

Improved Performance in DIII-D Hybrid Discharges with a Dominant 4/3 Tearing Mode

by

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in collaboration with

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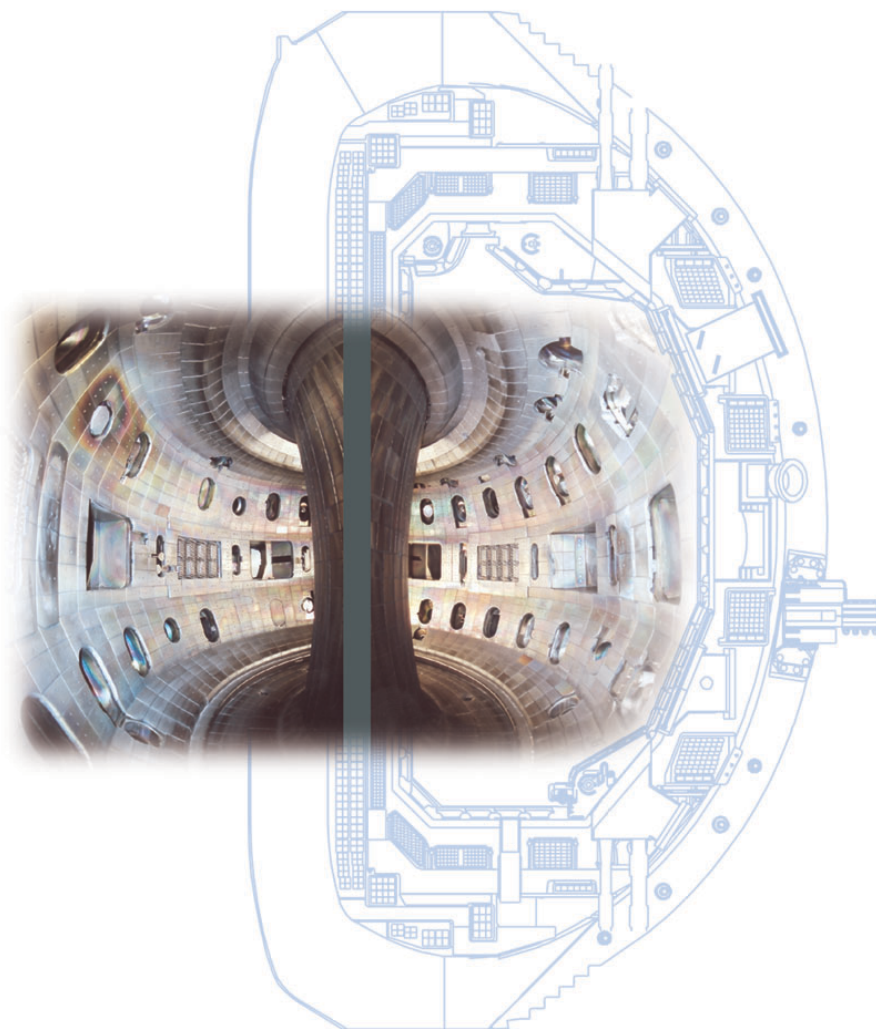
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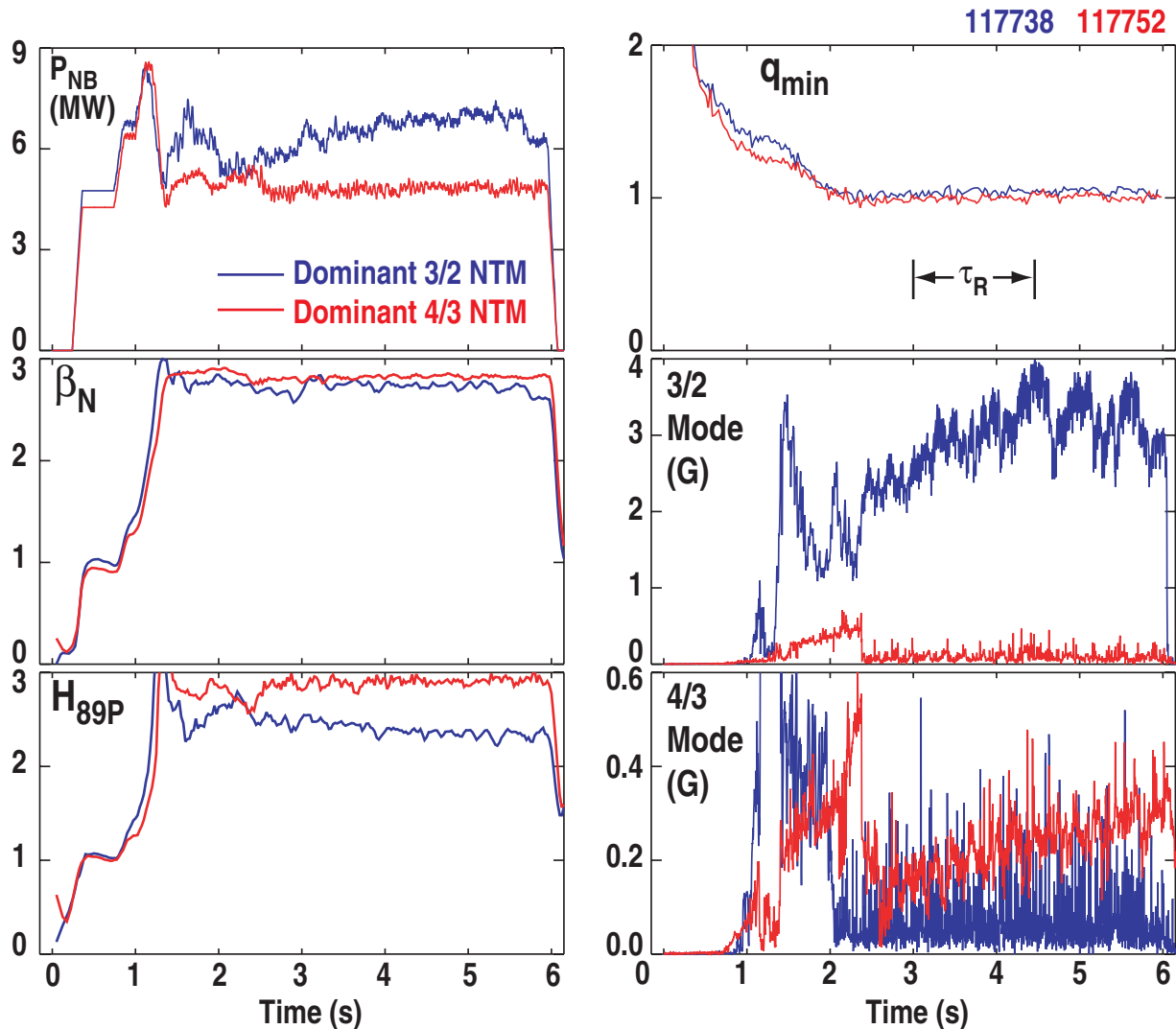
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Introduction

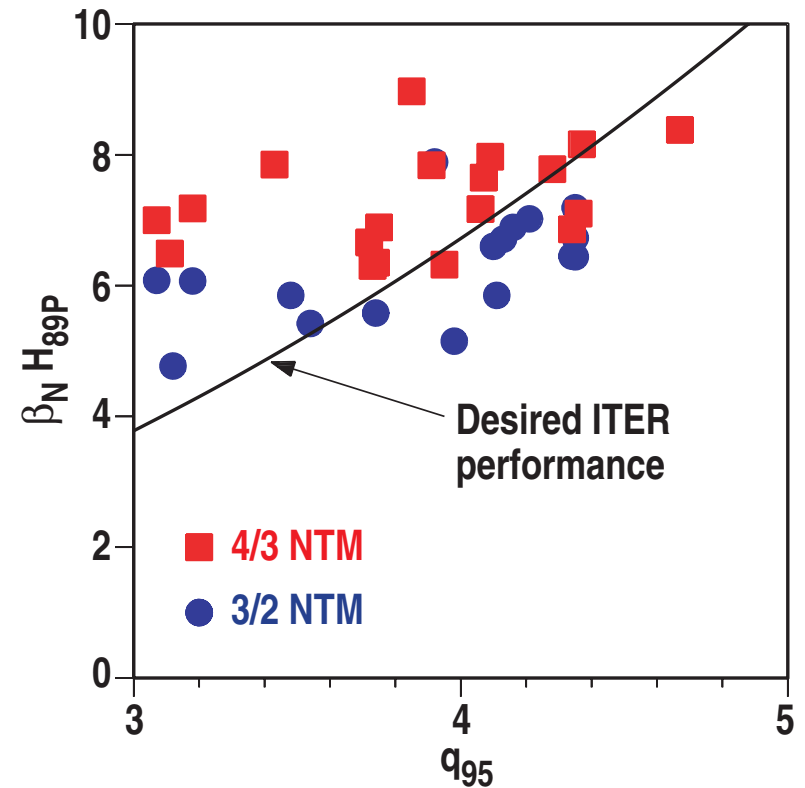
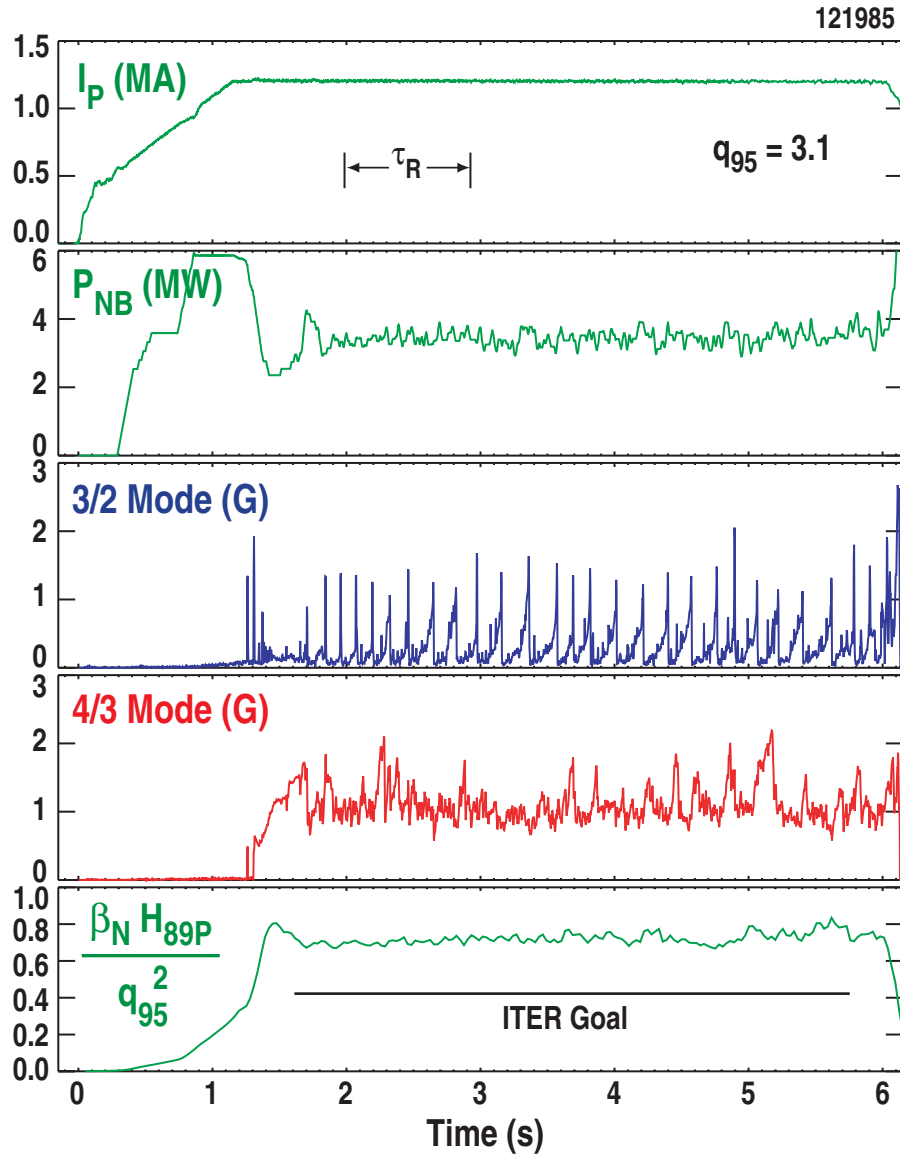
- **Hybrid regime in DIII-D refers to long duration, high performance discharges achieved by**
 - Preheating during current ramp to produce a safety factor profile with low central shear and $q > 1$
 - Quickly raising beta to near stability limit so that a neoclassical tearing mode (NTM) is triggered before sawteeth begin
- **Usually a small $m/n = 3/2$ NTM develops which (for $q_{95} > 4$) maintains $q > 1$ indefinitely and inhibits sawteeth**
- **This talk focuses on cases where an $m/n = 4/3$ NTM develops spontaneously instead, resulting in improved energy confinement albeit with sawteeth**

Hybrid Discharges Can Develop Either Dominant 4/3 NTM or Dominant 3/2 NTM, Depending on Initial Conditions



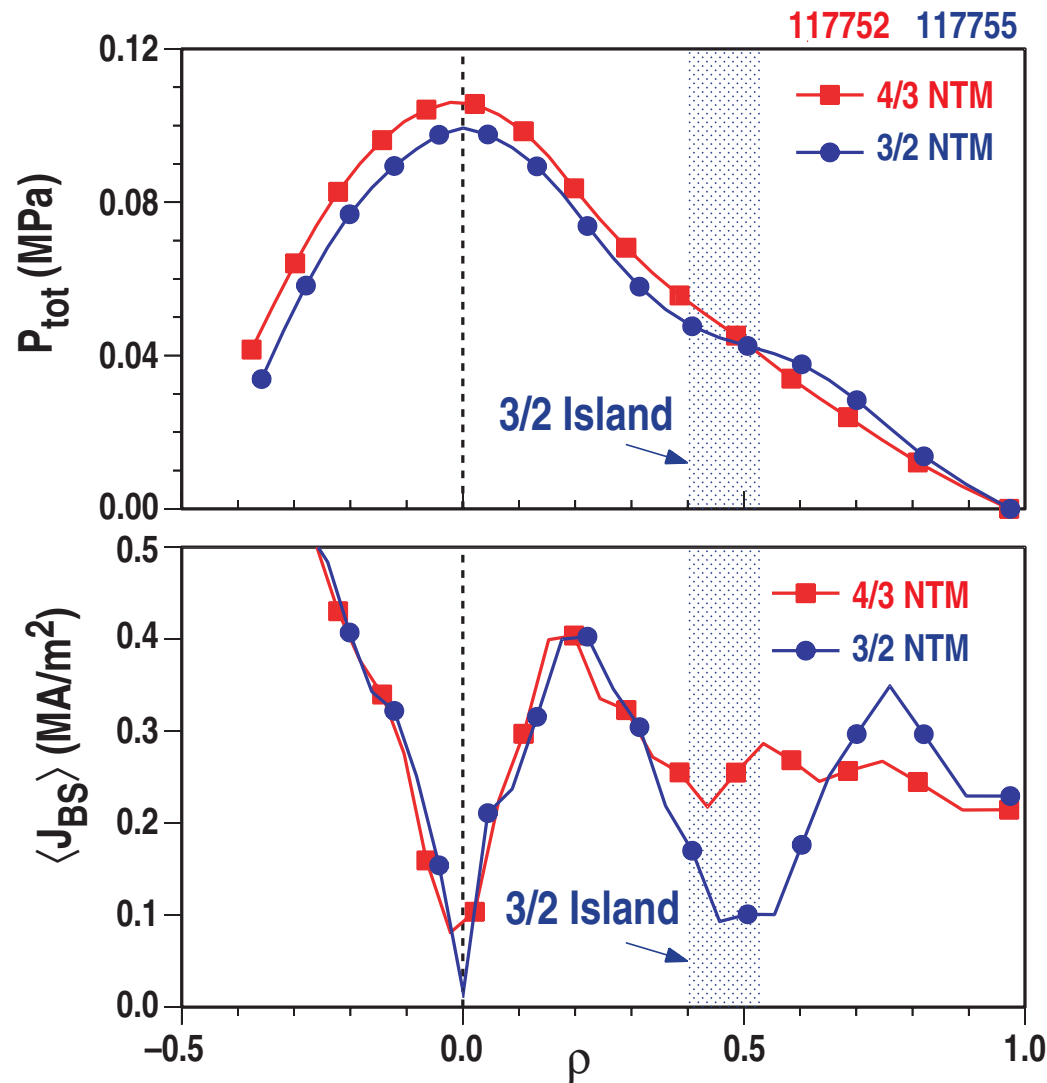
- 4/3 and 3/2 modes compete for same free energy source, so they don't usually co-exist
- Discharge with dominant 4/3 NTM has $\approx 25\%$ higher H_{89P} -Factor
- Dominant 4/3 NTM discharge also has $q_{min} < 1$ and sawteeth

Hybrid Discharges with Dominant 4/3 NTM Achieved Performance 70% Above Desired ITER Value for $\tau_{dur} = 4.8\tau_R$



- Hybrid discharges equal or exceed ITER baseline performance over a large range of q_{95} , especially when 4/3 NTM is dominant

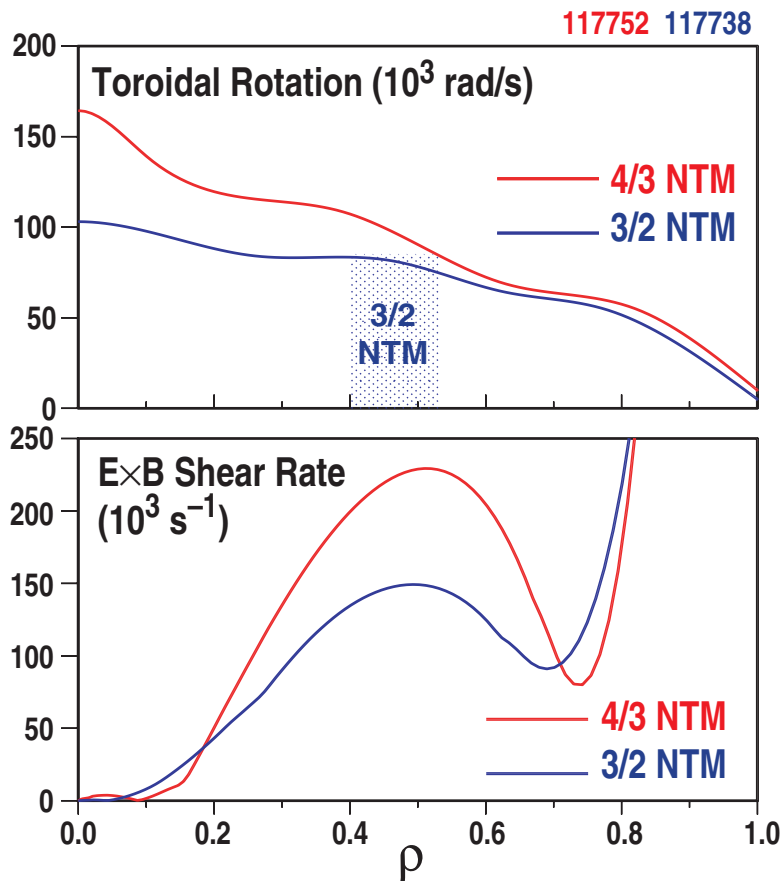
One Explanation for Lower Confinement in 3/2 NTM Hybrids is Flattening of Pressure Profile Near $q = 1.5$ Surface



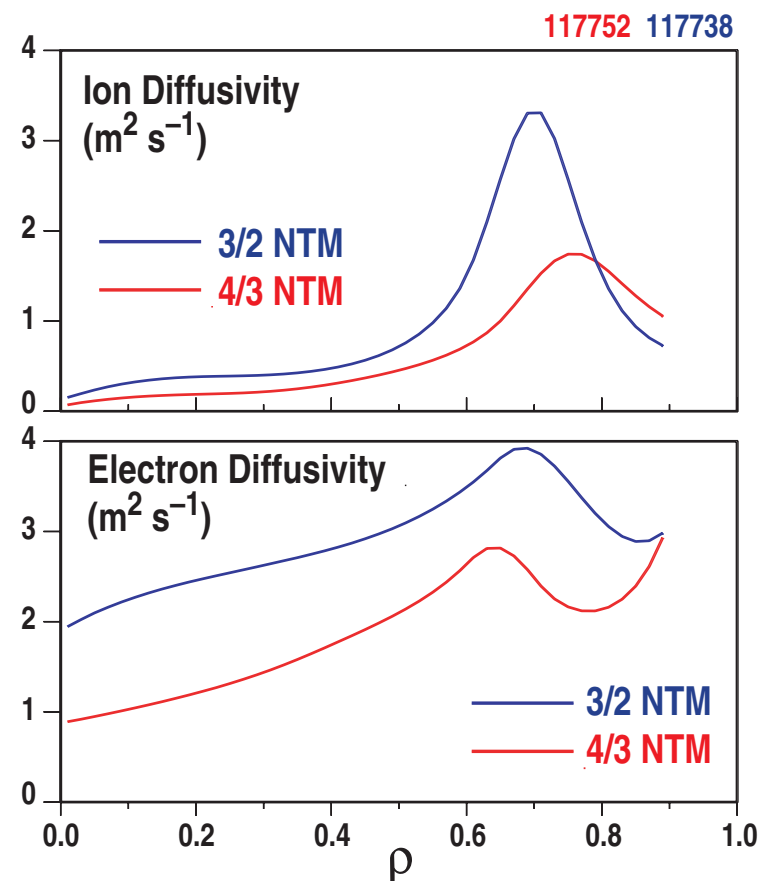
- Pressure profile and pressure-driven current densities are directly determined from MSE data
C.C. Petty et al., *Plasma Phys. Control. Fusion* (2005) p. 1077
- First direct measurement of “missing” bootstrap current near NTM island location
- Chang-Callen belt model gives $\approx 8\%$ loss of confinement due to 3/2 NTM ($\approx 2\%$ due to 4/3 NTM)

Second Explanation is Higher Toroidal Rotation in 4/3 NTM Hybrids Yields More $E \times B$ Shear and Thus Lower Transport

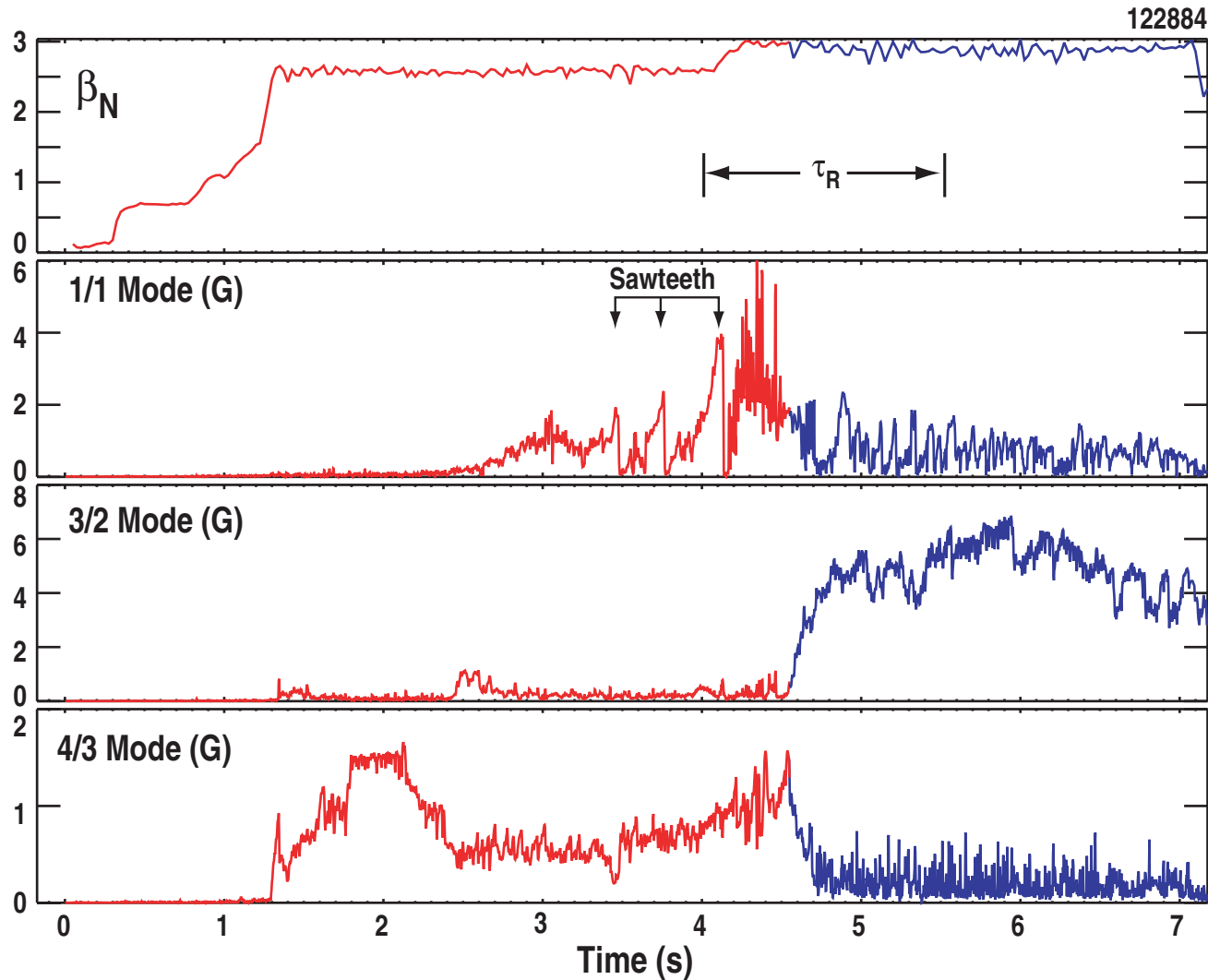
- 3/2 mode couples to vessel wall as well as 2/2 sideband near axis when $q_0 \approx 1$, dragging down toroidal rotation profile



- 4/3 NTM hybrids have reduced transport in both ion and electron channels over most of radius



Raising Beta Above $\beta_N = 2.8$ Results in 4/3 NTM Losing Dominance to 3/2 NTM



- After β_N increase, 3/2 NTM develops in less than one current redistribution time
- Sawteeth present during 4/3 NTM disappear when 3/2 NTM becomes dominant, which is favorable for not triggering a 2/1 NTM

Conclusions

- Hybrid discharges can develop either a dominant 3/2 NTM or a dominant 4/3 NTM, depending upon initial conditions
- Hybrid plasmas with dominant 4/3 NTM have up to 30% higher confinement times owing to
 - Less flattening of pressure profile near island location
 - More ExB shear from increased toroidal rotation because of reduced island dragging effects
- With $q_{95} = 3.1$, dominant 4/3 NTM hybrids have achieved $\beta_N H_{89P} / q_{95}^2 = 0.7$ for ≈ 5 current relaxation times, exceeding the ITER baseline by 70%

Additional Hybrid Presentations from DIII-D

- **Monday afternoon poster session:**

- Petrie “Radiating Divertor with Hybrid Plasmas” CP1.00016
- Politzer “Role of 3/2 Tearing Mode in Hybrid Scenario” CP1.00029
- Chu “3/2 Island and its effect on Central Tokamak Region” CP1.00030
- Makowski “Measured and Simulated MSE Signals of a tearing Mode” CP1.00031

- **Thursday morning poster session:**

- Wong “Density Fluctuation Measurements” QP1.00010
- McKee “Core Turbulence Structures” QP1.00021