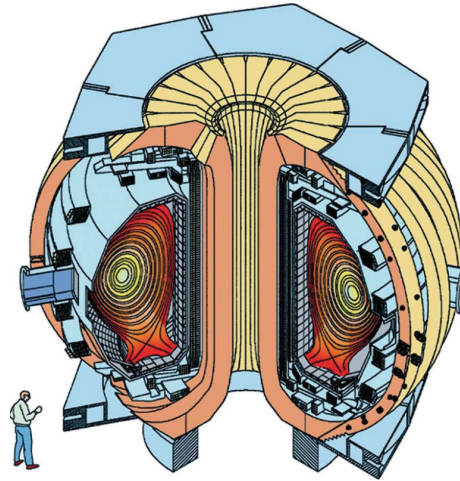


ITER Test Blanket Module Error Field Simulation Experiments

by
M.J. Schaffer

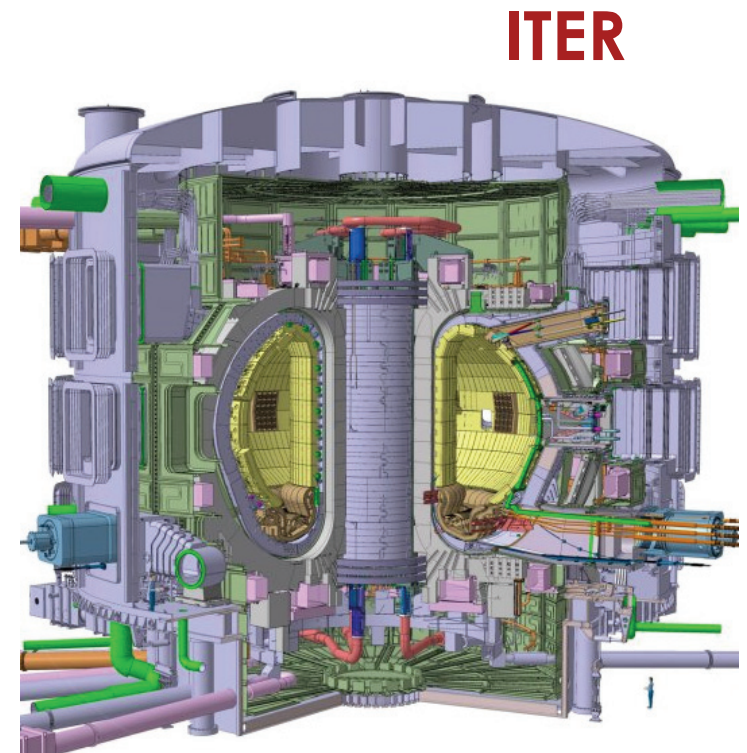


DIII-D

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Paper B13.00001



ITER

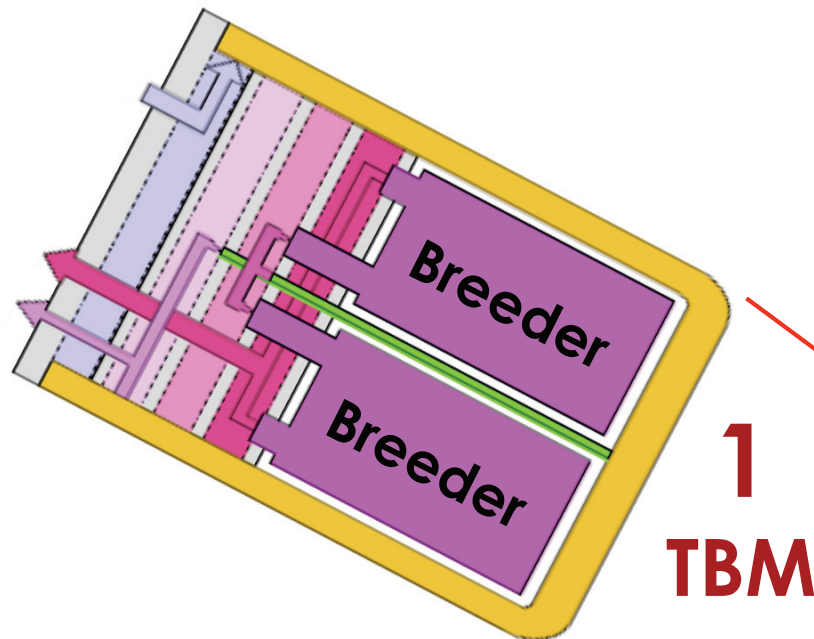
A Large International Team Participated

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X. Gao⁵, A.M. Garofalo¹, D.A. Gates⁶, C.M. Greenfield¹, W.W. Heidbrink⁷, G.J. Kramer⁶, R.J. La Haye¹,
S. Liu⁵, A. Loarte², M. F. F. Nave⁸, T.H. Osborne¹, N. Oyama⁹, J-K. Park⁶,
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R. Srinivasan¹⁰, T.S. Taylor¹, M.R. Wade¹, K.-I. You¹⁹, L. Zeng¹⁷, Y. Zhu⁷, and the DIII-D Team¹

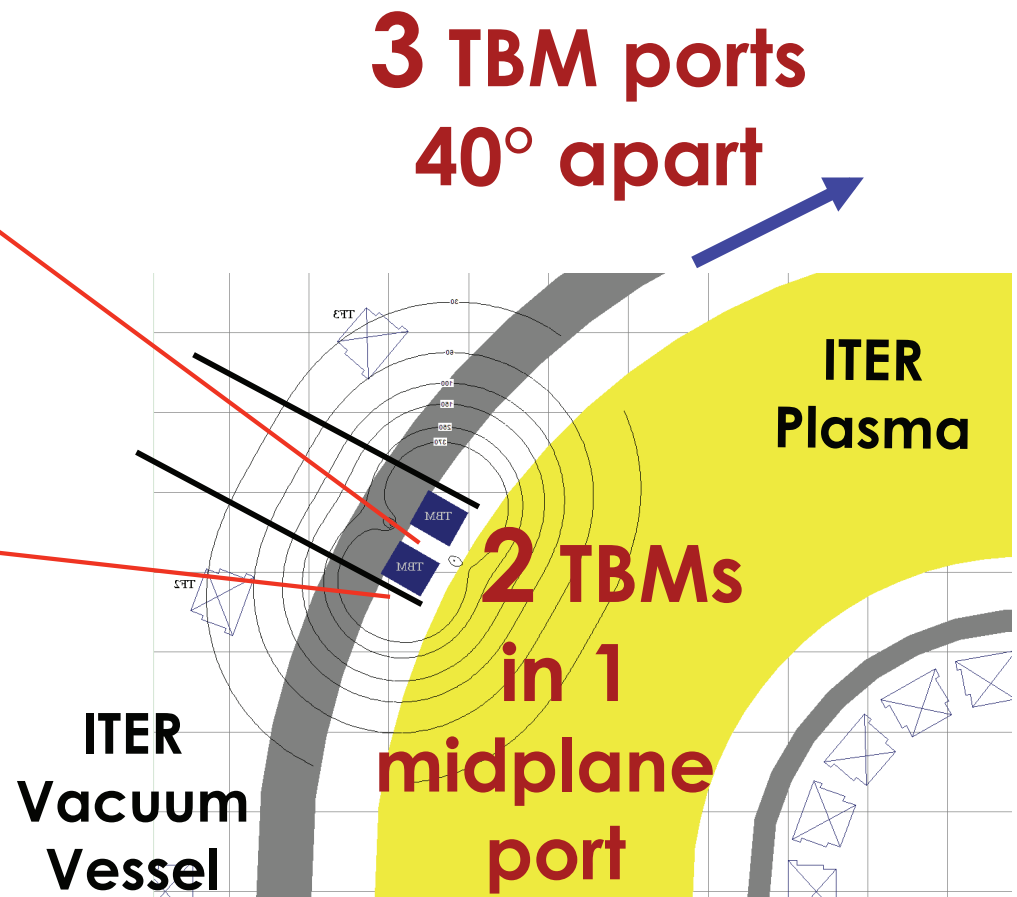
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Concern Raised About the Impact of Ferromagnetic Test Blanket Modules (TBM) on ITER Performance

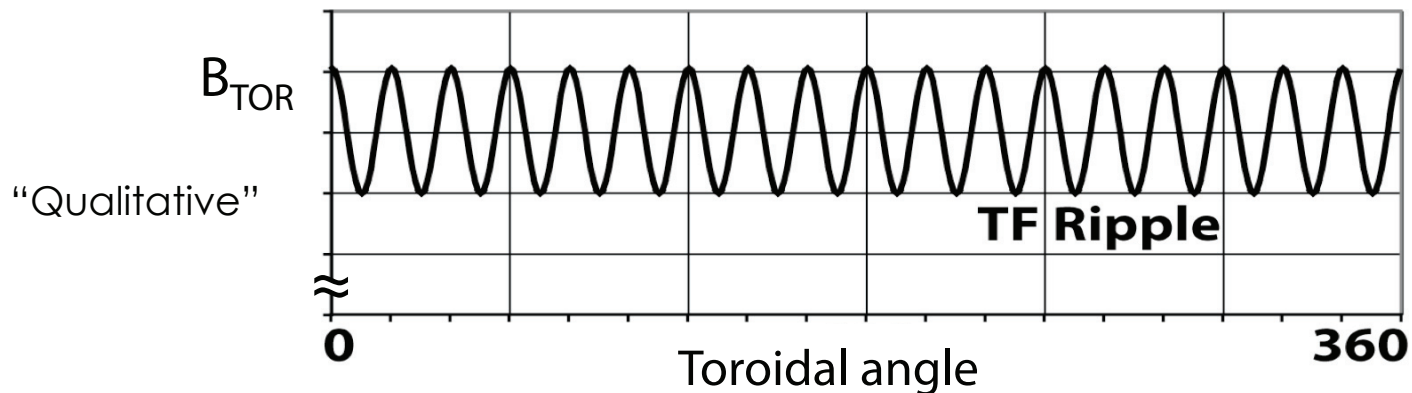


- Has ~ 1 ton of ferromagnetic steel
 - Thermal and neutronic ITER technology missions
 - B-field bumps



Toroidal Field Ripple Has Undesirable Effects on Tokamak Plasmas (Fast Ion Loss, Confinement Loss...)

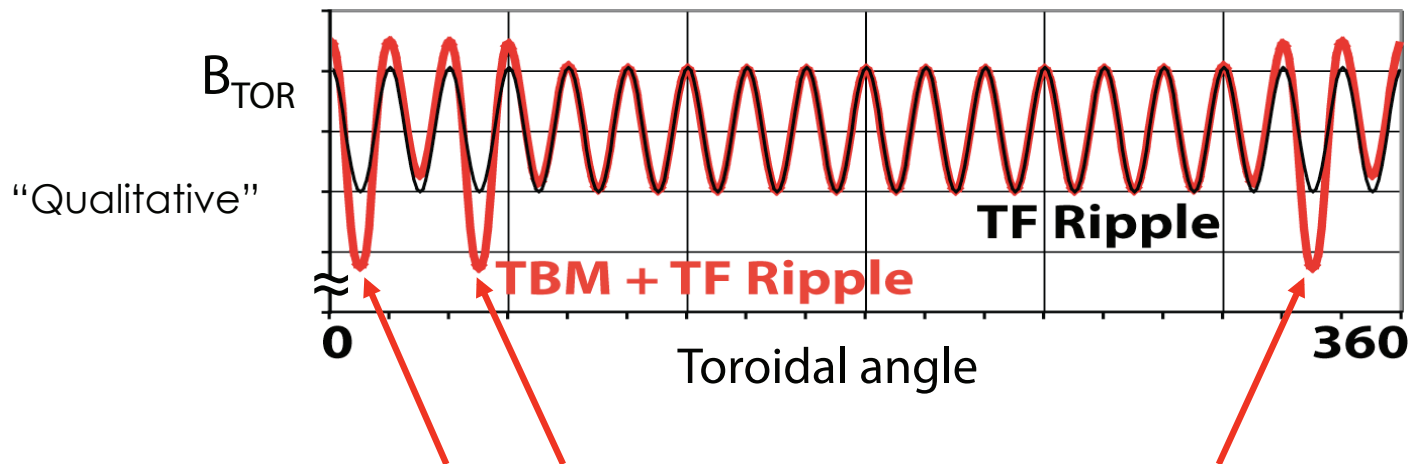
- Common magnetic field ripples from discrete toroidal field coils are uniformly periodic



- DIII-D matched the ITER TF-coil ripple amplitude for most TBM experiments
 - 0.35 ~ 0.40% for both

TBMs Complicate Ripple Geometry

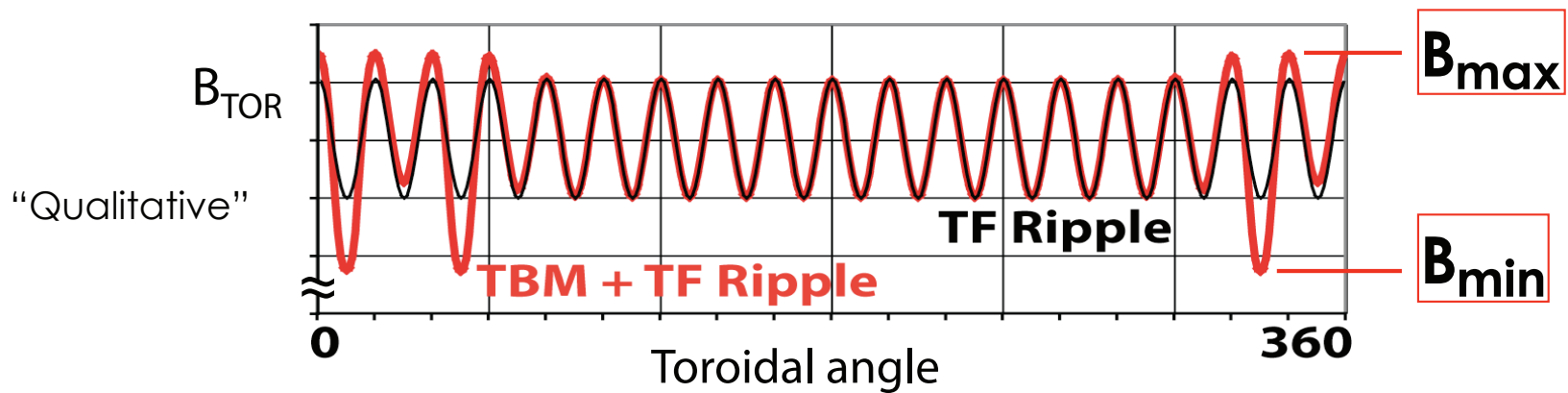
- Larger perturbations
- Broken periodicity



- **Magnetized TBM steel makes 3 local “dips”**
- **Cannot predict plasma consequences, NEED EXPERIMENTS**

Use the Combined “Local Ripple” $\equiv \delta$

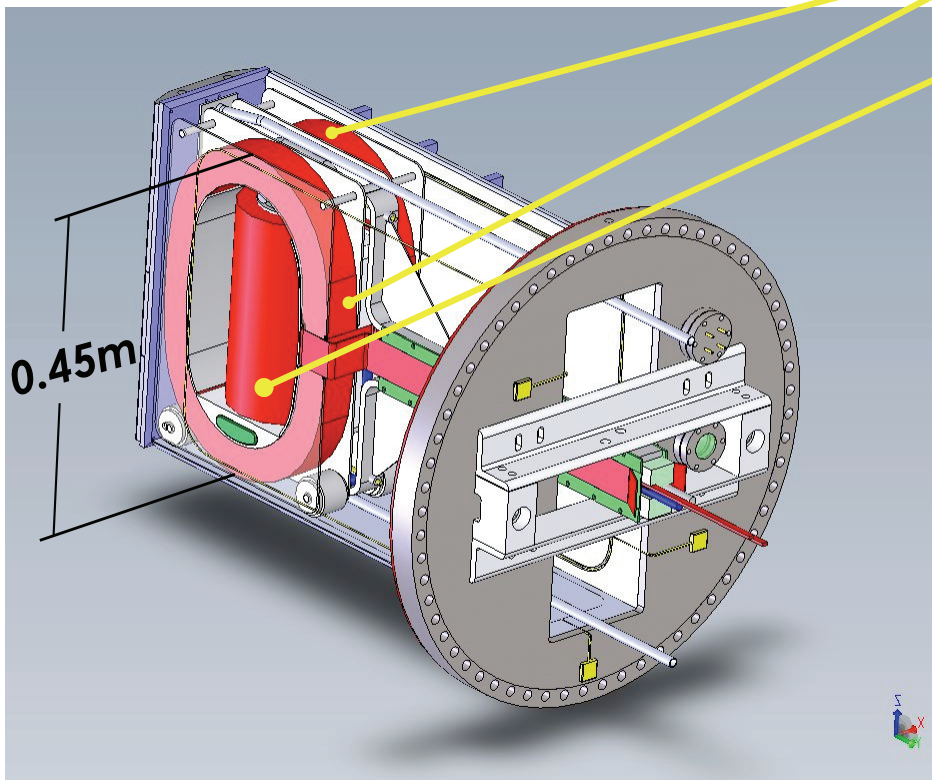
$$\delta = \frac{B_{\max} - B_{\min}}{B_{\max} + B_{\min}}$$



- ITER expects to have $\delta \approx 1.2\%$

We Designed a Mock-up of the TBM Field to Measure Its Effects on DIII-D Plasmas for ITER

Mock-up Approximates the Magnetization within One ITER TBM Port



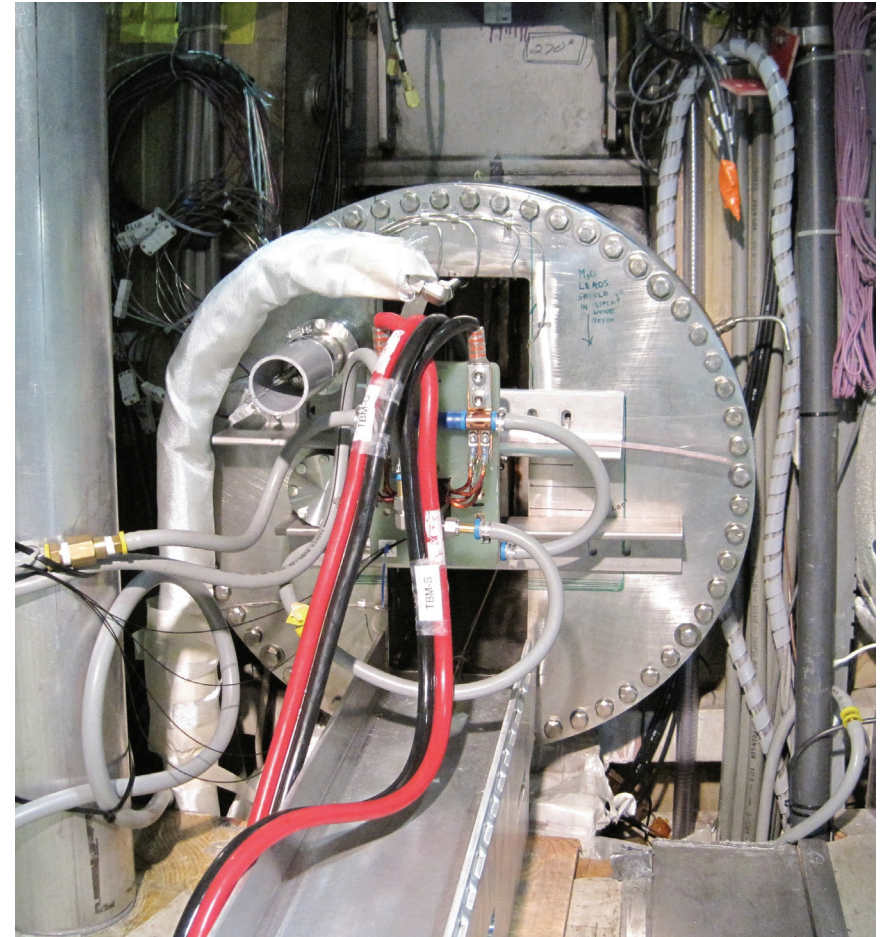
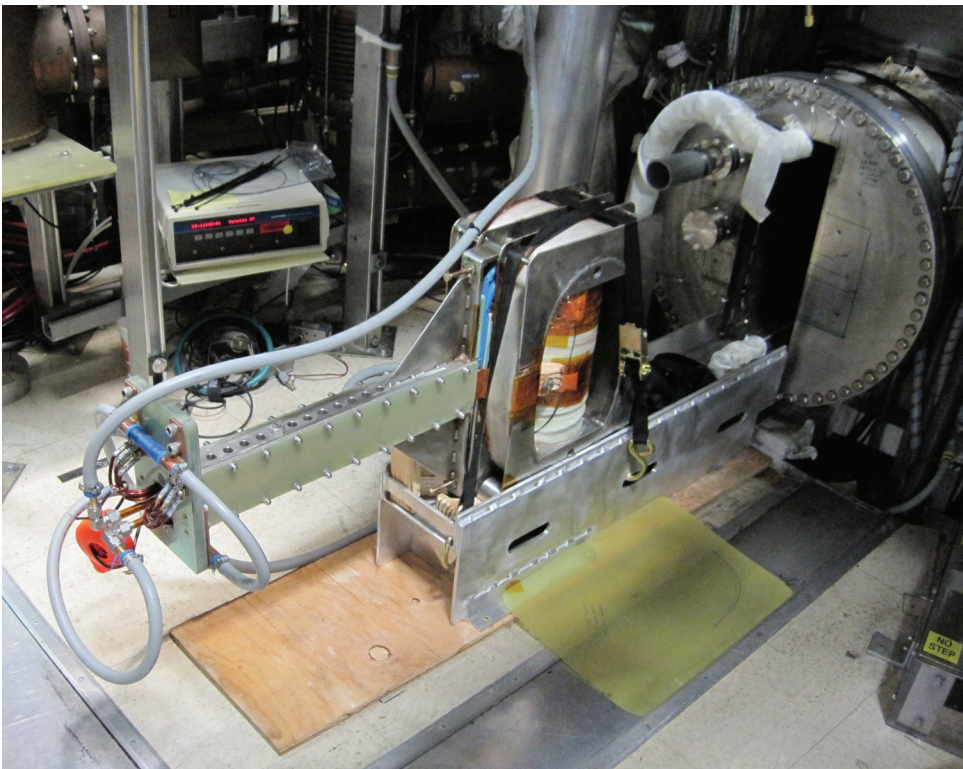
Racetrack coils \leftrightarrow TBM M_{TOR}

Vertical solenoid \leftrightarrow TBM M_{POL}

- **Matches ITER TBM field well**
 - M_{POL}/M_{TOR} vary to match q
- **Movable, $\Delta R = 0.28$ m**
 - 1.0 m in ITER
- **DIII-D local ripple ~ 3 x ITER local ripple**
 - Same surface-average ripple amplitude as 3 ITER TBM ports

TBM Mock-up at its DIII-D Midplane Port

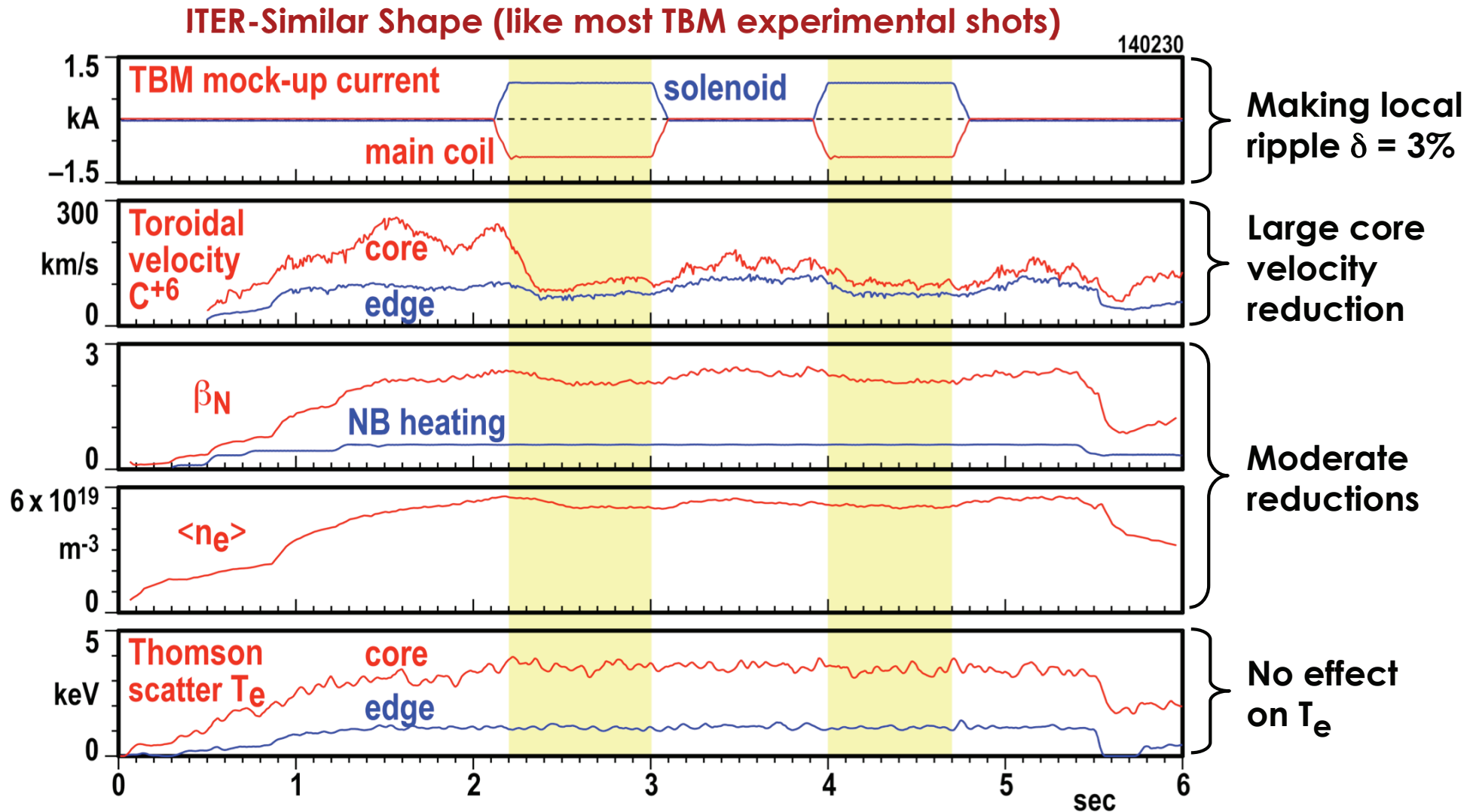
**Mock-up secured in its channel,
cooling water attached**



Mock-up rolled into port

Greatest Effect of TBM Field Was on Toroidal Velocity

– β , n_e and T_e Were Much Less Affected



Experimental Results Showed That Most Other TBM Mock-up Effects Were Small

Little or No Effects on:

Plasma Startup

L-mode Confinement

L-to-H Transition [P Gohil, UO4.xxxx9]

H-mode Pedestal (except n_e)

ELM Characteristics

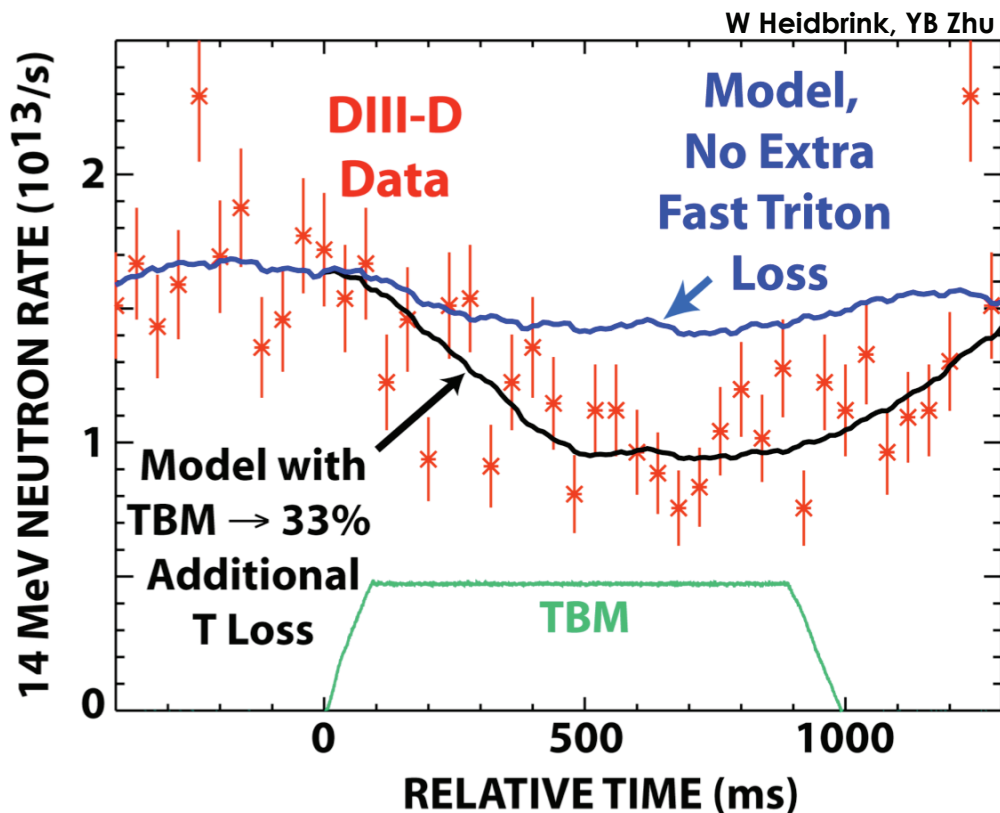
ELM Control by RMP

Thermal Loss to Bumper Limiters

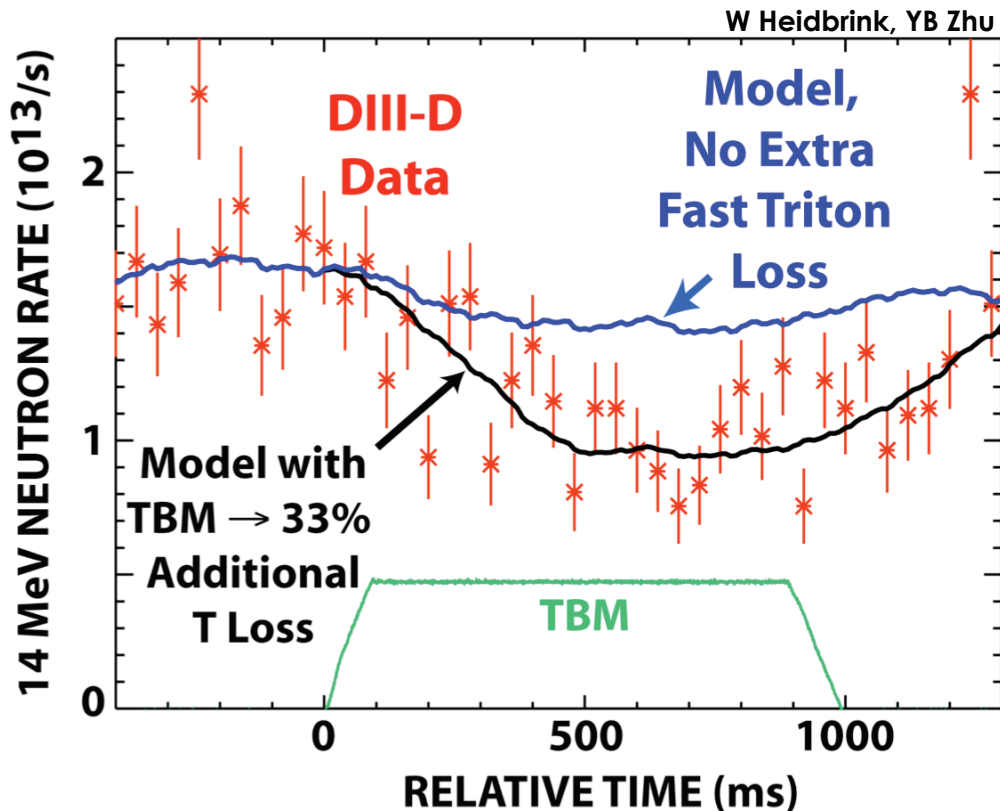
Divertor Power Distribution

Experimental Results Provide Validation of Fast Ion Loss Models for TBM-like Error Field

- Blue curve if TBM caused no loss of 1-MeV DD fusion tritons



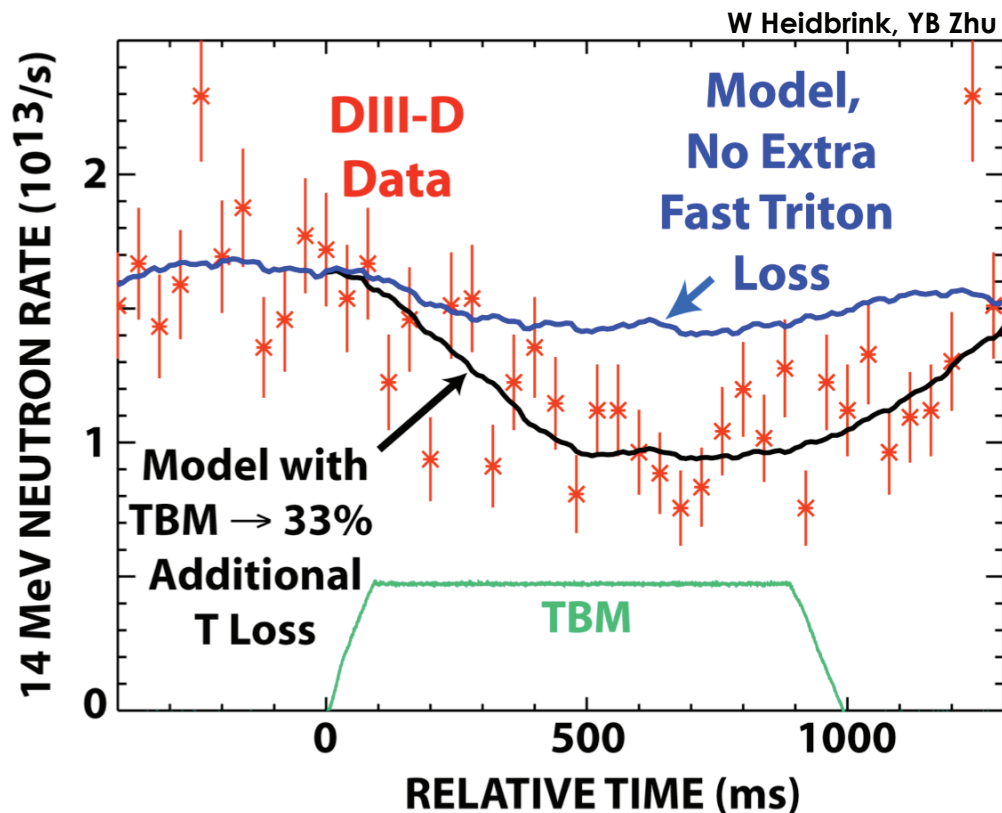
Experimental Results Provide Validation of Fast Ion Loss Models for TBM-like Error Field



- Blue curve if TBM caused no loss of 1-MeV DD fusion tritons
- Simple model gives 33% additional loss of T ions
 - T orbits large in DIII-D
 - Model agrees with data

[YB Zhu, UP9.xxx57]

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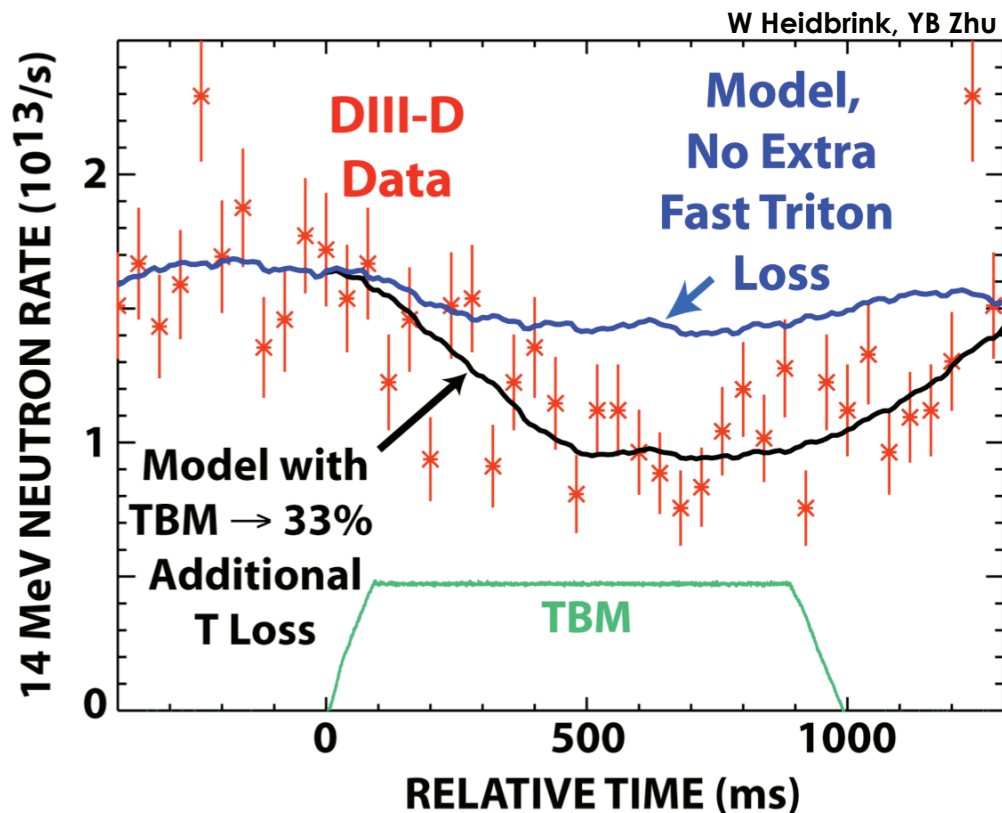


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[G Kramer, XO4.xxxx4]

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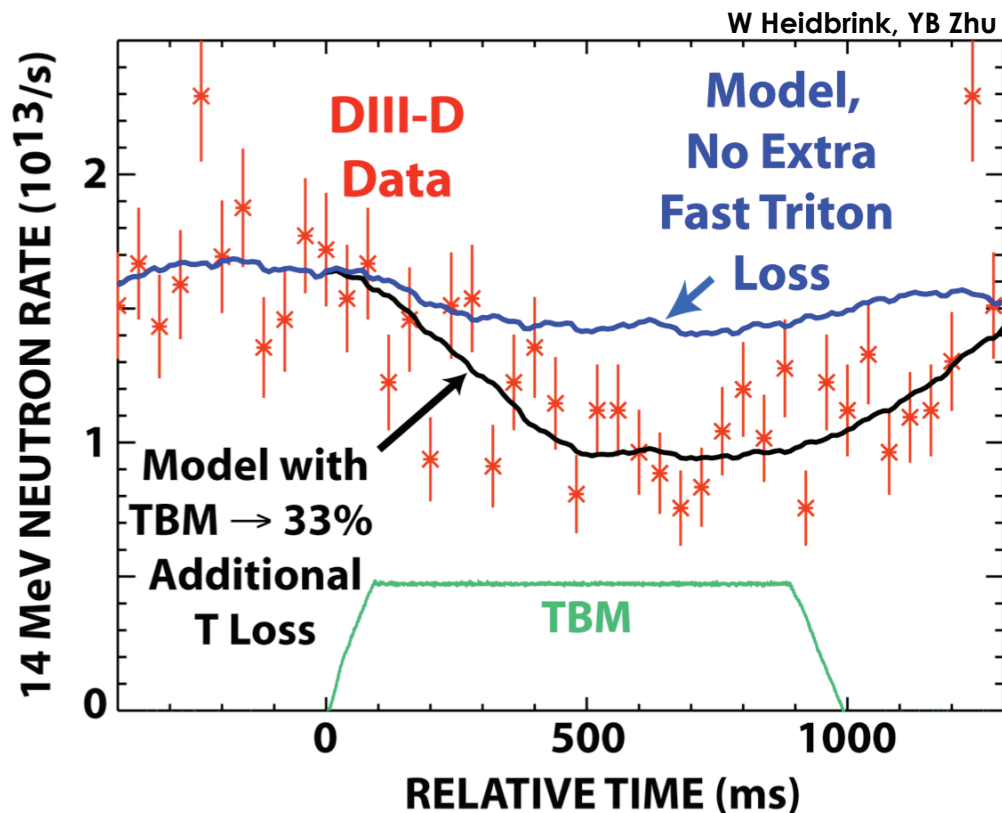


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- Four fast-ion codes account for measured heating of TBM mock-up protection tiles
 - SPIRAL, ASCOT, OFMC, DELTA5D

[YB Zhu, UP9.xxx57]

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Experimental Results Provide Validation of Fast Ion Loss Models for TBM-like Error Field



- Blue curve if TBM caused no loss of 1-MeV DD fusion tritons
- Simple model gives 33% additional loss of T ions
 - T orbits large in DIII-D
 - Model agrees with data
- SPIRAL full orbit code also computes 33% T loss
- Three fast-ion codes account for measured heating of TBM mock-up protection tiles (in progress)
 - SPIRAL, ASCOT, OFMC, DELTA5D
- These and similar models predict small, acceptable fast ion losses for ITER

[YB Zhu, UP9.xxx57]

[G Kramer, XO4.xxxx4]

[D Spong, BI3.xxxx6]



A Few TBM Error Field Effects May Be of Concern to ITER, but ... They May Also be Correctable

Of Concern to the ITER High-Gain Mission*

Locked modes

Braking of plasma
rotation

Reduction of H-mode
confinement

* if scaling is unfavorable
and affects fusion gain

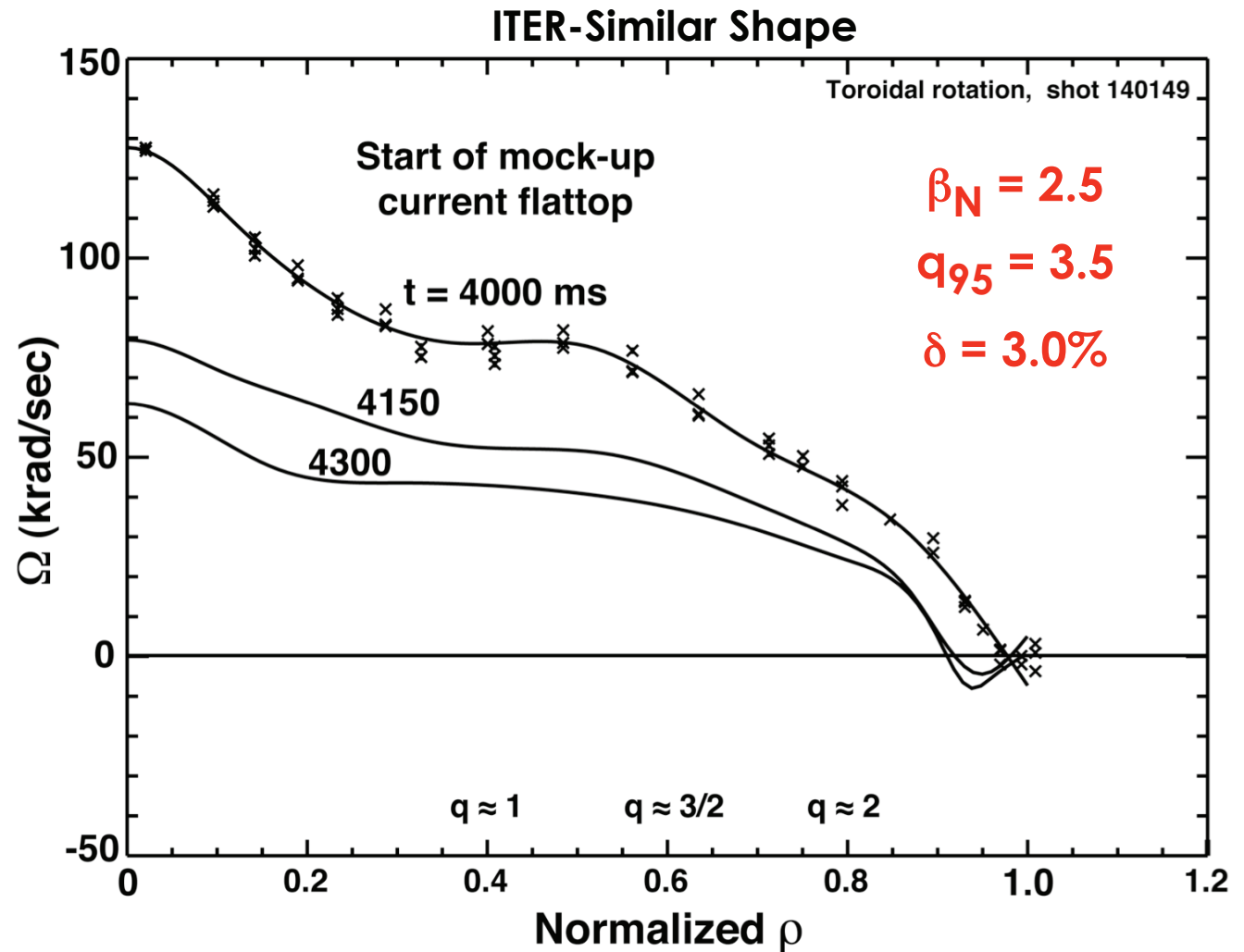
They May Be Correctable:

- n=1 error compensation eliminated TBM contribution to locking in L-mode experiments
- Numerical model predicts n=1 will reduce braking in H-mode plasma
- Past experience links H-mode confinement and rotation

Example: High Mock-up Field Locked Co-injected H-mode Plasma Rotation at High β

- **Plasma slowed**

- To $\sim 50\%$ of initial rotation in 300 ms
- $\Delta\Omega/\Omega \approx$ constant across plasma
- But not quite in steady torque balance



Example: High Mock-up Field Locked Co-injected H-mode Plasma Rotation at High β

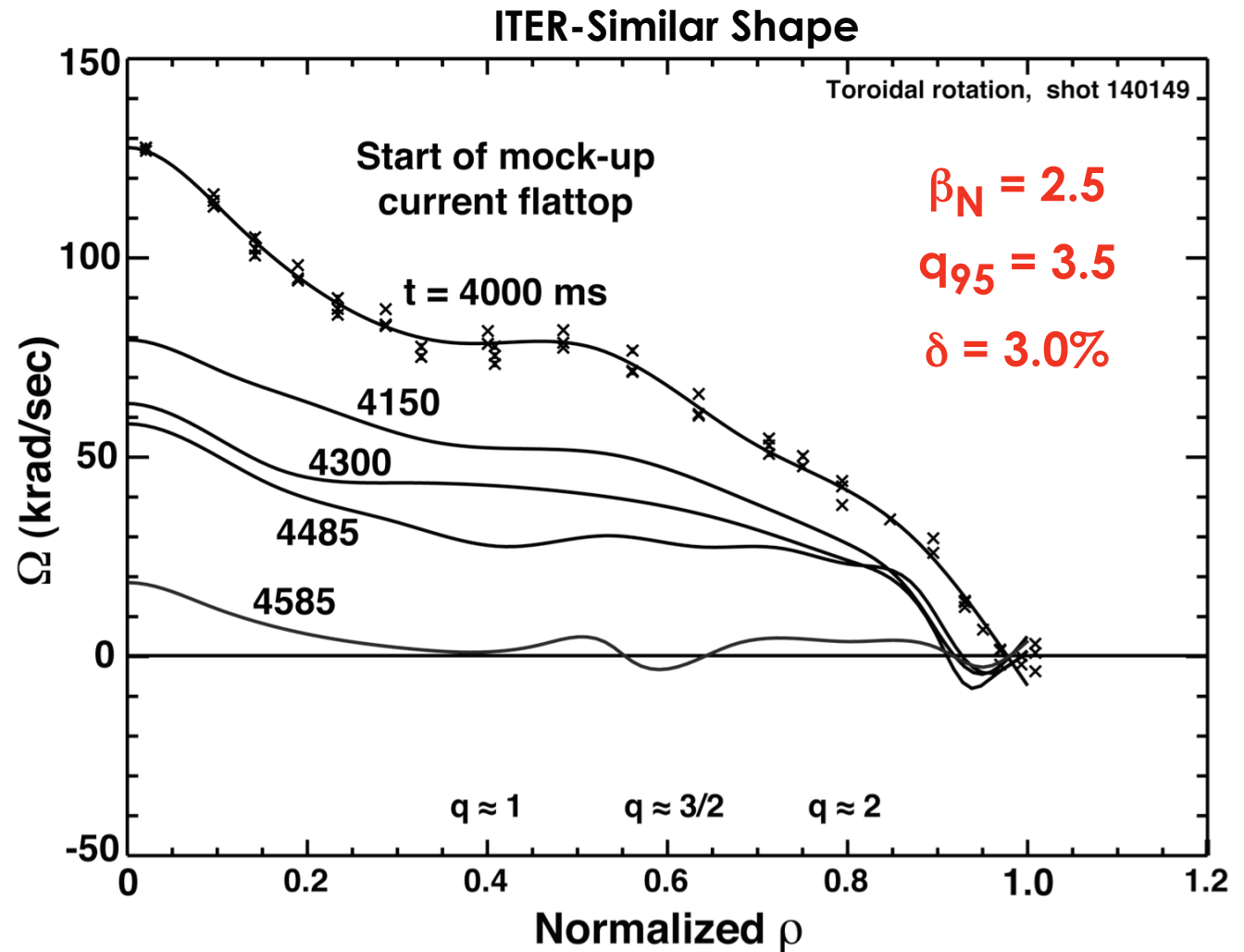
- **Plasma slowed**

- To ~50% of initial rotation in 300 ms
- $\Delta\Omega/\Omega \approx$ constant across plasma
- But not quite in steady torque balance

- **Gradually slowed**

- **Locking ensued**

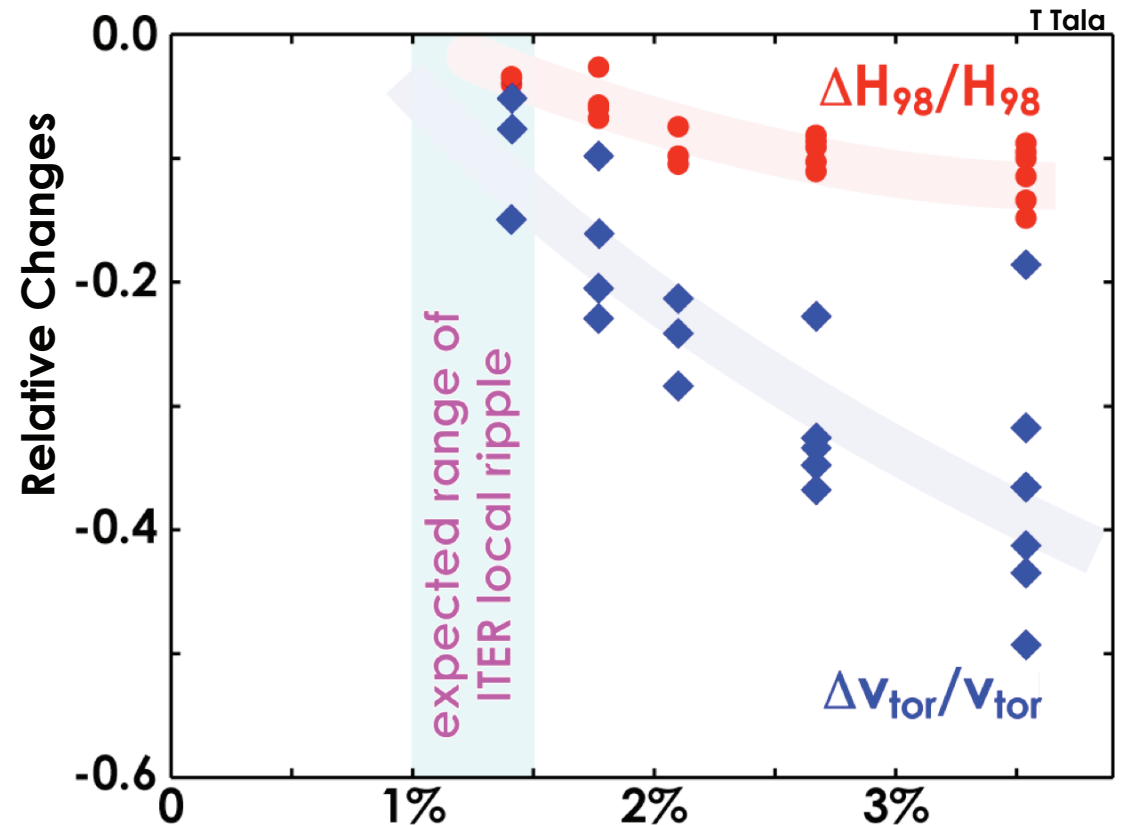
- Possibly at $q=1$ and $q=3/2$ surfaces



TBM Field Reduced H-mode Confinement Much Less Than Toroidal Velocity

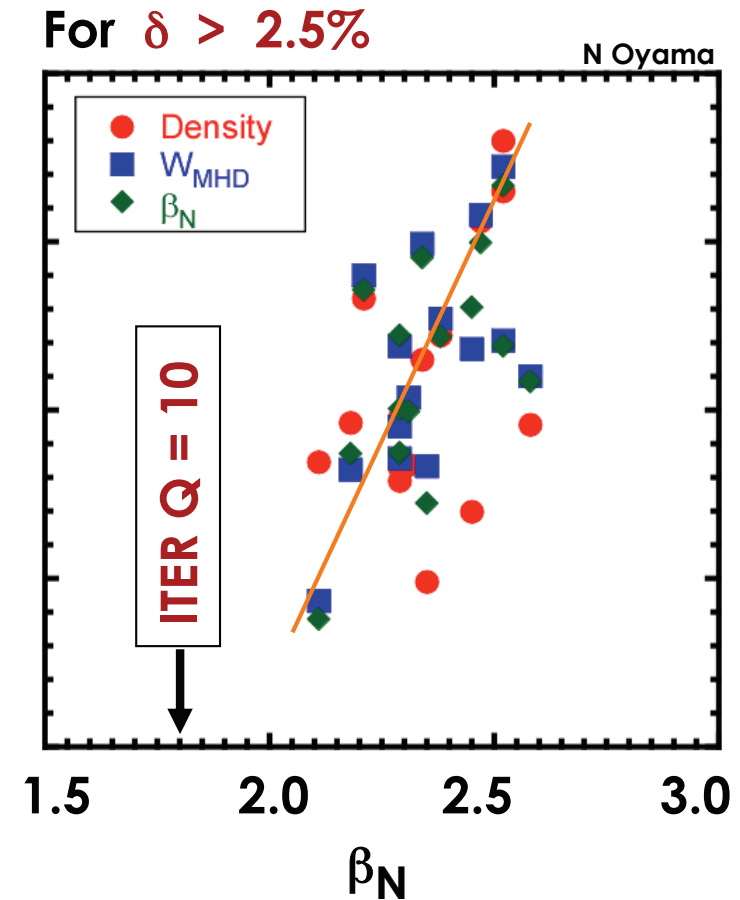
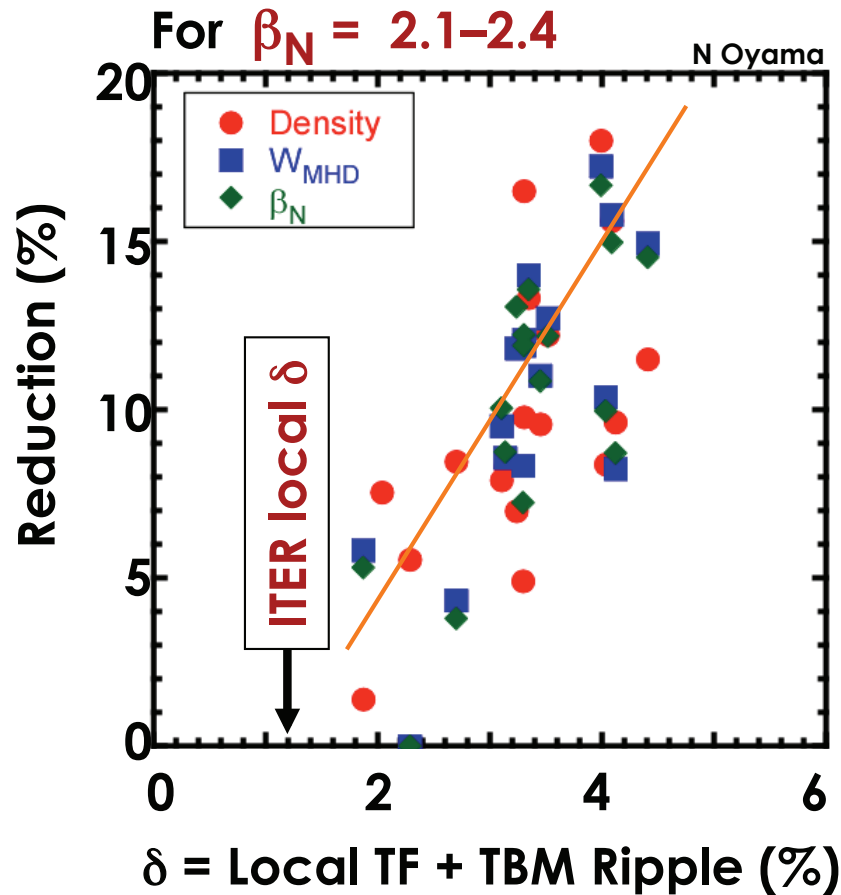
- **Confinement change** was $\sim 1/3$ of the **velocity change**

– Similar to confinement vs. angular momentum relation observed by PA Politzer et al., NF, 2008.

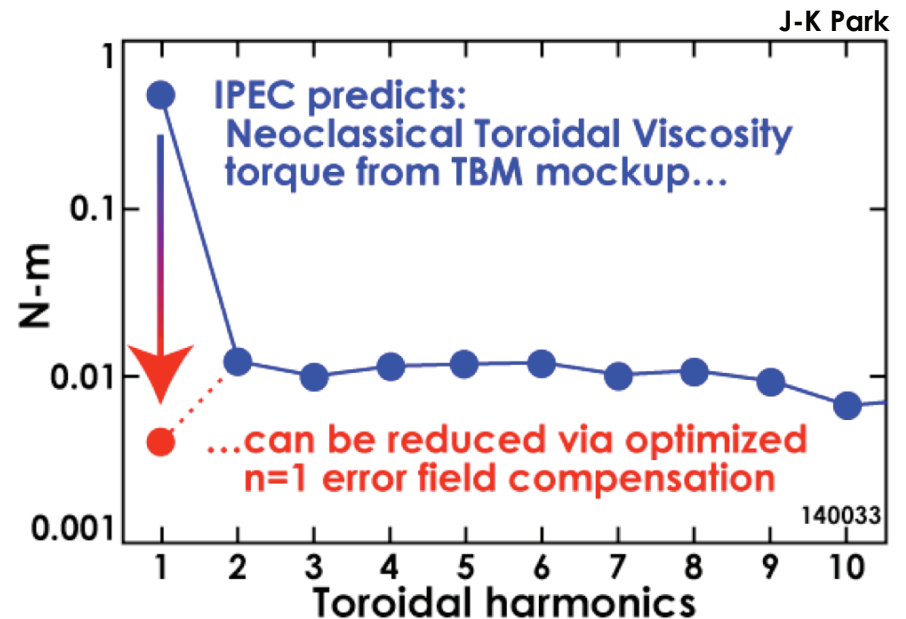
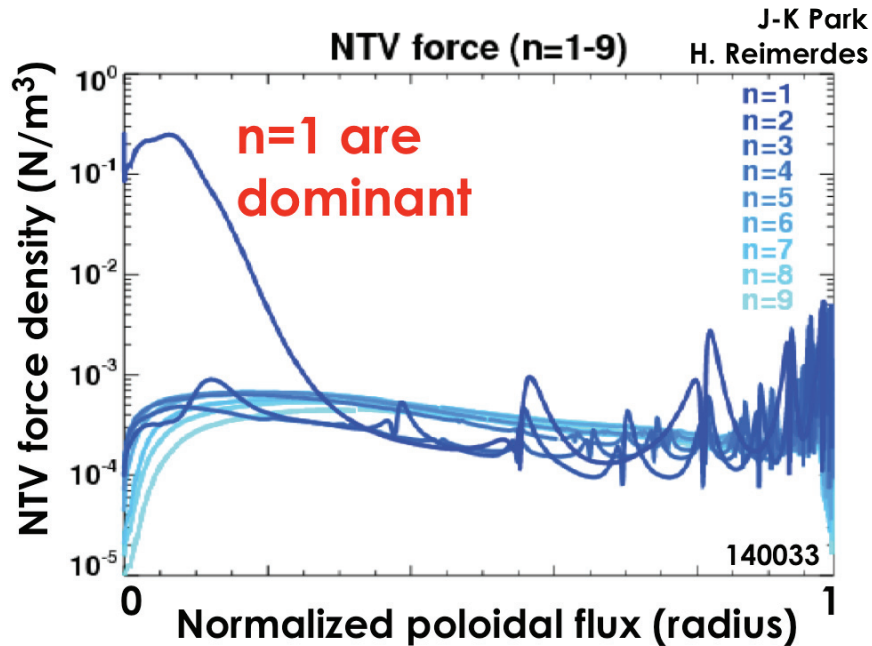


TBM Effects Were Small at ITER Baseline Parameters; We Did Systematic Scans Only at Higher Ripple and β

$R_{\text{midout}} = 2.30 \text{ m}$ $q_{95} = 3.5$ $B_T = 1.7 \text{ T}$ $I_p = 1.4 \text{ MA}$



The Ideal Perturbed Equilibrium Code (IPEC) Implicates n=1 Harmonics in Toroidal Rotation Braking

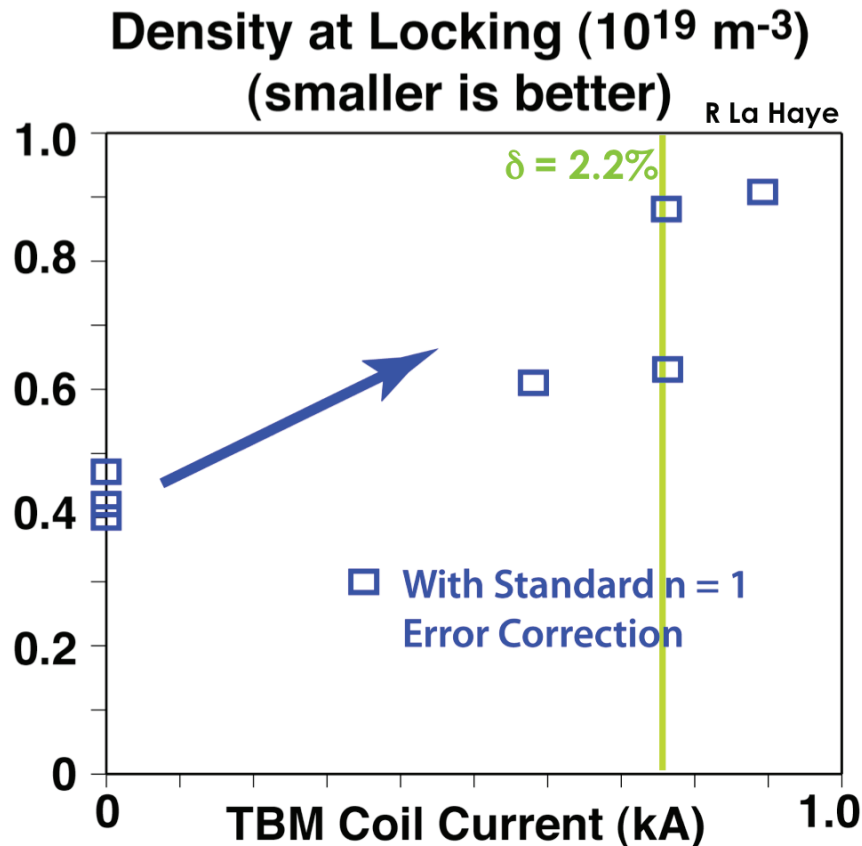


- Neoclassical Toroidal Viscous (NTV) braking by mock-up field is mostly from n=1 harmonics in central core
 - n=1 is amplified by H-mode

- A great opportunity...
- Expect to recover higher plasma velocity and thus higher confinement

[JK Park, PI2.00001, XO4.00003]

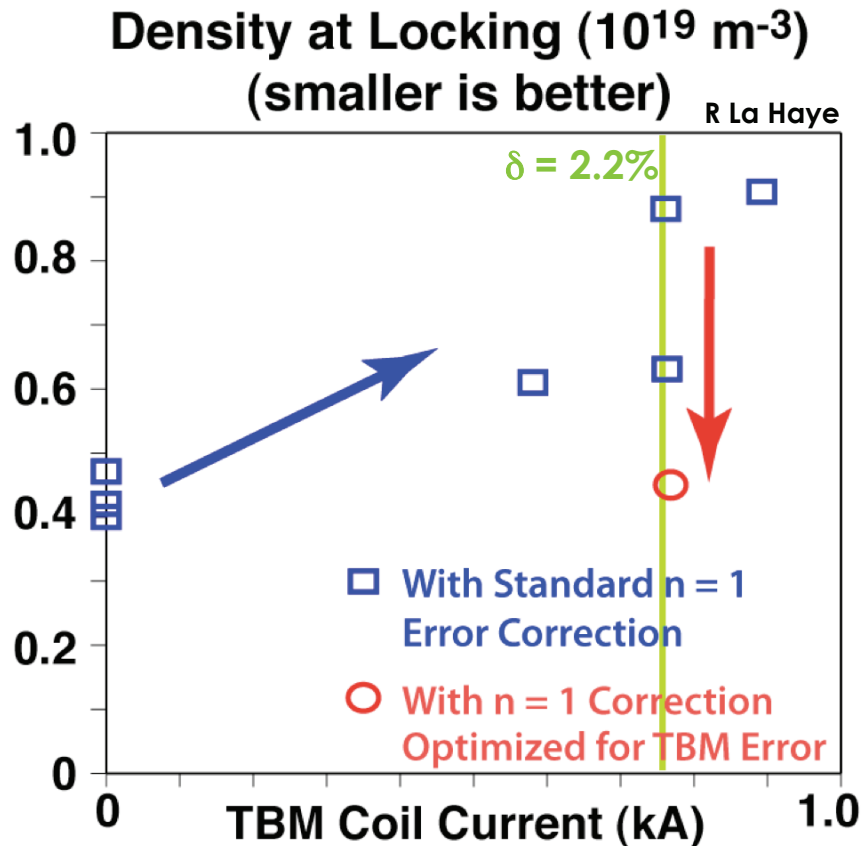
DIII-D n=1 Error Compensation Was Re-Optimized with TBM, and It Restored Locked Mode Tolerance at Low β



- TBM field raised the low-density threshold for locked mode avoidance
- n=1 re-compensation restored locked mode tolerance in low- β test

Using Standardized Ohmic Low-Density Locking Test Plasmas

DIII-D n=1 Error Compensation Was Re-Optimized with TBM, and It Restored Locked Mode Tolerance at Low β



Using Standardized Ohmic Low-Density Locking Test Plasmas

- TBM field raised the low-density threshold for locked mode avoidance
- n=1 re-compensation restored locked mode tolerance in low- β test
- IPEC analysis and experiment \approx agree
- Will n=1 compensation also restore H-mode confinement at high β ?

How to Scale from 1 Port in DIII-D to 3 Ports in ITER?

- If TBM effects vary linearly with ripple mirror ratios, and if they also sum linearly, then equivalence is $\delta_{\text{DIII-D}} = 3 \delta_{\text{ITER}}$
 - for case of 1 DIII-D vs. 3 ITER ports
- If TBM effects $\propto \delta^\alpha$, then the $(\delta_{\text{DIII-D}})^\alpha = (3 \delta_{\text{ITER}})^\alpha$ relation would be $3^\alpha (\delta_{\text{ITER}})^\alpha$ in DIII-D vs. $3 (\delta_{\text{ITER}})^\alpha$ in ITER
 - $\alpha \approx 2$ for DIII-D density reduction data (V Chuyanov)
 - DIII-D TBM experiments may be more pessimistic than ITER reality
- A different view: ITER rotation may be too small for the TBM error to be of concern

Summary

- DIII-D experiments did not reveal any reason why a TBM-like error field would seriously limit ITER plasma performance objectives
- IPEC numerical analysis indicates that NTV braking is dominated by plasma-amplified $n=1$ TBM error field
 - IPEC suggests rotation recovery by $n=1$ compensation
- A DIII-D experiment demonstrated $n=1$ compensation of the TBM contribution to locking in OH plasmas
- DIII-D plans to do experiments in 2011 to test if $n=1$ compensation can reduce rotation and confinement degradations in high- β H-mode plasmas