

Advanced Inductive Plasmas With Low Torque Startup

by

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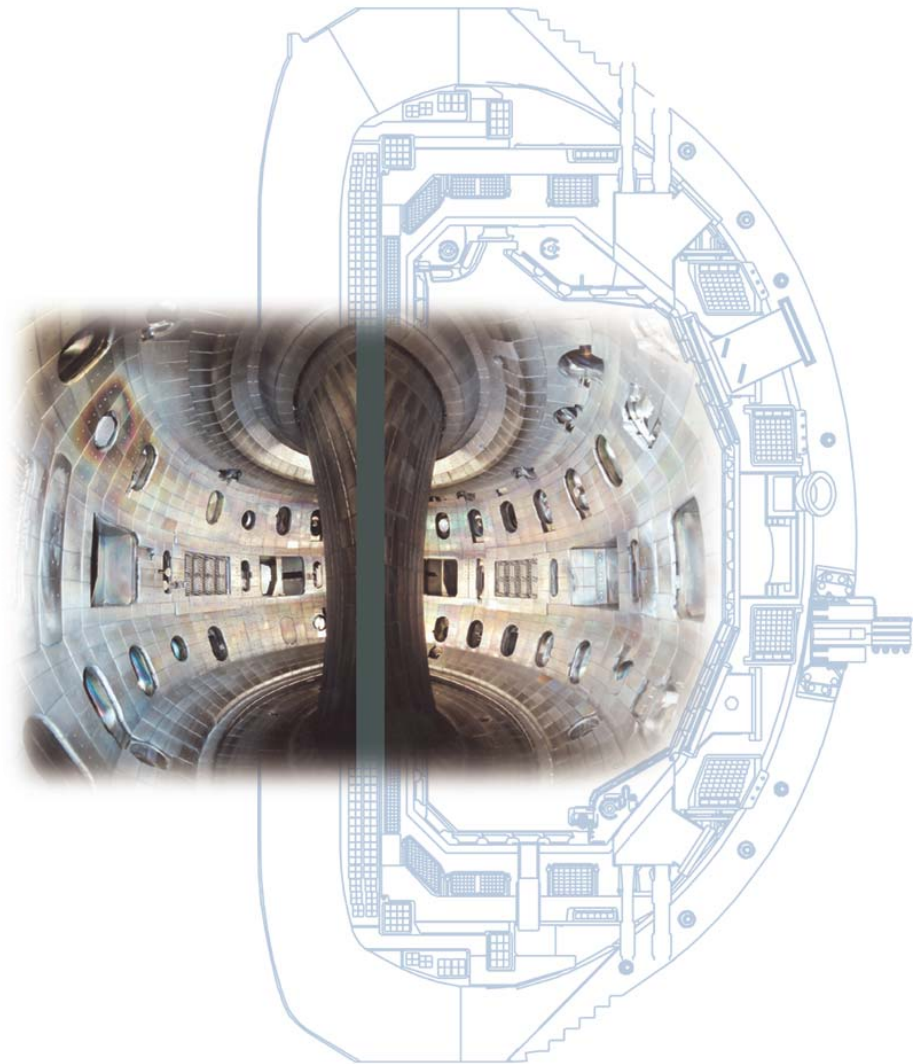
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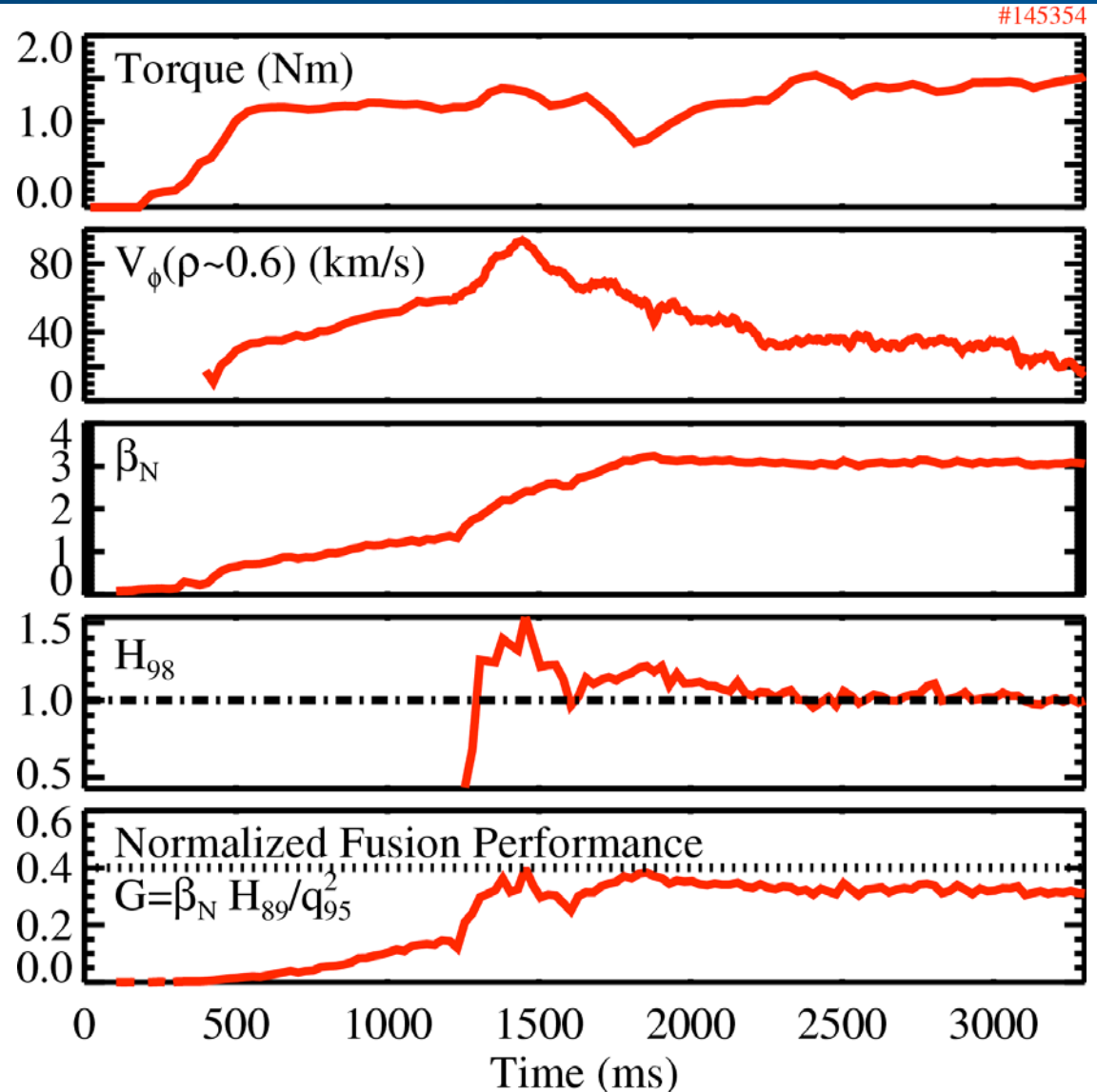


Motivation

- **Advanced inductive discharges offer an attractive alternative to the baseline scenario for ITER operation**
 - Higher confinement coupled with high β_N allows operation at lower plasma current while still reaching $Q=10$ mission
- **Goal is to develop plasmas with high fusion performance using rotation and relative momentum input comparable to that of ITER**
 - Improved confinement and stability often achieved by exploiting high levels of rotation
 - Due to the rapidly increasing moment of inertia with machine size compared with confinement time, it becomes more difficult to drive significant rotation in ITER and beyond
 - Key issue is determining access and performance characteristics at low rotation

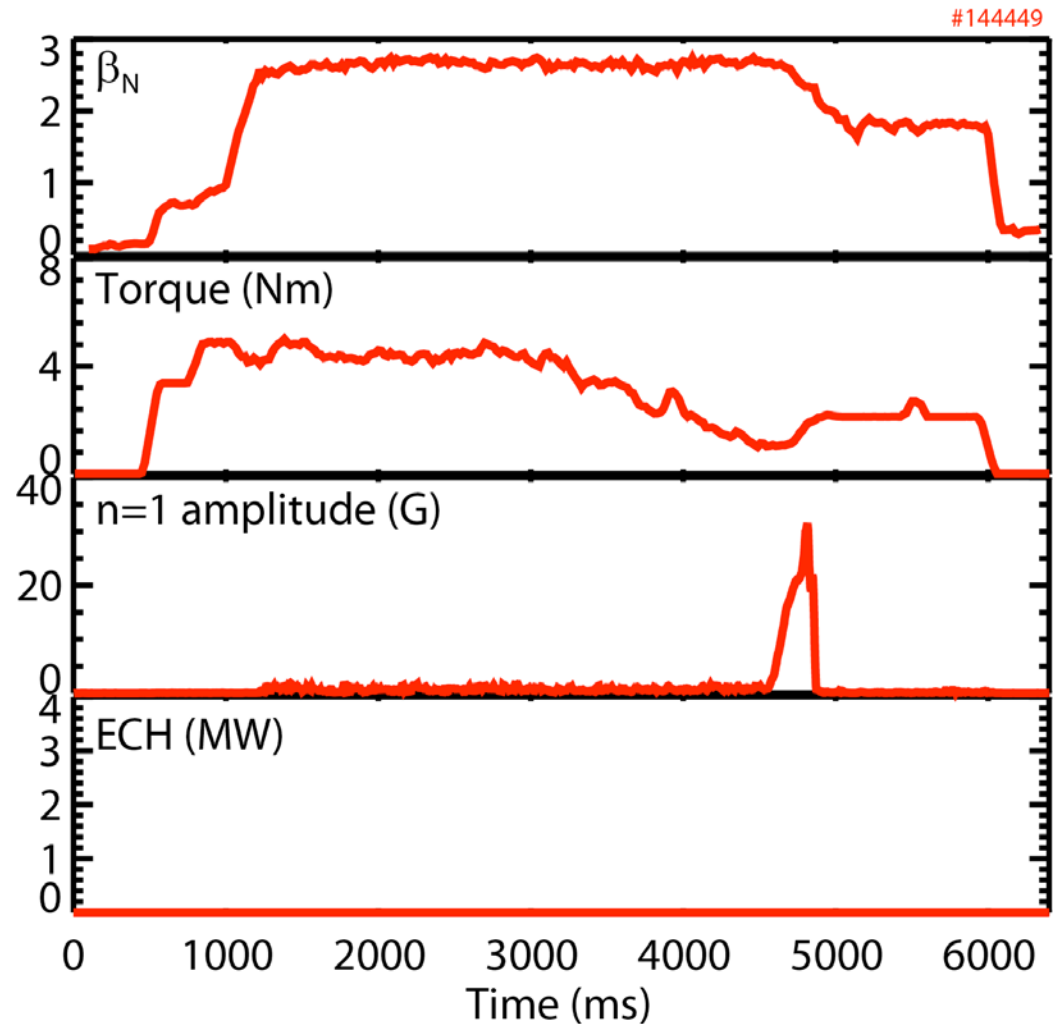
Advanced Inductive Discharge Approaching ITER Q=10 Equivalent Achieved with Low Torque Startup

- Torque ~ 1 Nm on DIII-D expected to drive similar rotation as ITER beams (~ 30 Nm)
- Does not utilize high torque in the startup
- $\beta_N \sim 3.1$, $H_{98} \sim 1$, $q_{95} \sim 4$ sustained for maximum duration of counter NBI
- Normalized fusion performance, $G = \beta_N H_{89} / q_{95}^2 \sim 0.35$
 - $G \sim 0.42 \rightarrow$ ITER $Q=10$



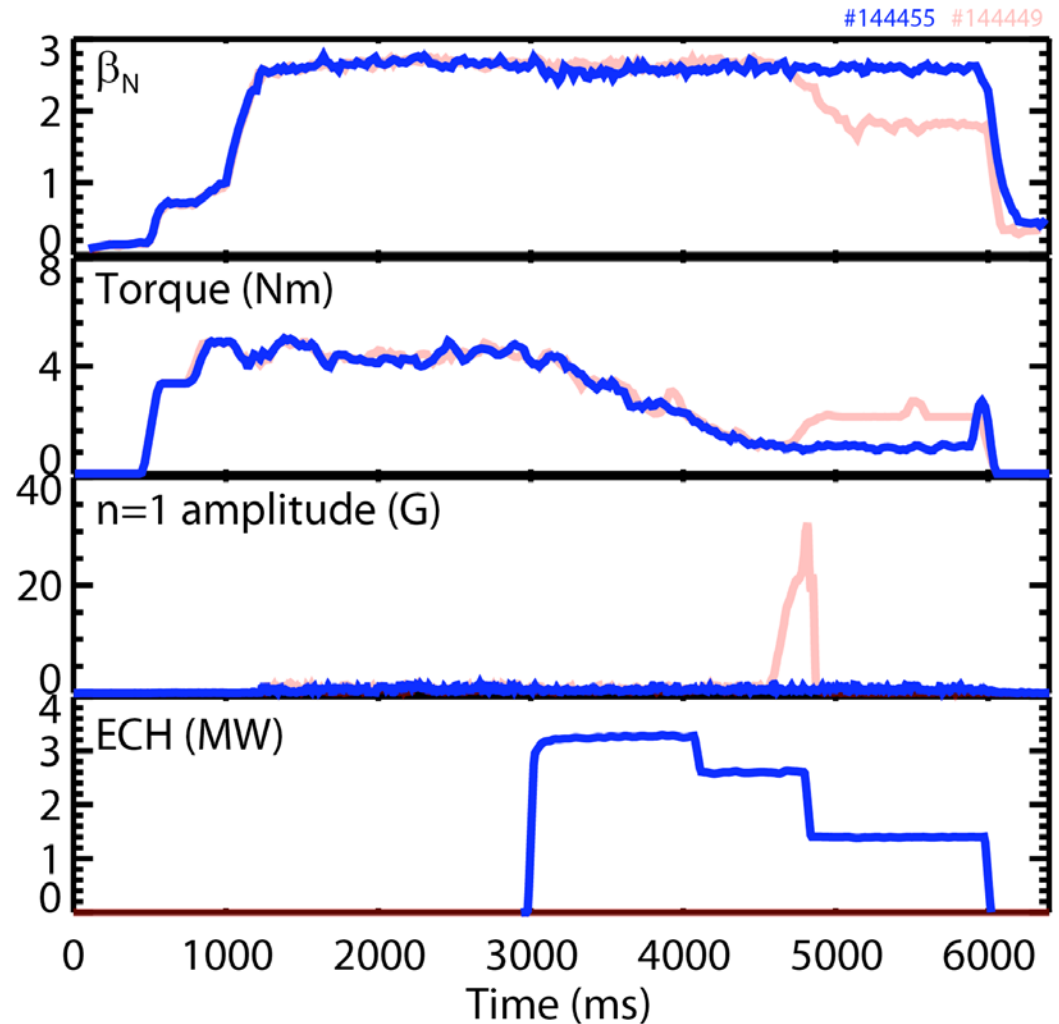
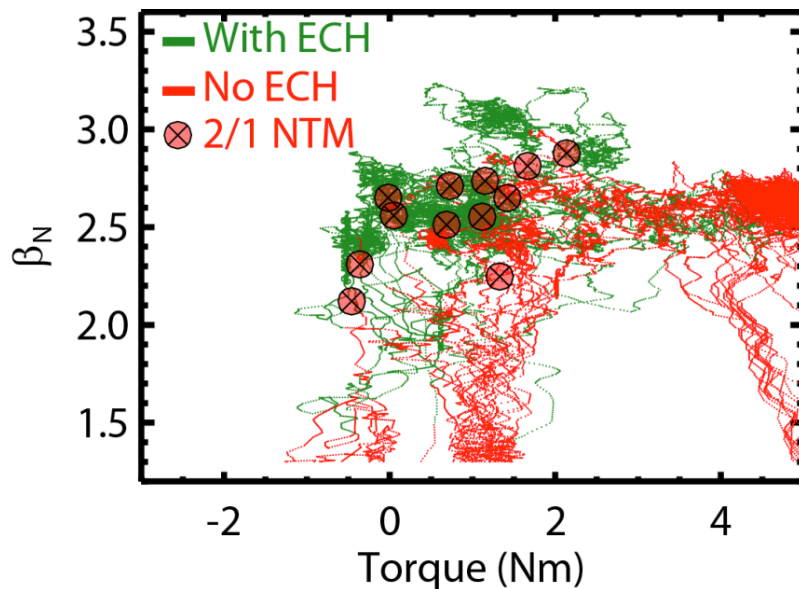
At Reduced Torque, Plasma Becomes Increasingly Susceptible to 2/1 Neoclassical Tearing Modes

- Generally slow and lock, terminating high performance phase
- Does not appear a result of uncorrected error fields
 - 2/1 NTM occurred at the same torque level even with Improved error field correction



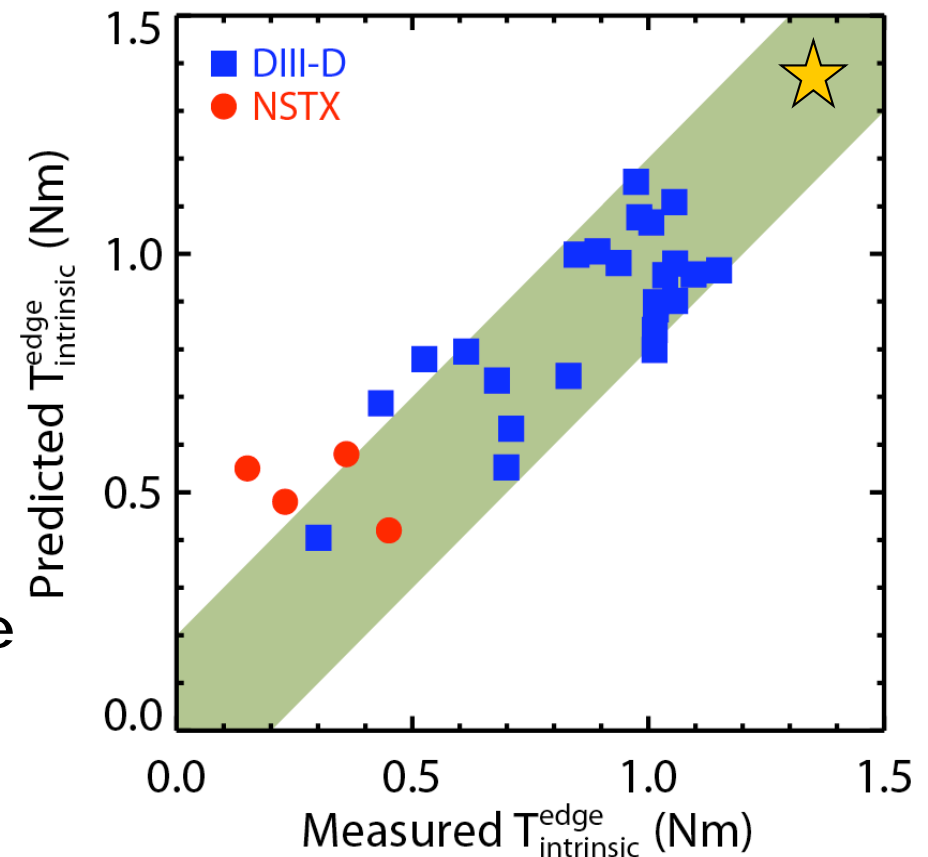
Use of Electron Cyclotron Heating Beneficial for Tearing Mode Control at Low Torque Regimes

- Application of ECH generally results in suppression of such modes
- Typically configured to drive current near $q=2$ surface
 - But actual current drive may not be essential here



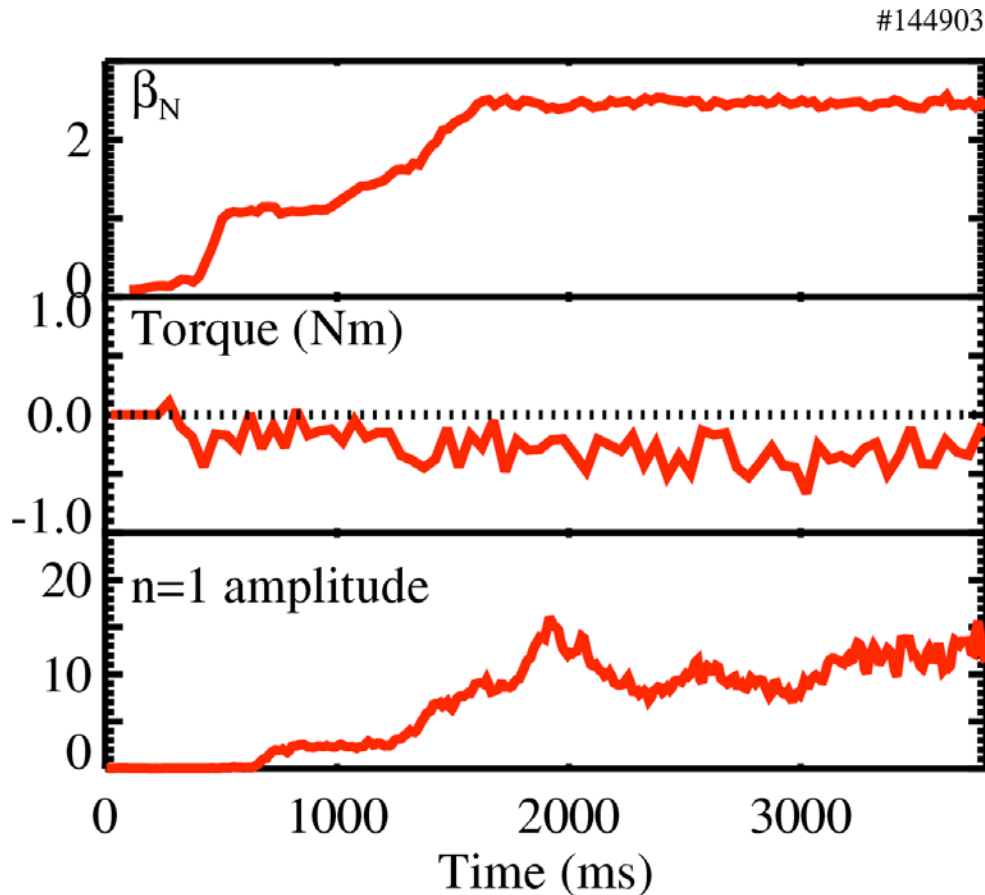
Intrinsic Drive in Low Torque AI Plasmas Closely Matches Previously Determined Scaling

- An excellent predictor of edge intrinsic torque has been established for DIII-D H-modes
 - Empirically includes physics associated with turbulent Reynolds stress and thermal ion orbit loss
[Solomon et al., *Nucl. Fusion* (2011)]
- New advanced inductive discharges show notable levels of intrinsic torque
 - Significantly exceed range of data set used to construct scaling
- If holds for ITER scenario 2, compute intrinsic torque of only 2 Nm
 - Much smaller than even NBI torque



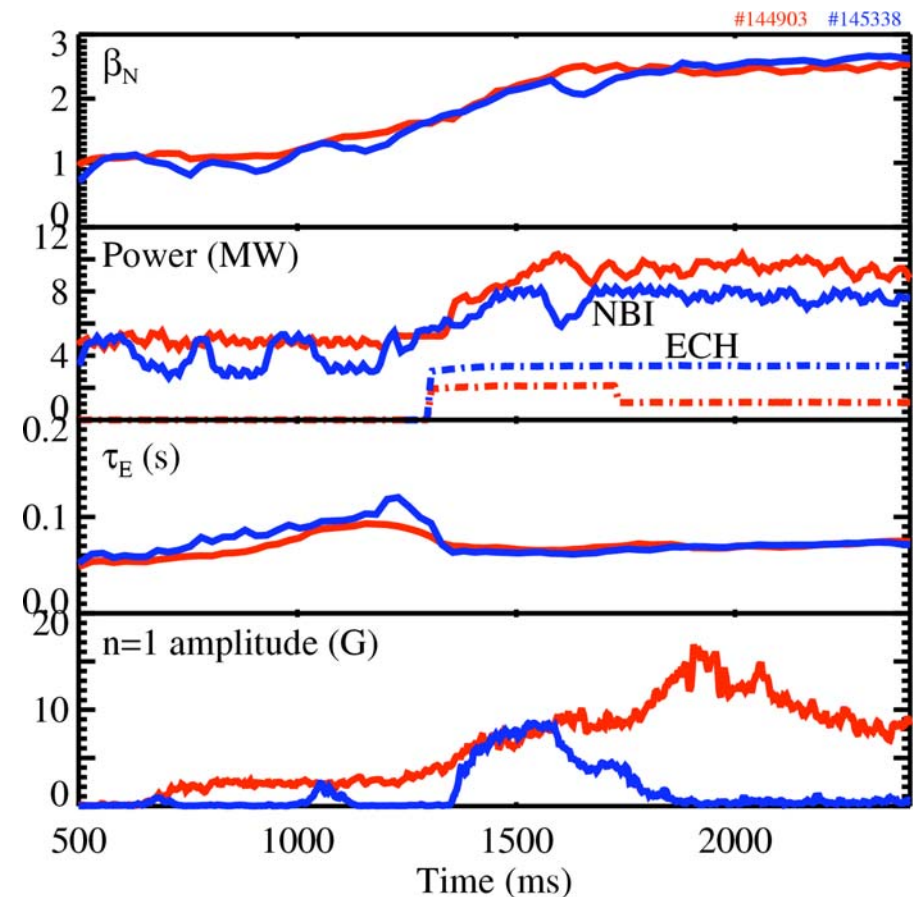
Advanced Inductive Discharges Have Been Run With Zero Net Input Torque For Full Discharge

- Using only modest EC power (~ 1 MW), 2/1 NTM is reduced in amplitude sufficiently to allow operation at $\beta_N \sim 2.5$
 - Maximum achievable with available heating power in these conditions



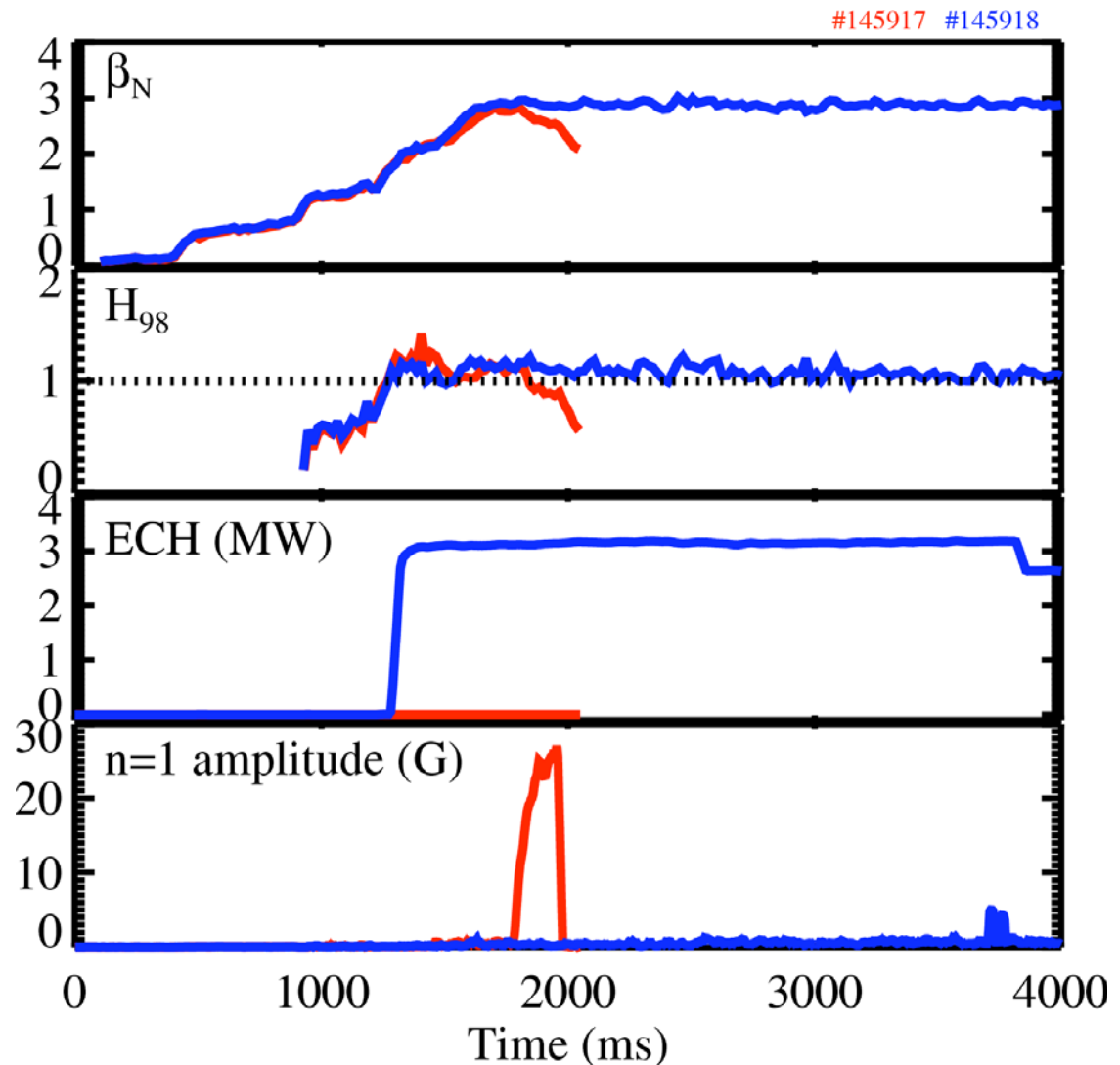
Advanced Inductive Discharges Have Been Run with Zero Net Input Torque For Full Discharge

- Using only modest EC power (~ 1 MW), 2/1 NTM is reduced in amplitude sufficiently to allow operation at $\beta_N \sim 2.5$
 - Maximum achievable with available heating power in these conditions
- Additional EC power allows complete stabilization of 2/1 mode
 - Energy confinement time the same
 - Similar β_N for similar combined NBI+ECH
- Significant cost to confinement for using ECCD aimed at $q=2$
 - Avoids disruptive locked mode
 - But gain is not notably improved



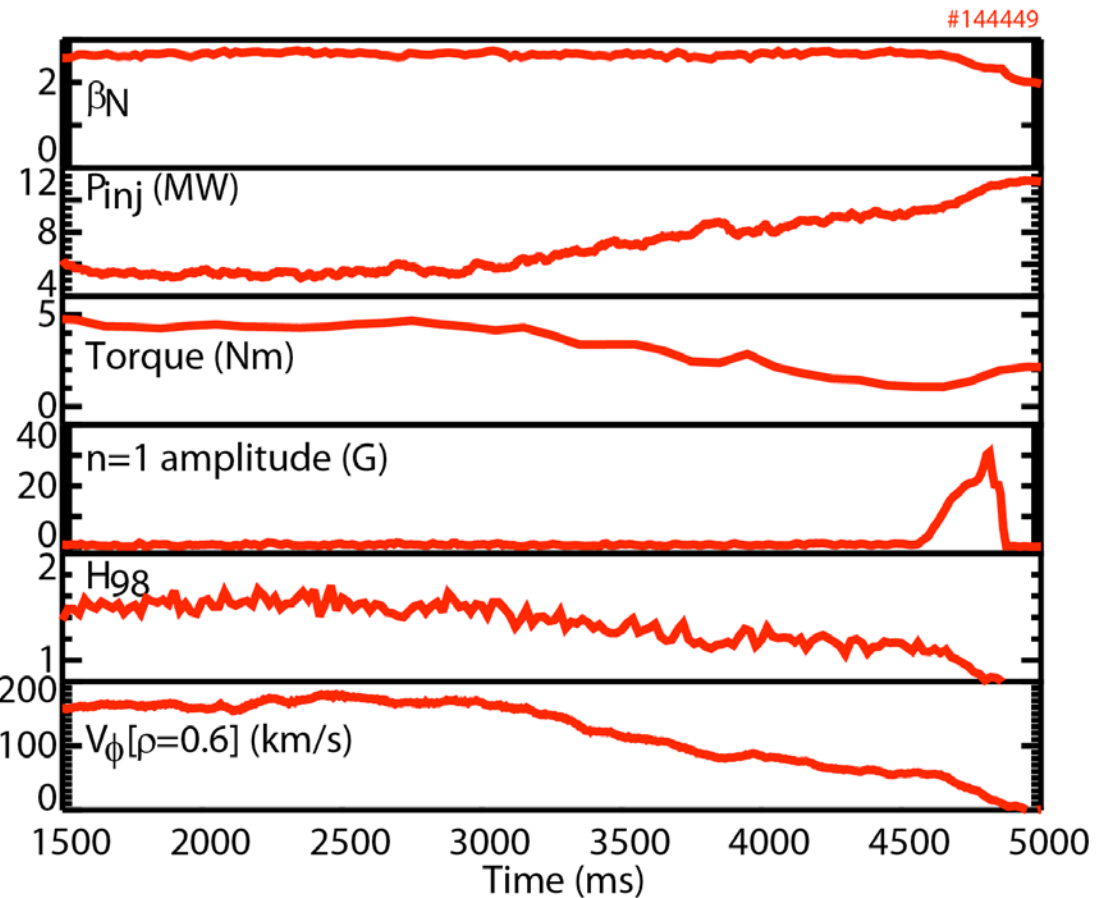
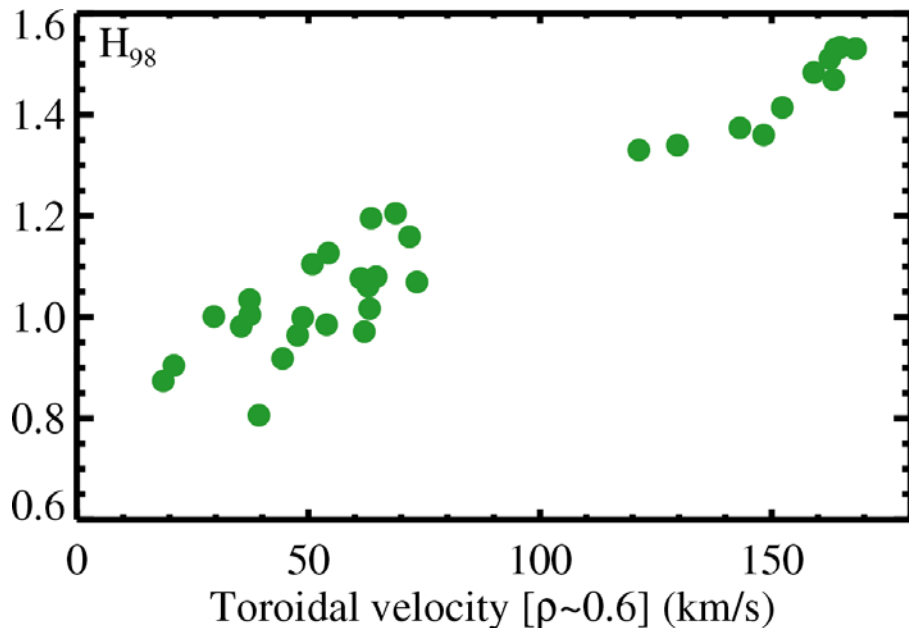
Advanced Inductive Plasmas with High $\beta_N \sim 3$ Achieved Using More Central ECH (no $q=2$ ECCD)

- Torque ~ 1 Nm, $H_{98} \sim 1$
- In this case, $\rho_{ECH} \sim 0.3$ (cf $\rho_{q=2} \sim 0.6$) and configured for heating without current drive
- Insensitivity to deposition location or heating vs current drive suggests different mechanism than standard NTM stabilization



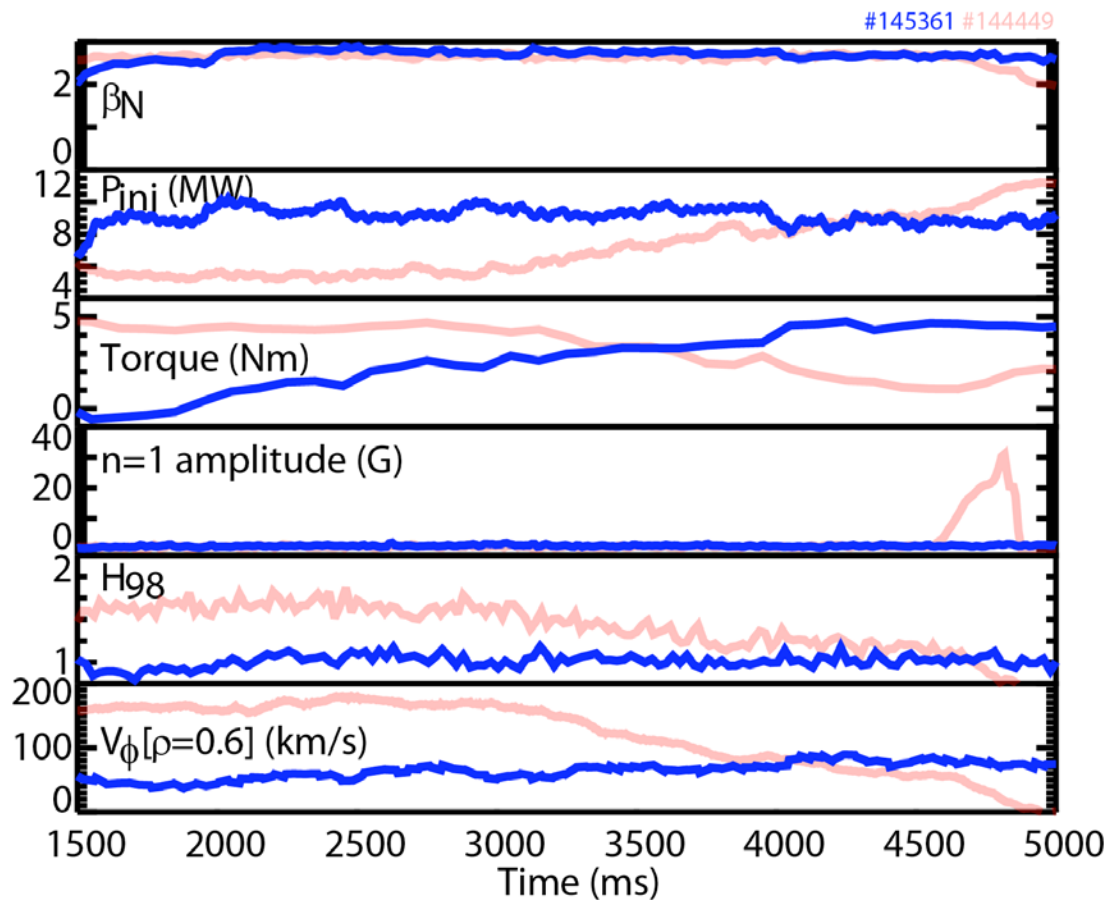
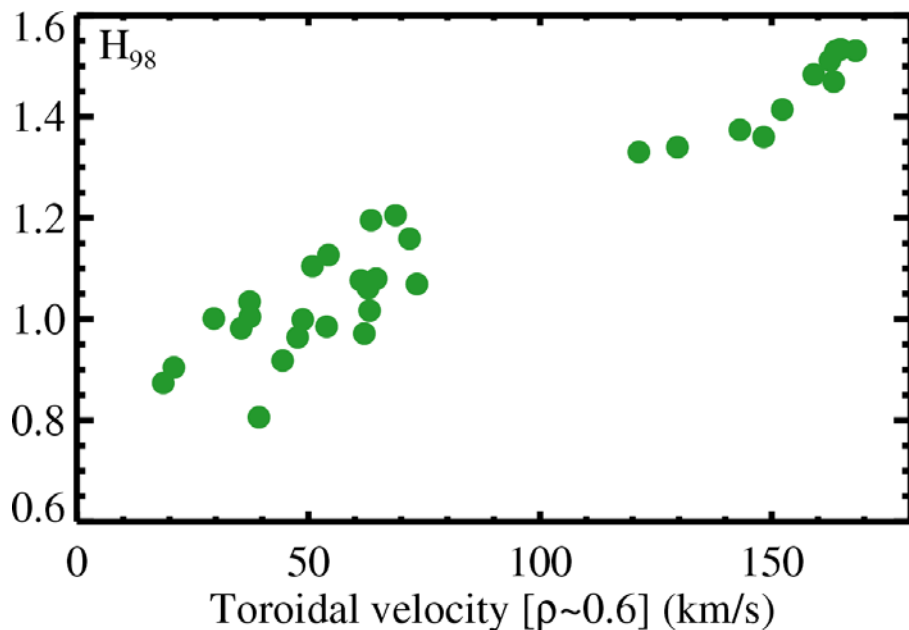
Significant Reduction in Confinement Observed as Torque Is Reduced

- For fixed β_N , power demand increase $\sim 70\%$ at low torque
- H_{98} reduced from >1.5 to approx 1.0
- No obvious evolution in q-profile
- Similar to previously reported results [Politzer, Nucl. Fusion **48**, 075001 (2008)]



Higher Confinement Conditions Are Not Simply Recoverable by Ramping Up Torque

- Relatively difficult to spin the plasma up from low rotation state
 - Torque increased a factor of 4, rotation barely increased a factor of 2
- Path-dependence in achieving high H factor
- Interesting case to study with transport models



Summary

- **Advanced inductive discharges with normalized fusion performance $G \sim 0.35$ approaching ITER $Q=10$ equivalent have been demonstrated with torque ~ 1 Nm and $q_{95} \sim 4$**
- **Mechanism by which ECH assists low torque operation not yet clear**
 - Apparently different than replacement of helical deficit in bootstrap current associated with island
 - ECH leading to direct modification of classical Δ' stability parameter?
- **High β_N AI discharges exhibit significant levels of intrinsic torque**
 - As expected from previously determined scaling of DIII-D intrinsic drive
- **Significant reduction in confinement observed at reduced rotation**
 - Want to find ways to recover performance to further enhance attractiveness of scenario for low rotation operation