

# H-mode Power Threshold, Pedestal and ELM Characteristics and Transport in Hydrogen Plasmas in DIII-D

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# Motivation

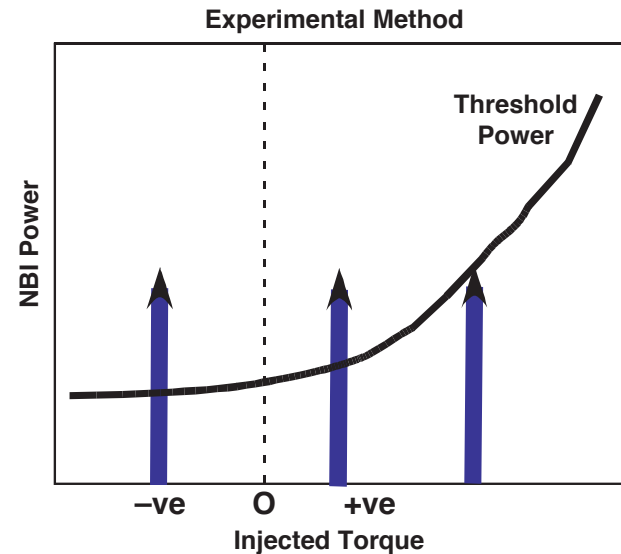
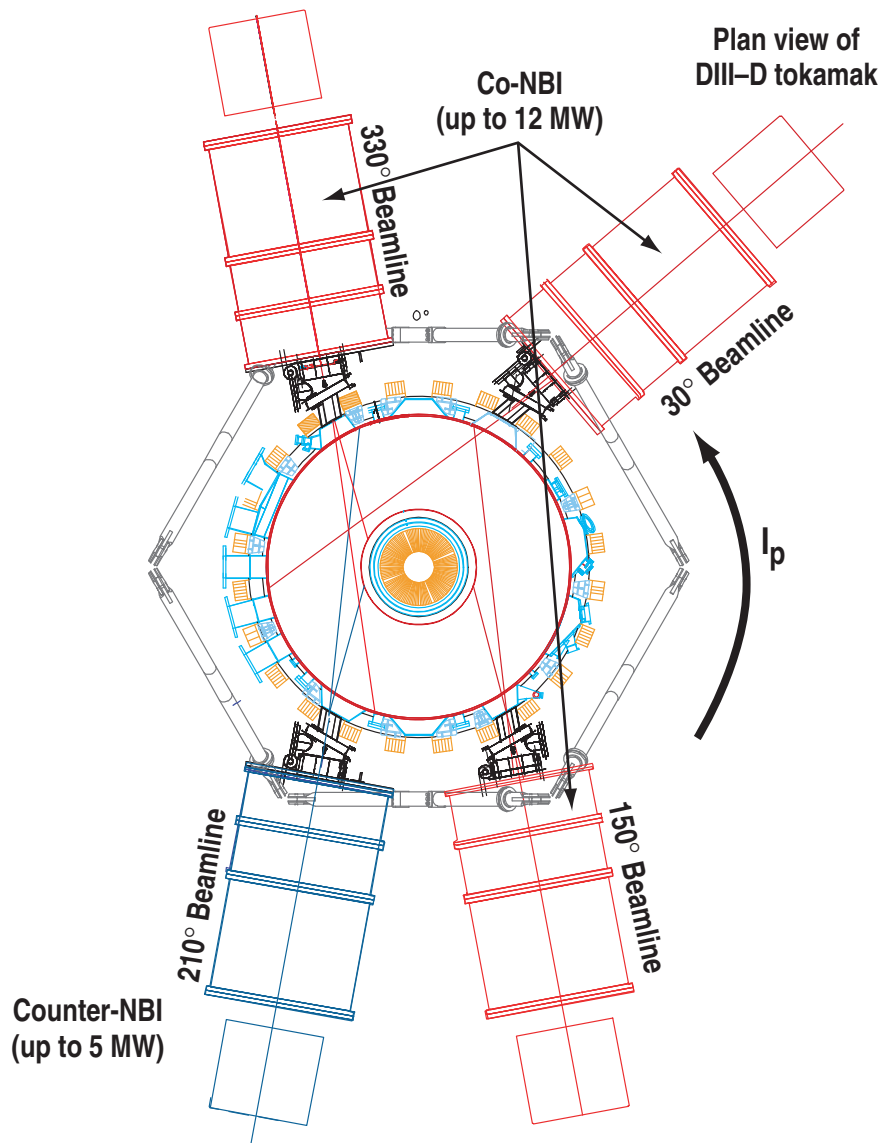
## Operations

- **First (non-activated) operational phase in ITER is planned to be with hydrogen and/or helium plasmas**
  - Important phase for developing control hardware and techniques (e.g. for ELMs NTMs, etc)
  - Determine the interaction with plasma facing components (e.g. divertor heat loads and erosion, etc)
  - Dependent on obtaining H-mode plasmas

## Physics

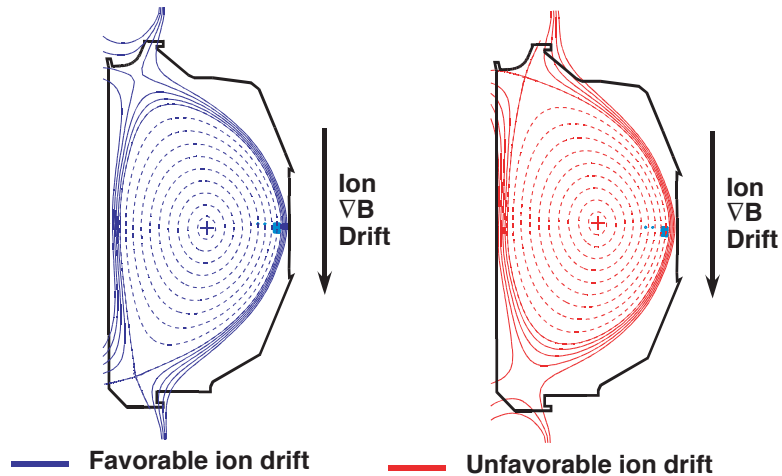
- **Dependence of the pedestal width on  $\beta_{\theta}^{\text{ped}}$  or  $\rho_{i\theta}$** 
  - Mass dependence can resolve this issue

# The Injected Torque was Controlled by Careful Selection of the DIII-D Neutral Beams

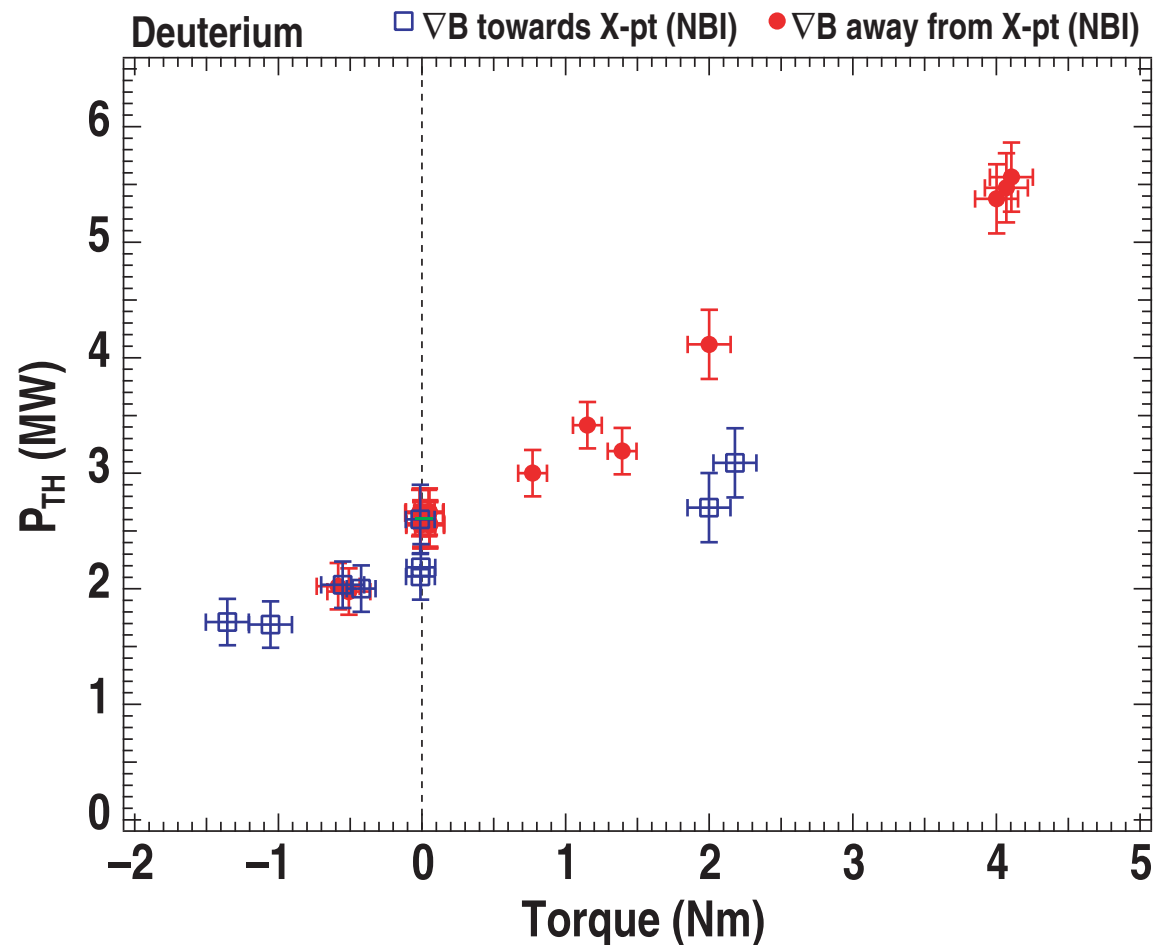


- Experiments performed with
  - NBI ( $H^0$ )  $\rightarrow$   $H^+$  plasmas
  - NBI ( $D^0$ )  $\rightarrow$   $D^+$  plasmas
- Capability for performing simultaneous co-current and counter-current NBI
  - Provides independent control of torque and power

# For D Plasmas with D-NBI, the H-Mode Power Threshold Varies Strongly with the Applied Beam Torque

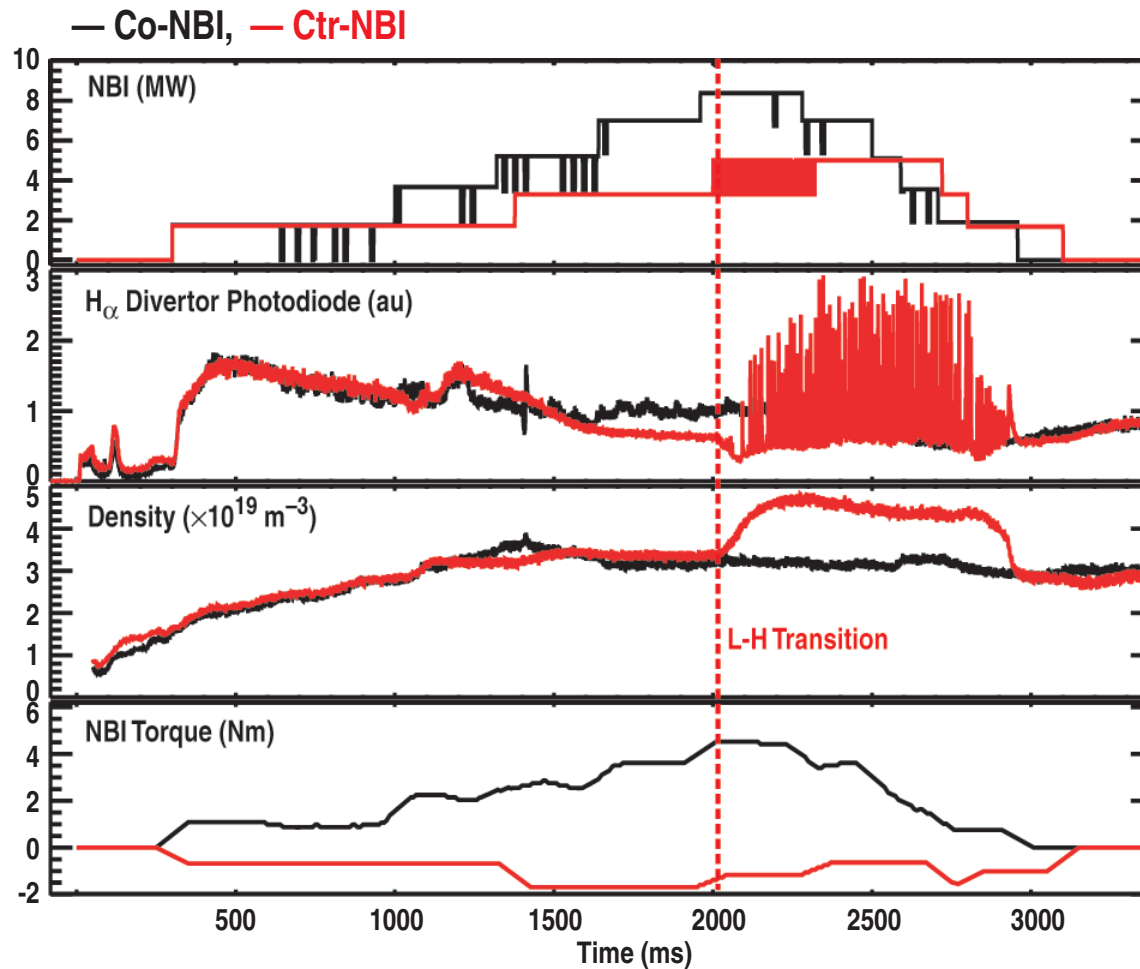


- Factor of ~3 increase in  $P_{TH}$  for discharges with unfavorable ion drift
- Factor of ~2 increase in  $P_{TH}$  with favorable ion drift



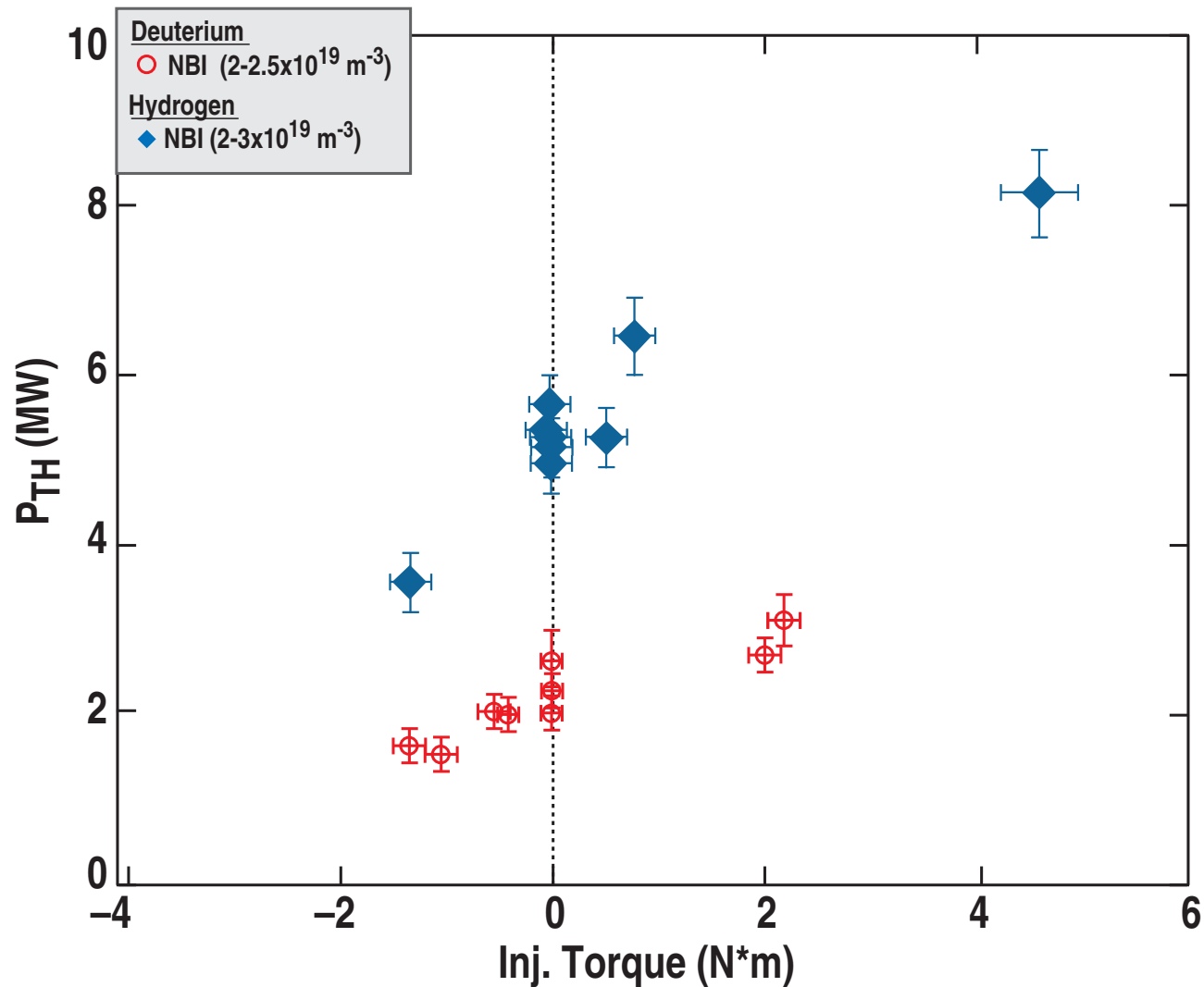
# Hydrogen Plasmas: The H-mode Power Threshold with Counter-NBI is at Least a Factor of 2 Lower Than with All co-NBI

- NBI( $H^0$ )  $\rightarrow$   $H^+$  plasmas
- With co-NBI: stays in L-mode at medium target density ( $3 \times 10^{19} \text{ m}^{-3}$ )
- Hydrogen purity (H/H&D) was above 90% in L-mode



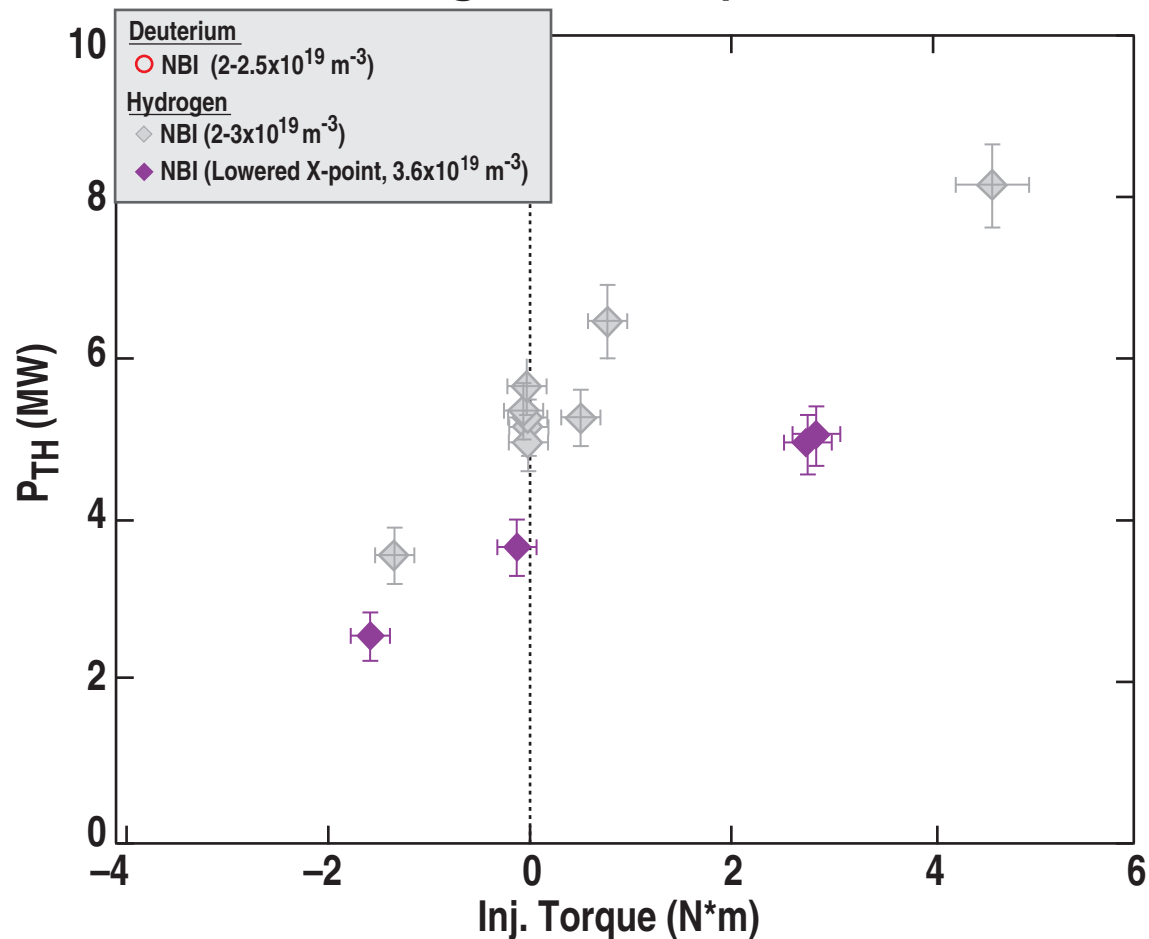
# The H-mode Threshold Power Increases With Injected Torque in Hydrogen Plasmas (Similar to Deuterium Plasmas)

- Hydrogen threshold power is twice that for deuterium at zero torque



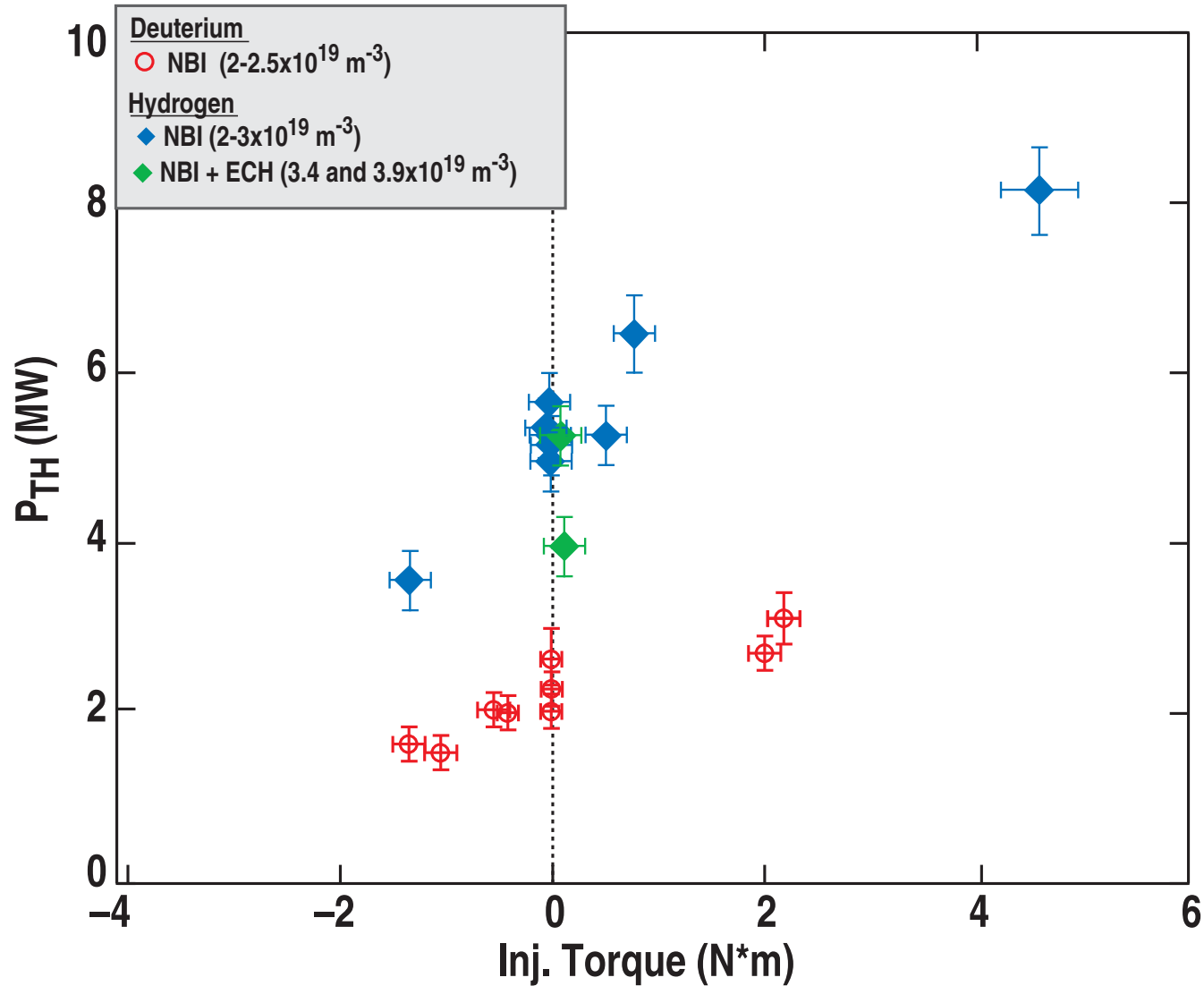
# The H-mode Power Threshold is Dependent on the Location of the Plasma X-point From the Divertor Surface

- Threshold power lowered by 20%–40% by reducing height of X-point above lower divertor from 26 to 10 cm
- Trend of increasing threshold power with increasing torque still present





# Application of ECH Lowers the Required Threshold Power Slightly (~15%) Compared to the NBI Discharges





# Pedestal Width Studies With H and D Plasmas Allows Determination of Width Dependence on $\beta_{\theta}^{\text{ped}}$ or $\rho_{i\theta}$

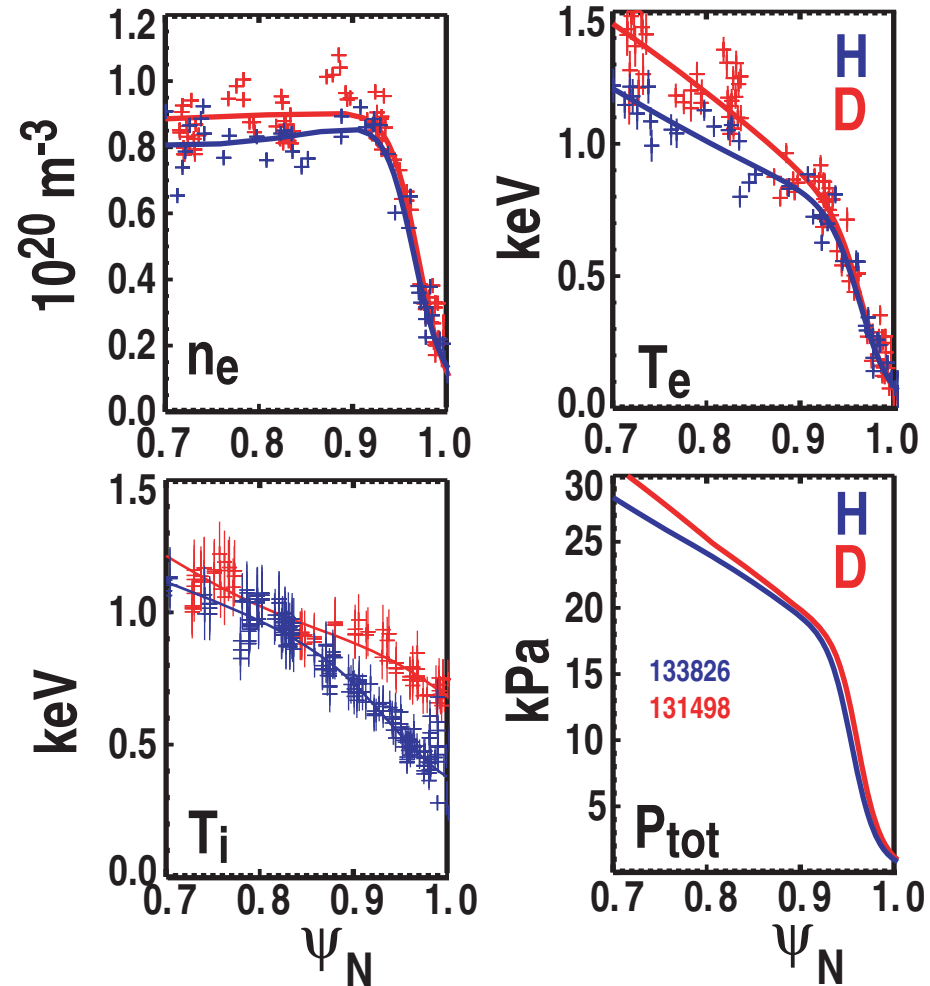
- Comparing the pedestal widths in D and H plasmas provides an opportunity to break the degeneracy

- $\rho_{i\theta}$  scales with ion mass as  $\sqrt{M_i}$
- $\beta_{\theta}^{\text{ped}}$  has no explicit mass dependence

- $\sqrt{\beta_{\theta}^{\text{ped}}}$  scaling:  $\Delta_p^H/\Delta_p^D = 1$

- $\rho_{i\theta}$  scaling:  $\Delta_p^H/\Delta_p^D = 0.7$

- Actual  $\Delta_p^H/\Delta_p^D = 1.15$ 
  - Consistent with  $\sqrt{\beta_{\theta}^{\text{ped}}}$  scaling



# Conclusions

- **The net power required to access the H-mode increases with applied torque**
  - For both hydrogen and deuterium
  - Hydrogen threshold power is twice that for deuterium at zero torque
  - Trend with torque is favorable for ITER where low input torque is expected
  - The threshold power is sensitive to the plasma geometry; specifically, to the X-point height above divertor surface
- **Present power threshold scaling studies do not include torque, plasma rotation or plasma geometry effects**
  - may explain the large range/error in prediction for present and future fusion devices
- **The pedestal width is consistent with a  $\sqrt{\beta_{\theta}^{\text{ped}}}$  scaling**