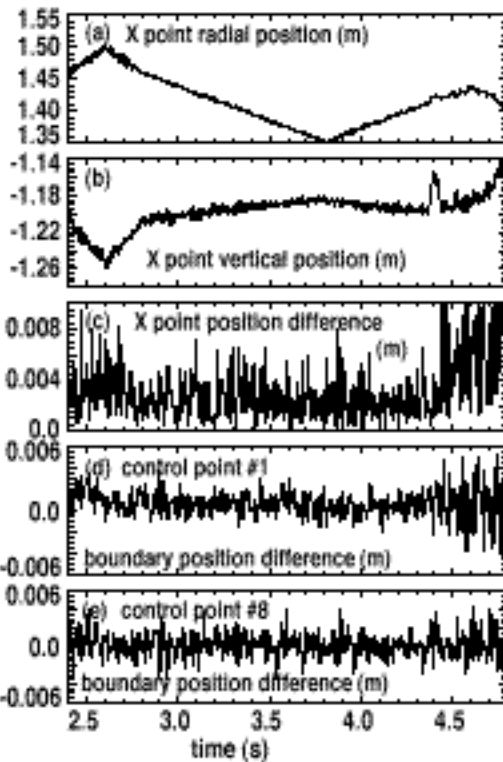
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  - **Test and optimize** controller performance with linear and nonlinear model simulations;
- **Advantages:**
  - **Controllers allow significant variation** in plasma equilibrium without switching to new controller;
  - **New equilibrium/shape development** without consuming valuable experimental time on tokamak;
  - **Improved control performance:** accuracy/precision, dynamic response, disturbance/noise rejection.
  - **Ability to balance tokamak constraints** with desired control.
- **Disadvantages:**
  - Requires reasonably **accurate system response models**;
  - Possible increase in difficulty of tuning control algorithms “by hand” between shots (but with theoretical reduction in need for such tuning);



## Plasma Response Model Based on Rigid Plasma Displacements, Simple Force Balance Relations

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- Goal: Use “**minimally complex**” models which provide sufficiently accurate representation of experiment.
- Simple **radial force balance**:

$$B_{zm}^{appl} = B_z^{Shafranov} = -\frac{\mu_0 I_p}{4\pi R_m} \Gamma(R_m, \beta_p, l_i)$$

$$\equiv -\frac{\mu_0 I_p}{4\pi R_m} \left( \ln \left| \frac{8R_m}{a\sqrt{\kappa}} \right| + \frac{2\kappa}{\kappa^2 + 1} \beta_p + \frac{l_i}{2} - 1.5 \right)$$

- Simple **vertical force balance**:

$$\mathbf{0} = -2\pi R_m I_{p0} \frac{\partial B_R}{\partial z} \delta z + I_{p0} \frac{\partial M_{pc}}{\partial z} \delta I_c + \frac{\partial F_z}{\partial \beta_p} \delta \beta_p + \frac{\partial F_z}{\partial l_i} \delta l_i$$

- **Plasma response objects** from perturbed force balance:

$$R_m \text{ Current-Resp } \frac{\partial R_m}{\partial I_c} \equiv \left[ \frac{B_{zm0}}{R_0} n + \frac{\mu_0 I_{p0}}{4\pi R_0^2} (\Gamma_0 - 1) \right]^{-1} \frac{\partial B_{zm}}{\partial I_c} + \frac{\partial R_m}{\partial z} \frac{\partial z}{\partial I_c}$$

$$R_m \text{ } I_p \text{-Response } \frac{\partial R_m}{\partial I_p} \equiv \left[ \frac{B_{zm0}}{R_0} n + \frac{\mu_0 I_{p0}}{4\pi R_0^2} (\Gamma_0 - 1) \right]^{-1} \frac{\mu_0 \Gamma_0}{4\pi R_0}$$

$$\frac{\partial R_m}{\partial \beta_p} \equiv \left[ \frac{B_{zm0}}{R_0} n + \frac{\mu_0 I_{p0}}{4\pi R_0^2} (\Gamma_0 - 1) \right]^{-1} \frac{\mu_0 I_{p0}}{4\pi R_0} \left( \frac{2\kappa}{\kappa^2 + 1} \right) + \frac{\partial R_m}{\partial z} \frac{\partial z}{\partial \beta_p}$$

$$\frac{\partial R_m}{\partial l_i} \equiv \left[ \frac{B_{zm0}}{R_0} n + \frac{\mu_0 I_{p0}}{4\pi R_0^2} (\Gamma_0 - 1) \right]^{-1} \frac{\mu_0 I_{p0}}{8\pi R_0}$$

$$Z_c \text{ Curr-Resp } \frac{\partial z}{\partial I_c} \equiv \frac{(\partial M_{pc} / \partial z)}{-2\pi B_{zJ0} n_J}, \quad Z_c \beta_p \text{-Resp } \frac{\partial z}{\partial \beta_p} \approx \frac{\int R B_{R0} J_\phi dA_p}{\beta_{p0} B_{zJ0} n_J}$$

## Rigid Radial and Vertical Plasma Displacement Assumption Closes Plasma Response Equations

---

- Plasma current distribution assumed to shift rigidly in both radial and vertical directions:

$$\delta J_\varphi = \left. \frac{\partial J_\varphi}{\partial R_m} \right|_{Rigid} \delta R_m + \left. \frac{\partial J_\varphi}{\partial z} \right|_{Rigid} \delta z$$

- Variations in flux at conductors, isoflux segment control points derived from simple Green functions and rigid shift:

$$\delta \psi_c = \frac{\partial \psi_c}{\partial J_\varphi} \left( \left. \frac{\partial J_\varphi}{\partial R_m} \right|_{Rigid} \delta R_m + \left. \frac{\partial J_\varphi}{\partial z} \right|_{Rigid} \delta z \right)$$

$$\delta \psi_{iso} = \frac{\partial \psi_{iso}}{\partial J_\varphi} \left( \left. \frac{\partial J_\varphi}{\partial R_m} \right|_{Rigid} \delta R_m + \left. \frac{\partial J_\varphi}{\partial z} \right|_{Rigid} \delta z \right)$$

- Variations in B-field (Z,R) components at X-point grid locations similarly derived from simple Green functions and rigid shift:

$$\delta B_{z,R}^{grid} = \frac{\partial B_{z,R}^{grid}}{\partial J_\varphi} \left( \left. \frac{\partial J_\varphi}{\partial R_m} \right|_{Rigid} \delta R_m + \left. \frac{\partial J_\varphi}{\partial z} \right|_{Rigid} \delta z \right)$$

## Certain Effects Included in the Minimal Model are Key to Producing Accurate Plasma Response

---

- Explicit effects of  $\kappa$  significantly greater than 1:

$$B_z^{Shafranov} \equiv -\frac{\mu_0 I_p}{4\pi R_m} \left( \ln \left| \frac{8R_m}{a\sqrt{\kappa}} \right| + \frac{2\kappa}{\kappa^2 + 1} \beta_p + \frac{l_i}{2} - 1.5 \right)$$

- Radial response to variation in  $\beta_p$  and  $l_i$ :

$$\frac{\partial R_m}{\partial \beta_p} \equiv \left[ \frac{B_{zm0}}{R_0} n + \frac{\mu_0 I_{p0}}{4\pi R_0^2} (\Gamma_0 - 1) \right]^{-1} \frac{\mu_0 I_{p0}}{4\pi R_0} \left( \frac{2\kappa}{\kappa^2 + 1} \right) + \frac{\partial R_m}{\partial z} \frac{\partial z}{\partial \beta_p}$$

$$\frac{\partial R_m}{\partial l_i} \equiv \left[ \frac{B_{zm0}}{R_0} n + \frac{\mu_0 I_{p0}}{4\pi R_0^2} (\Gamma_0 - 1) \right]^{-1} \frac{\mu_0 I_{p0}}{8\pi R_0}$$

- Response of *vertical* position to variation in  $\beta_p$ :

$$\frac{\partial z}{\partial \beta_p} \approx \frac{\int R B_{R0} J_\phi dA_p}{\beta_{p0} B_{zJ0} n_J}$$

- Cross-coupling of radial and vertical responses:

$$\frac{\partial R_m}{\partial I_c} \equiv \left[ \frac{B_{zm0}}{R_0} n + \frac{\mu_0 I_{p0}}{4\pi R_0^2} (\Gamma_0 - 1) \right]^{-1} \frac{\partial B_{zm}}{\partial I_c} + \frac{\partial R_m}{\partial z} \frac{\partial z}{\partial I_c}$$

- Plasma (resistive) current response:

$$L_p \frac{dI_p}{dt} + R_p I_p + \frac{\partial \psi_p}{\partial R_m} \frac{dR_m}{dt} + \frac{\partial \psi_p}{\partial z} \frac{dz}{dt} + \frac{\partial \psi_p}{\partial I_c} \frac{dI_c}{dt} + I_{p0} \frac{\partial L_p}{\partial l_i} \frac{dl_i}{dt} = V_L$$

## Plasma Perturbation Experiments Were Performed on DIII-D to Validate Plasma Response Model

---

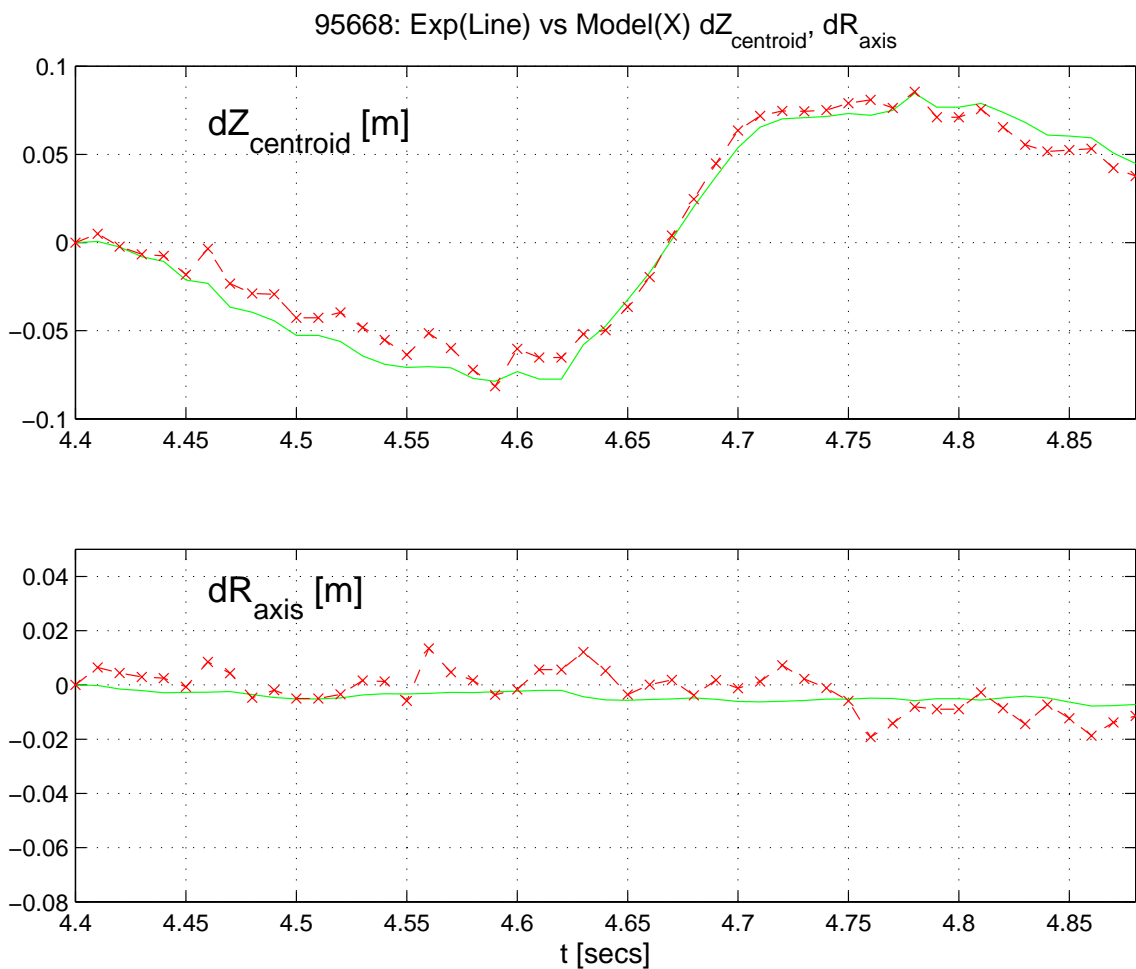
- **Wide range of equilibria perturbed in dedicated or piggyback experiments:**
  - ⇒ Lower/upper single null, double null;
  - ⇒ Ohmic → High  $\beta_p$ ; Low → High  $I_i$ .
- **Many plasma position/shape parameters varied:**
  - ⇒ Major radius, vertical position (rigid shifts);
  - ⇒ Inner/outer gaps (independently);
  - ⇒ Top gap (for lower single null);
  - ⇒ X-point radial/vertical position;
  - ⇒ Elongation (plasma height/width separately);
- **Several different waveforms** programmed to vary plasma position/shape parameters dynamically:
  - ⇒ Triangle, step, sinusoidal waveforms;
  - ⇒ Periods ranging from 20 → 400 msec.
- **Actual coil current,  $\beta_p$  history from experiment used to calculate model-predicted plasma/diagnostic response:**

$$\begin{bmatrix} \delta R_m \\ \delta B_{probe} \\ \delta \psi_{isoflux} \\ \mathbb{M} \end{bmatrix} = [C] \begin{bmatrix} \delta I_c \\ \delta \beta_p \\ \delta I_i \\ \mathbb{M} \end{bmatrix}$$

# Simple Model Reproduces Experimental Ohmic Plasma Response with High Accuracy:

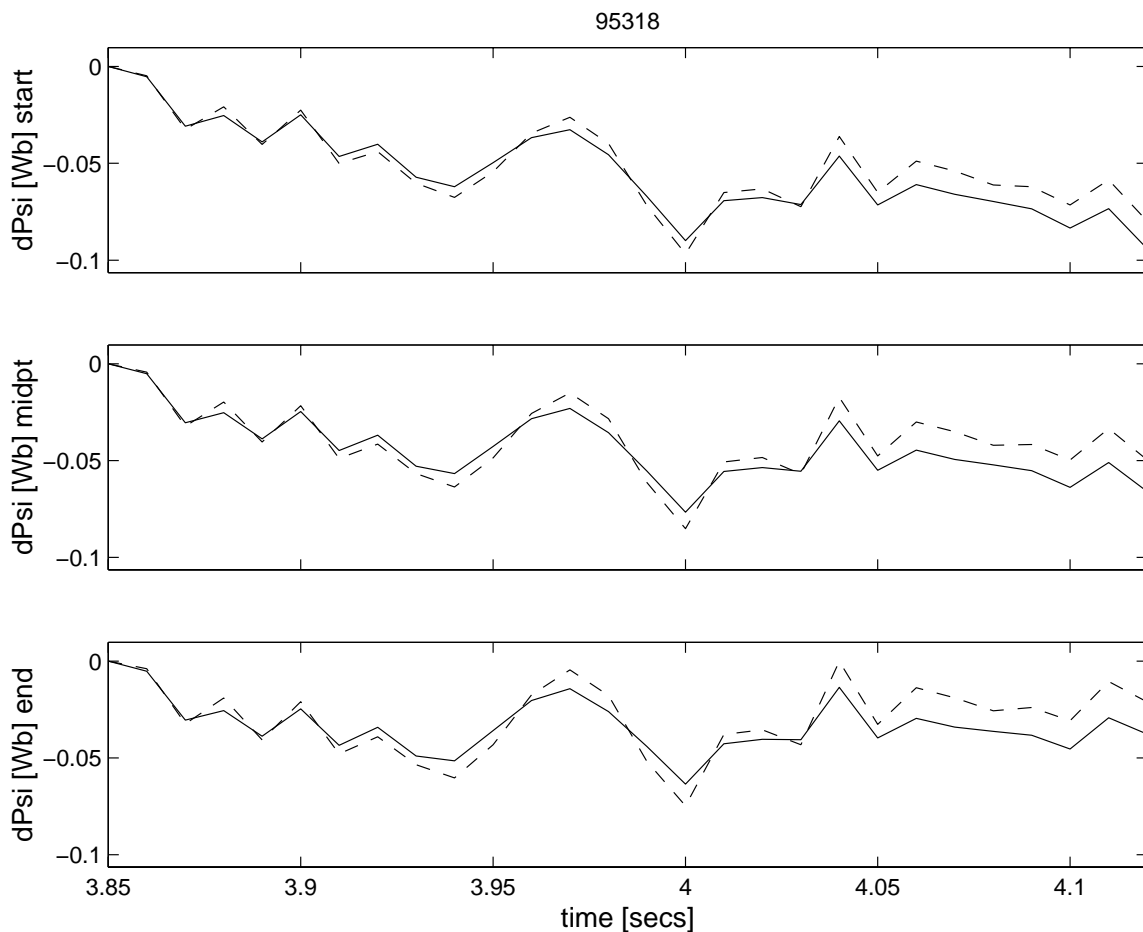
$$\Delta z, \Delta R_m$$

- **Experiment** (equilib. reconstruction)=**line**, **Model**=**X**;
- $dZ_{\text{centroid}}, dR_{\text{axis}}$  = centroid-vertical, axis-radial positions;
- **Vertical position intentionally perturbed with pre-programmed waveform to obtain model test data:**



# Simple Model Reproduces Experimental Ohmic Plasma Response with High Accuracy: $\Delta\psi$ (inboard-midplane isoflux segment)

- Model accurately reproduces  $\Delta\psi$  inside/outside plasma, near plasma boundary, for ramping major radius;
- Experiment=solid, Model=dashed;

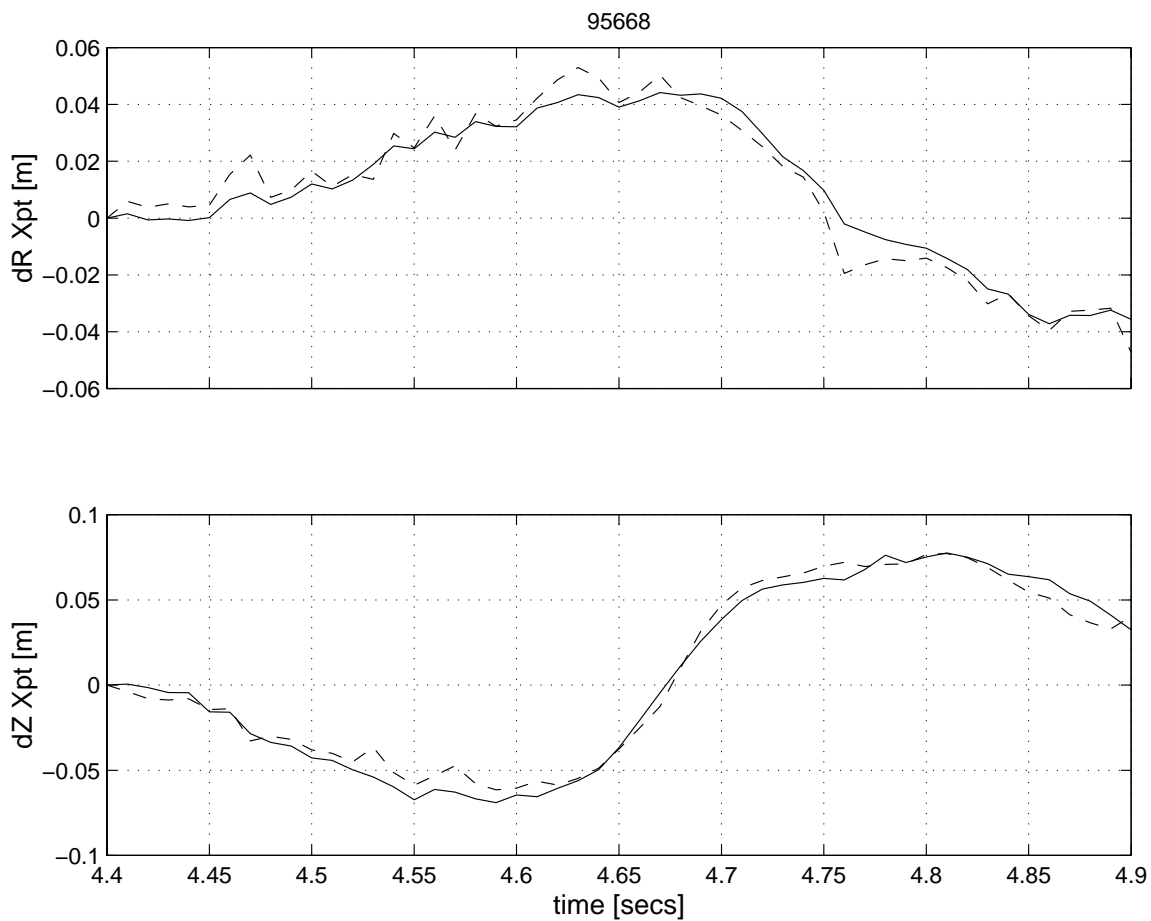




# Simple Model Reproduces Experimental Ohmic Plasma Response with High Accuracy:

## X-Point $\Delta z_x, \Delta R_x$

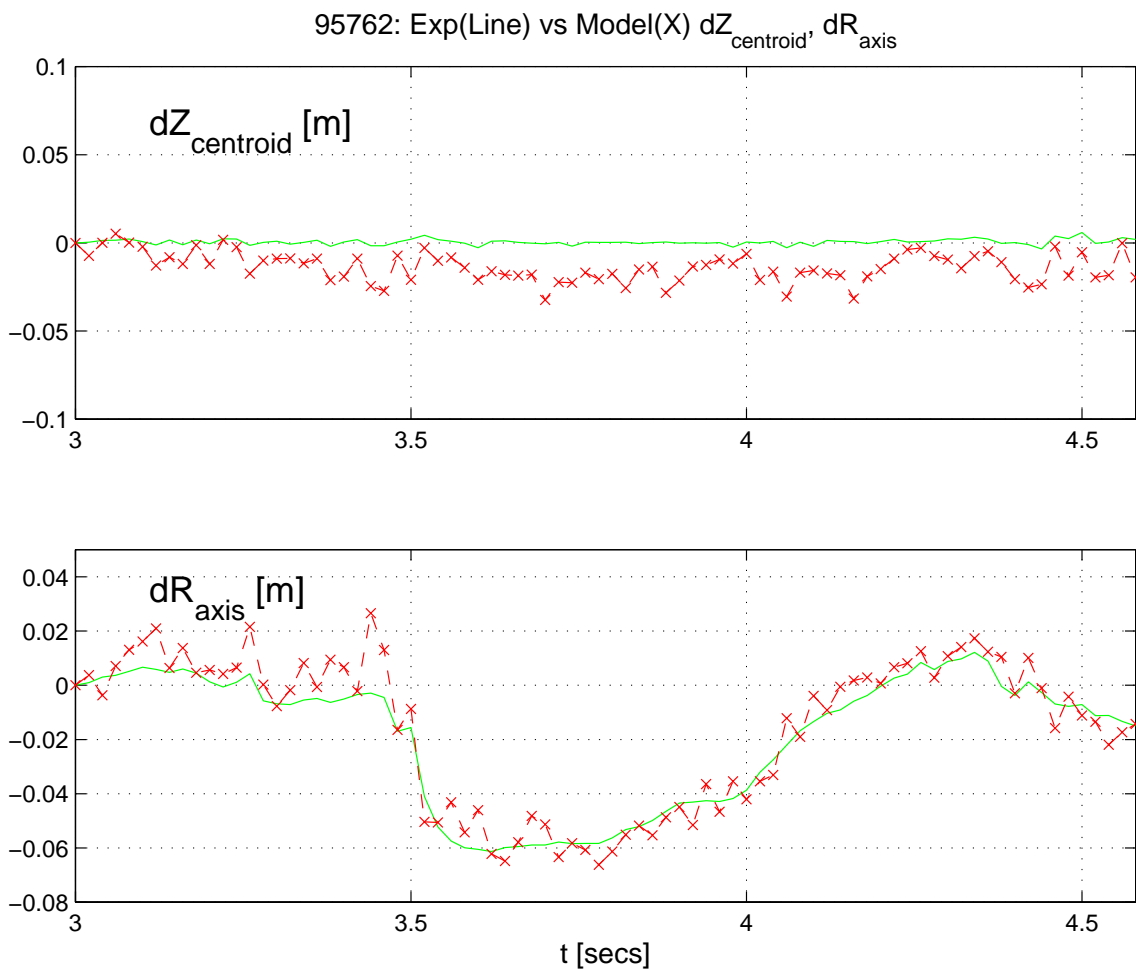
- Experiment (equilib. reconstruction)=solid, Model=X;
- $dZ_x, dR_x = X$ -point position;
- Vertical position intentionally perturbed with pre-programmed waveform to obtain model test data:



# Simple Model Reproduces Experimental High- $\beta_p$ Plasma Response with High Accuracy:

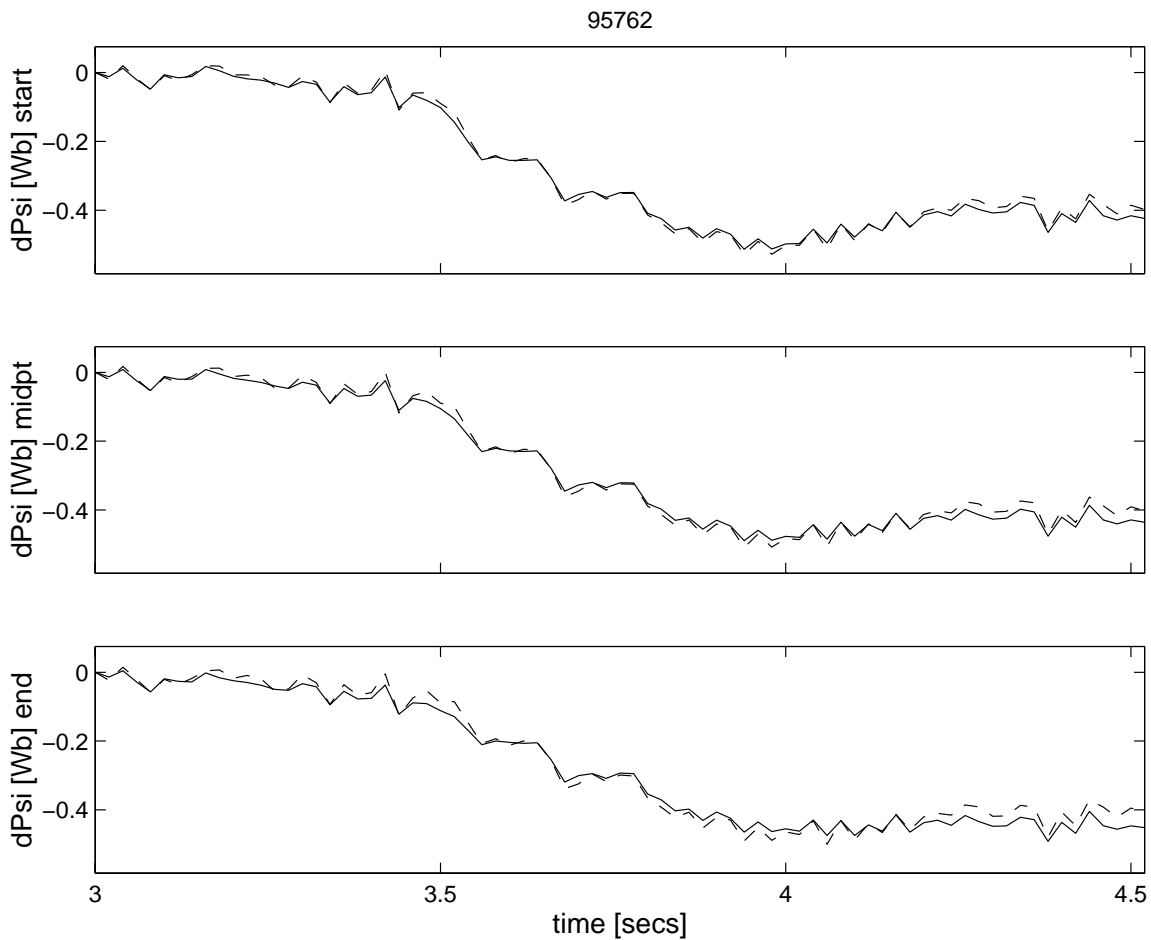
$\Delta z, \Delta R_m$

- Experiment (equilib. reconstruction)=line, Model=X;
- $dZ_{\text{centroid}}, dR_{\text{axis}}$  = centroid-vertical, axis-radial positions;
- $\beta_p$  dropped from  $\sim 1.1$  to  $\sim 0.5$  at  $t \sim 3.5$  s due to Ar gas puff  
 $\Rightarrow$  plasma moved radially inboard:



# Simple Model Reproduces Experimental High- $\beta_p$ Plasma Response with High Accuracy: $\Delta\psi$ (inboard-midplane isoflux segment)

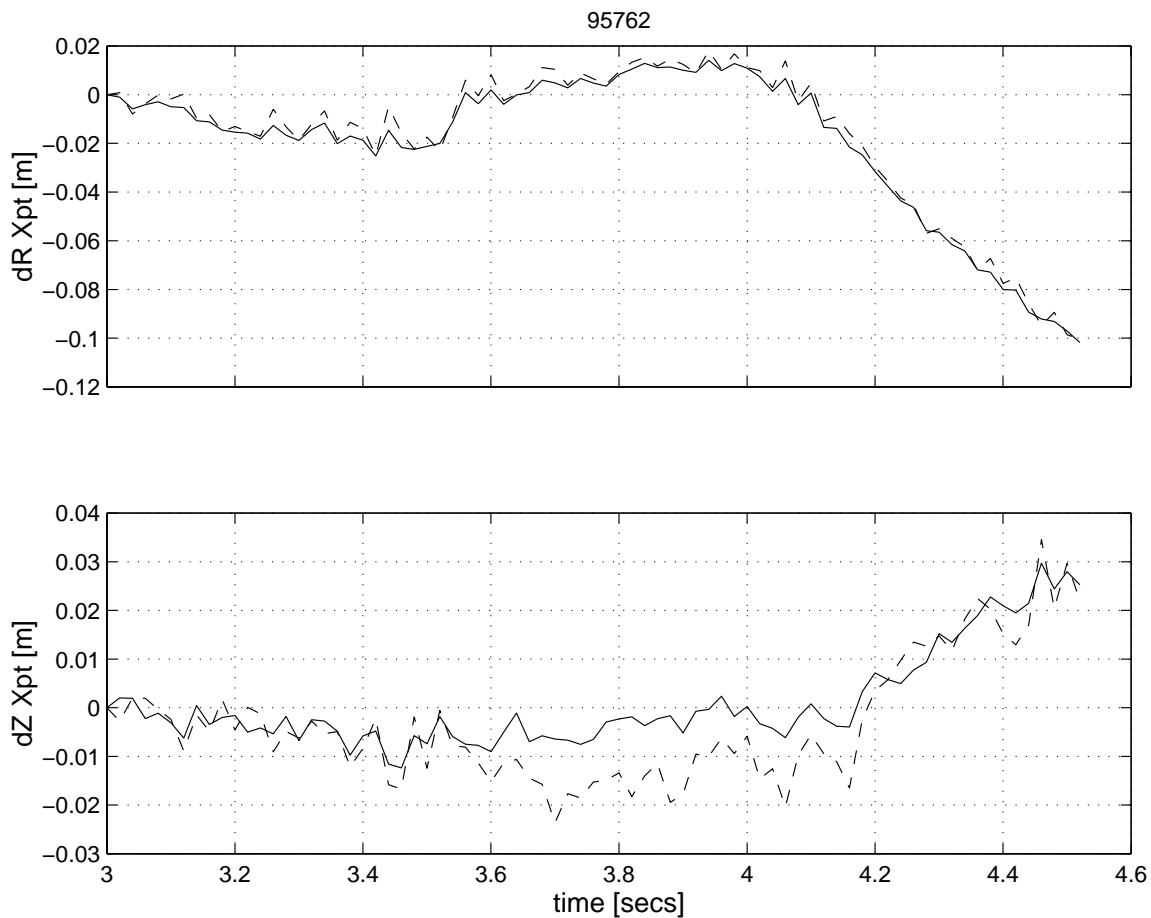
- Model accurately reproduces  $\Delta\psi$  inside/outside plasma, near plasma boundary, for ramping major radius;
- Experiment=solid, Model=dashed:



# Simple Model Reproduces Experimental High- $\beta_p$ Plasma Response with High Accuracy:

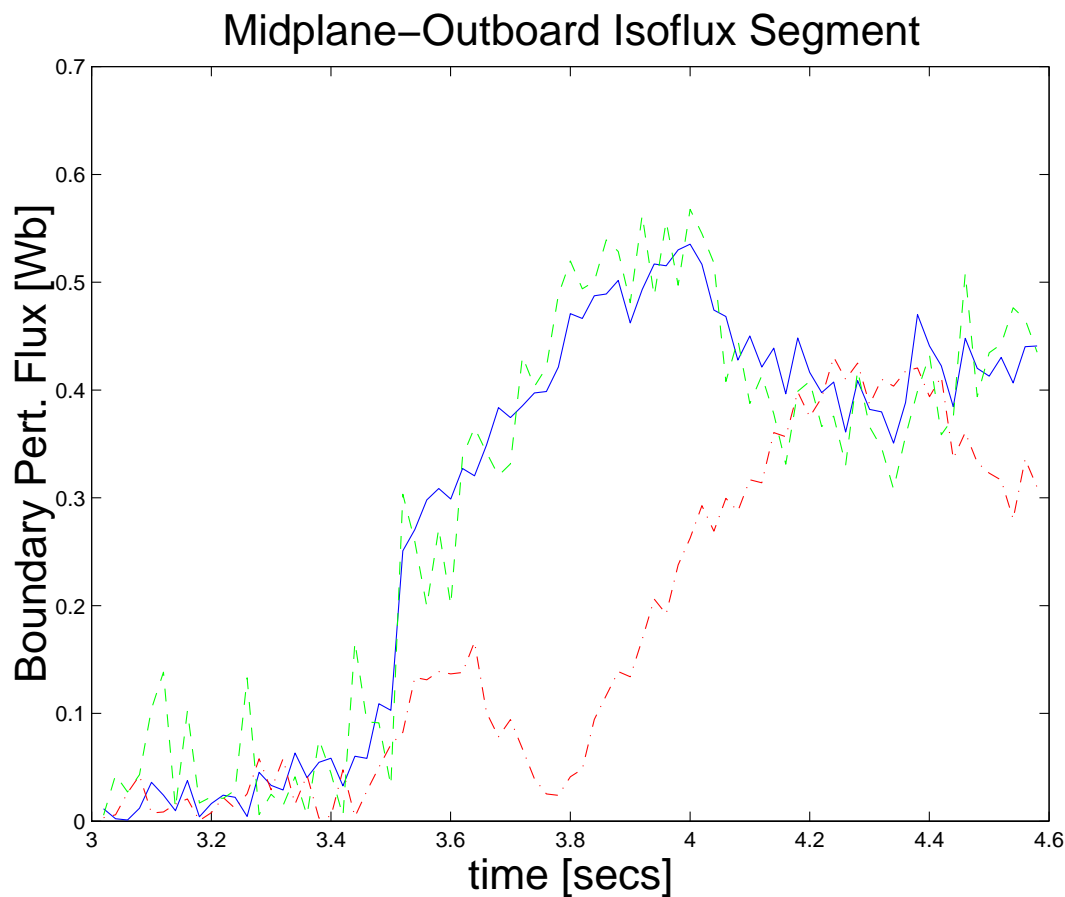
## X-Point $\Delta z_x, \Delta R_x$

- Experiment (equilib. reconstruction)=solid, Model=X;
- $dZ_x, dR_x = X$ -point position;
- $\beta_p$  dropped from  $\sim 1.1$  to  $\sim 0.5$  at  $t \sim 3.5$  s due to Ar gas puff  
 $\Rightarrow$  plasma moved radially inboard:



## A “Response Model” Based on Vacuum Response (No Plasma Contribution) Can Be VERY Inaccurate

- Example: Radial shift of high- $\beta_p$  single null plasma;
- Experiment (solid blue), plasma model (dashed green) differ significantly from vacuum model (dash-dot red) over interval  $3.5 < t < 4.2$  s:



Shot 9576201–Sep–1998

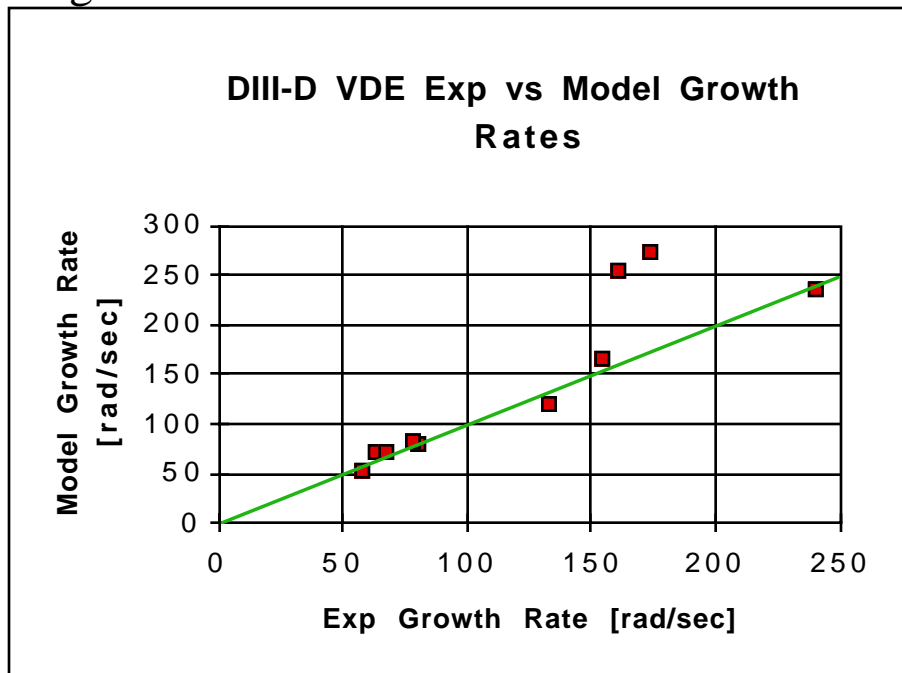
## Accurate Model of Vertical Plasma Response is Important for Designing Good Dynamic Controller

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- **Present DIII-D plasma control** has dynamically “separated” vertical position and stability control:
  - **Stability control uses velocity** ( $\propto dZ/dt$ ) feedback provided by fast ( $\sim 60 \mu\text{sec}$ ) separate cpu;
  - **Position control provided as part of shape control** with longer cycle time ( $\sim 1 \text{ msec}$ ) cpu;
- **MIMO controllers on DIII-D must** be designed to provide good shape control and **minimize interaction with (fast) stability control loop**;
- **Good dynamic control therefore requires accurate model of vertical plasma response**:
  - Open loop vertical growth rate;
  - Effect of stabilization (by fast velocity feedback) on vertical plasma displacement mode.
- Effect of **vertical motion is also important in** order to produce **correct radial plasma response**;

## Rigid Current-Conserving Model Vertical Growth Rate Matches DIII-D VDE Experiments

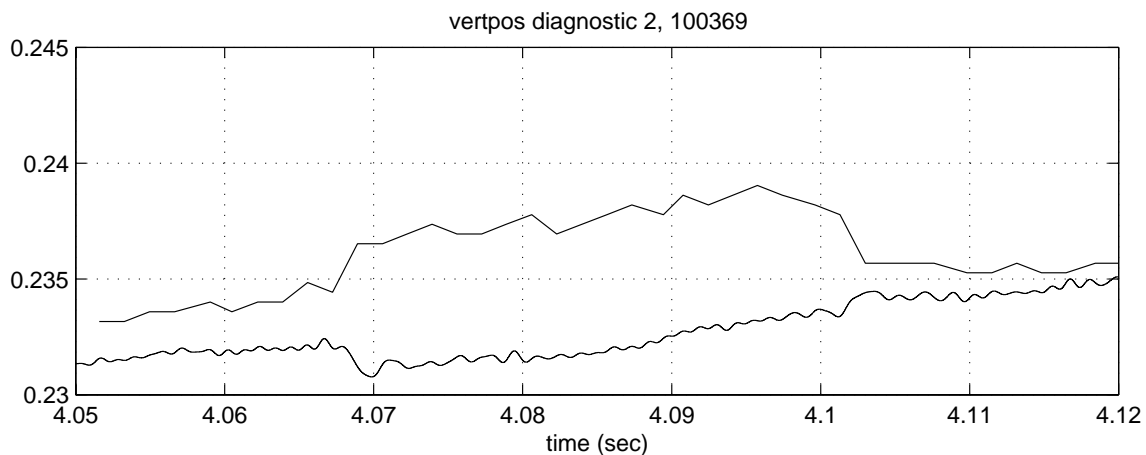
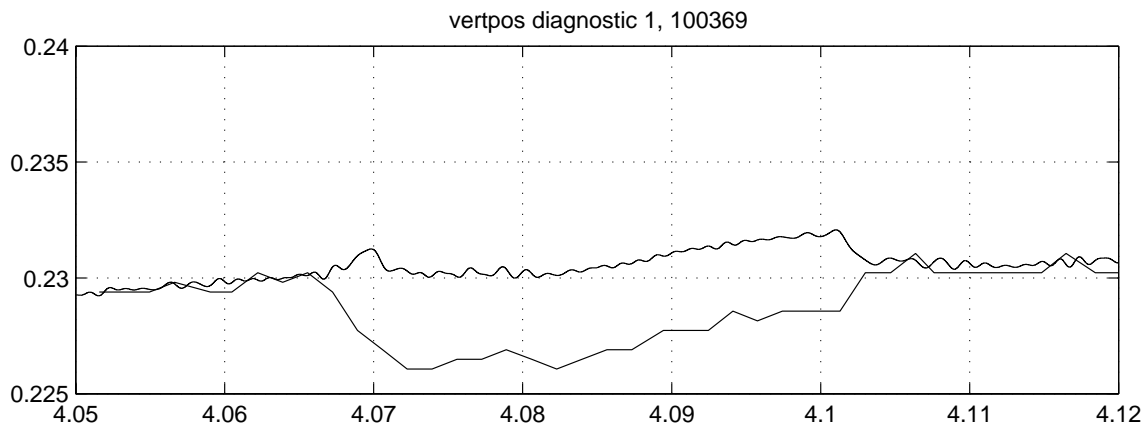
- Growth rates from current-conserving ( $\delta I_p=0$ ) rigid vertical model compared with experimentally measured VDE growth rates:



- Most VDE's yield vertical growth rate within ~10% of predicted value;
- Small subset of discharges produce VDE's with vertical growth rates differing by as much as 50% from predicted values:
  - Source of difference not yet well understood;
  - ~ 50% error in growth rate is tolerable for dynamic controller design in such cases.

## Incorrect Model of Vertical Instability Response Produces **Incorrect** B-Field Measurements

- Open loop simulation with **vertical instability removed** (to roughly model effect of fast vertical velocity control loop) **shows incorrect response of outboard poloidal field probes:**

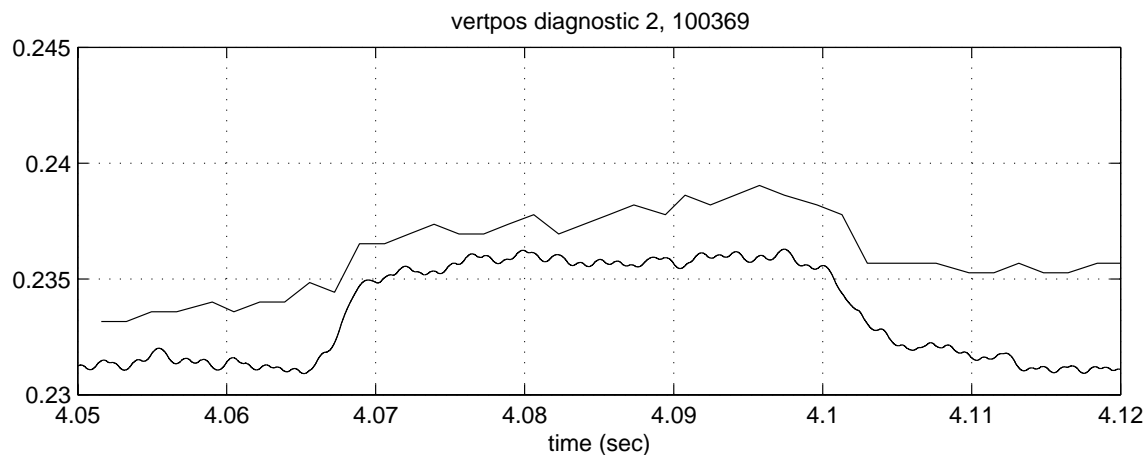
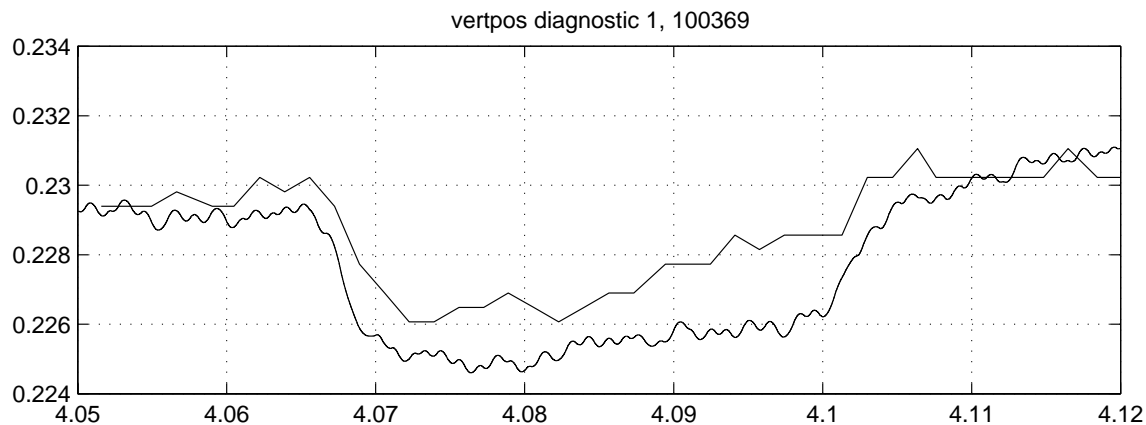


- Vertical model affects both vertical and *radial* response: cross-coupling terms are significant in up-down asymmetric plasmas (this case = Upper Single Null).



## Inclusion of Accurate Vertical Instability Mode in Plasma Model Produces **Correct** Probe Measurements

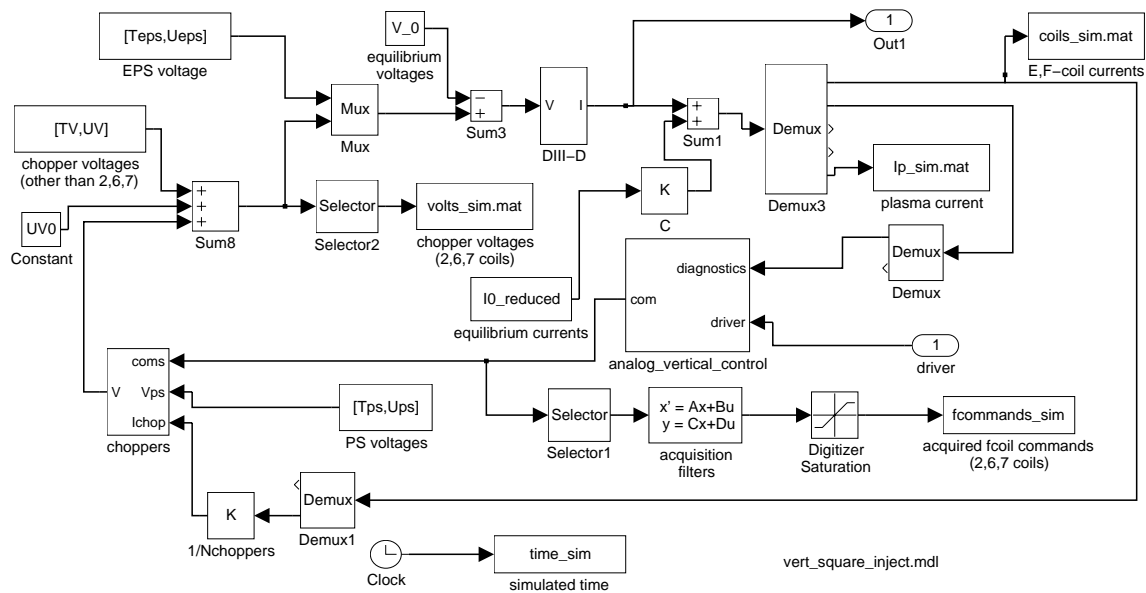
- Closed loop simulation with **vertical instability included** (and effect of fast velocity control loop correctly included in model) **shows correct response of outboard poloidal field probes:**



- Vertical model affects both vertical and *radial* response: cross-coupling terms are significant in up-down asymmetric plasmas (this case = Upper Single Null).

# Accurate Representation and Simulation of Vertical Response in Tokamaks is Complex Problem

- **Simulation/validation of intrinsically unstable open-loop plasma model is difficult:** little more than growth rate can be validated;



- **Simulation/validation of closed-loop response in DIII-D requires** accurate modeling of highly nonlinear **chopper power supplies** and **digital controller** action (as well as unstable plasma).

## CONCLUSIONS

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- **High dynamic accuracy** of performance in DIII-D isoflux boundary/X-point control **requires accurate plasma response models** for controller design.
- **“Minimal” analytically-based plasma models:**
  - **Are sufficient** to accurately describe plasma response for control;
  - **Have been developed and extensively validated** against DIII-D experiments.
  - **Have been used to design successful dynamic shape controllers for DIII-D.** [See poster JP1.33, this session]
- **Physics terms** important to plasma response have been identified:
  - Effect of plasma elongation;
  - Effects of variation in  $\beta_p$  and  $l_j$ ;
  - Cross-coupling of vertical and radial displacement;
  - Validated physics understanding allows **models to be extended to other operating regimes and devices** more reliably.

***PLASMA MODEL***

***VERTICAL MODE***

***MODEL VALIDATION  
EXAMPLES***