

**Abstract Submitted for the 54th Annual Meeting  
Division of Plasma Physics  
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Category Number and Subject: 5.6.2. DIII-D Tokamak

Theory       Experiment

**High  $\beta_N$  Steady state Scenario Development on DIII-D,\***  
C.T. Holcomb, M.J. Lanctot, *LLNL*; T.C. Luce, J.R. Ferron, R.J. Buttery, *GA*; J.M. Park, *ORNL*; F. Turco, J.M. Hanson, *Columbia U.*; M. Okabayashi, *PPPL* — On DIII-D, on- and off-axis neutral beams and electron cyclotron heating have expanded access to a wide range of  $q$ -profiles. Plasmas have been sustained with  $q_{\min}=1.3-2.5$  to evaluate the suitability for high  $\beta_N$ , high performance steady state operation. Nearly stationary plasmas were sustained for two current profile relaxation timescales (3 s), with  $q_{\min}=1.5$ ,  $\beta_N=3.5$ , and performance that projects to  $Q=5$  in ITER. The duration of the high  $\beta_N$  phase is limited only by the available NBI energy. Low-order tearing modes are absent and the predicted ideal-wall  $n=1$  kink  $\beta_N$  limit is  $>4$ . To achieve a steady state, higher  $\beta_N$  is needed to increase the bootstrap current. Higher  $q_{\min}$  decreases the required external current drive near the axis and can increase the stability  $\beta_N$  limit. Experiments to produce  $\beta_N=4-5$  and  $q_{\min}\geq 2$  with  $B_T=1.75-2$  T were limited to  $\beta_N<3.3$  by relatively low energy confinement ( $H_{89}<2$ ) rather than tearing modes. Low  $H_{89}$  is likely due to a combination of increased thermal transport at high  $q_{\min}$  (low poloidal flux), and depositing more power at larger radius. We will discuss upcoming experiments to achieve higher  $\beta_N$  and improved confinement.

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