

**Abstract Submitted for the 51<sup>st</sup> Annual Meeting**  
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**SOL Width Studies for ITER Ramp-up,**\* D.L. Rudakov, J.A. Boedo, R.A. Moyer, *UCSD*; R.A. Pitts, *ITER*, A.W. Leonard, G.L. Jackson, *GA*; C.J. Lasnier, *LLNL*; P.C. Stangeby, *UTIAS*; J.G. Watkins, *SNL*; L. Zeng, *UCLA* – The present ITER scenarios contain limiter phases in Ohmic or L-mode for ramp-up and ramp-down. Dependencies of scrape-off layer (SOL) density and temperature e-folding lengths on discharge parameters were studied in inner-wall limited (IWL) and lower single null (LSN) discharges on DIII-D in order to benchmark the currently used power flux scaling based on divertor measurements from a number of tokamaks. Scans of the plasma current ( $q_{95}$ ), density, and heating power have been performed. The power flux e-folding lengths derived from the reciprocating probe measurements near the outboard midplane in the IWL configuration are on the average a factor of  $\sim 2$  larger than those predicted by the scaling for an outer-wall limited configuration. Probe measurements in LSN configuration were also benchmarked against infrared camera data from the lower divertor.

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