

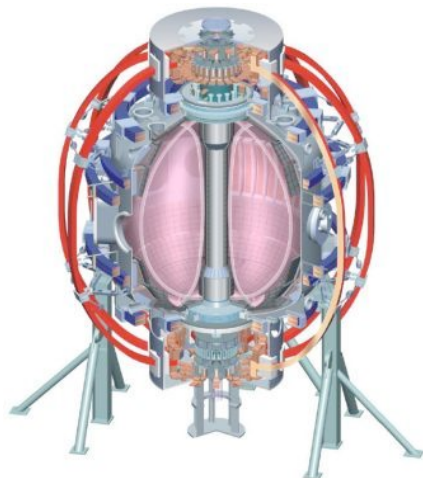
# Studies of Resonant and Non-Resonant 3-D Fields in NSTX

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and the NSTX Research Team**

**14<sup>th</sup> Workshop on Active Control of MHD Stability  
Nov. 11<sup>th</sup>, 2009**

Work supported by US DOE contract  
no. DE-AC02-09CH11466



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# What are 3-D Fields?

## Aren't Tokamaks Axisymmetric Enough?

- 3-D fields come from static coil shifts, tilts, non-circularities, coil motion, and current feeds...and can be deliberately applied.
  - “Error Fields” are subset...use this term when the 3-D field is not deliberate.
- $10^{-3}$  level  $n=1$  error fields can cause disruptive performance degradation.
  - Can cause stationary islands in low-density discharges (Error Field Penetration).
  - Can be amplified by marginally stable kink-modes, leading to rotation damping.
- $n>1$  3-D fields can degrade performance, or be used for control.
  - Neoclassical Toroidal Viscosity (NTV) can brake (or accelerate) the plasma.
  - Edge localized instabilities can be controlled (RMP).
  - Transient and steady state divertor heat loading patterns can be modified.

### *Purpose of this talk:*

*Explain how NSTX mitigates the deleterious 3-D field effects, utilizes positive effects. Primarily use data from the 2009 campaign, but reach back farther where it is helpful.*

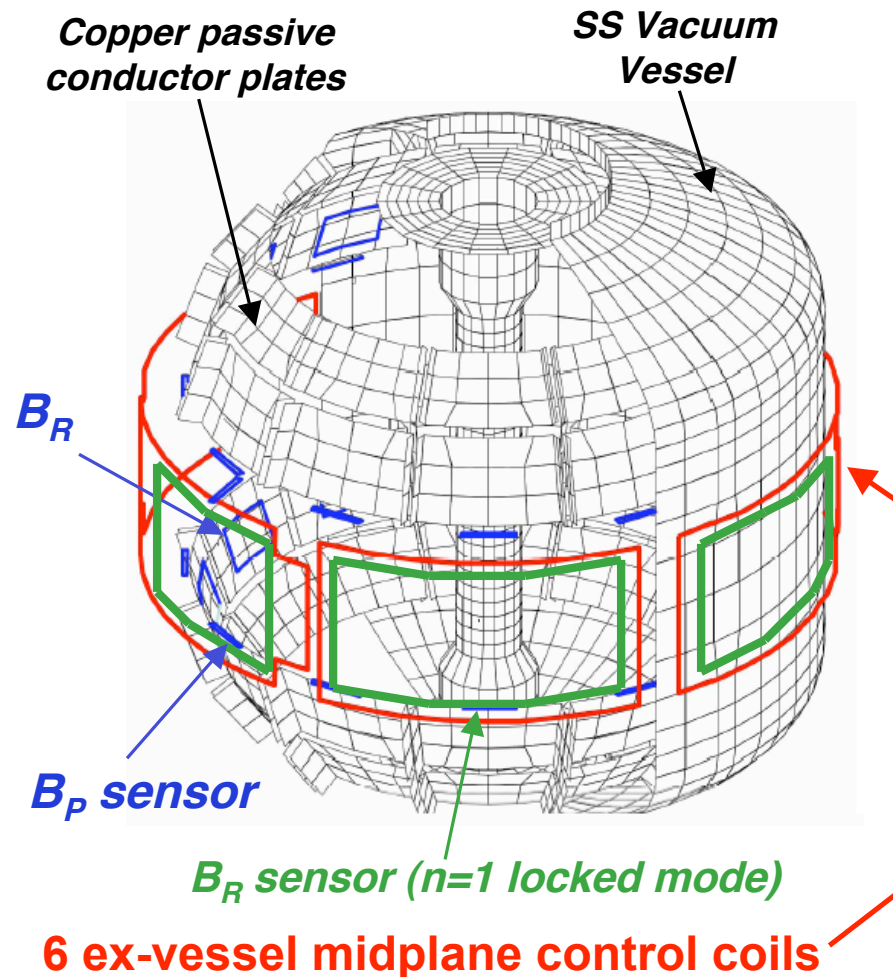
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- Background:
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  - The IPEC code
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# High- $\beta$ Plasmas, Non-Axisymmetric Field Detection, and Midplane Radial Field Coils Facilitate 3-D Field Studies

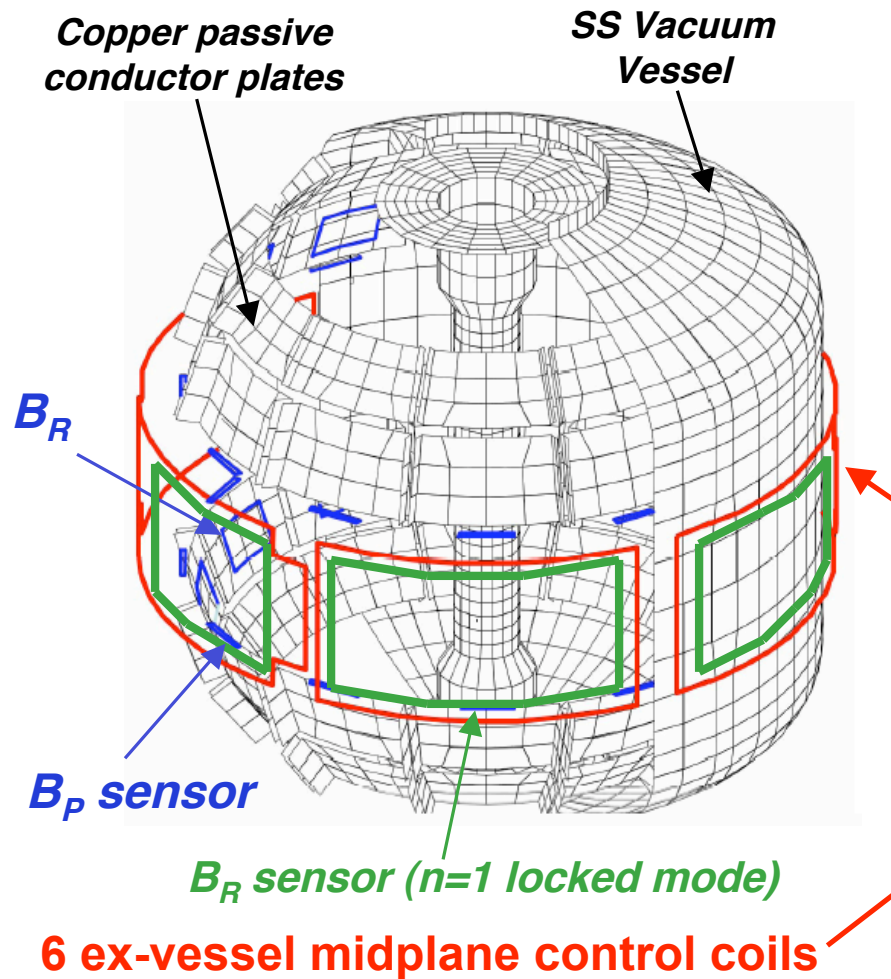


VALEN Model of NSTX (Columbia Univ.)

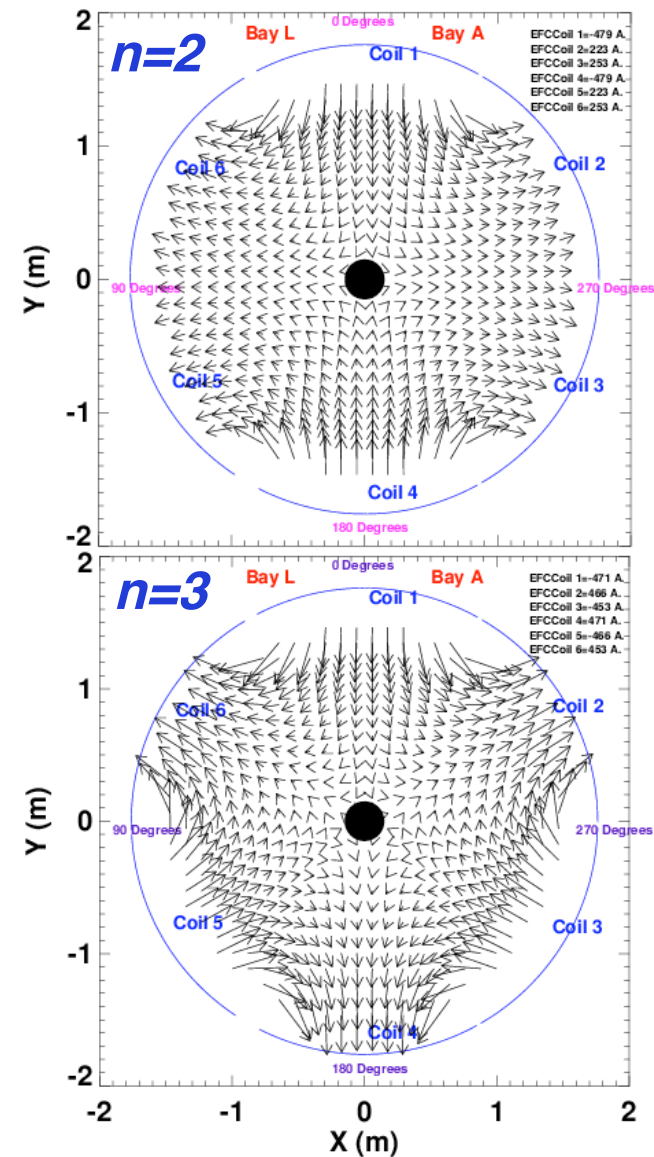
- Routine operation above the no-wall  $\beta$ -limit.
  - $\sim 6$  MW of 90kV NB injection.
  - Nearby copper stabilizing plates.
- Extensive non-axisymmetric field detection.
  - 24 off-midplane internal  $B_p$  sensors
  - 24 off-midplane internal  $B_R$  sensors
- Excellent profile diagnostics.
  - 30-point MPTS
  - 51-point toroidal CHERS
- 6 midplane radial field coils for 3-D field application.
  - Simultaneous  $n=1$  & 3
  - Simultaneous  $n=0$  & 2
  - Custom configurations
- Fast and slow non-axisymmetric feedback facilitated by the General Atomics Plasma Control System.



# High- $\beta$ Plasmas, Non-Axisymmetric Field Detection, and Midplane Radial Field Coils Facilitate 3-D Field Studies



VALEN Model of NSTX (Columbia Univ.)



# The Ideal Perturbed Equilibrium Code (IPEC) is Used to Understand 3-D Field Effects

- IPEC<sup>1</sup> gives ideal 3-D free-boundary tokamak equilibria.
- Solves the perturbed force balance equation

$$\vec{F}(\vec{\xi}) = \vec{\nabla} \delta p - \delta \vec{j} \times \vec{B}_0 - \vec{j}_0 \times \delta \vec{B} = 0$$

$$\delta \vec{B} = \vec{\nabla} \times (\vec{\xi} \times \vec{B}_0)$$

$$\delta \vec{j} = (\vec{\nabla} \times \delta \vec{B}) / \mu_0$$

$$\delta p = -\vec{\xi} \cdot \vec{\nabla} p_0 - \gamma p_0 (\vec{\nabla} \cdot \vec{\xi})$$

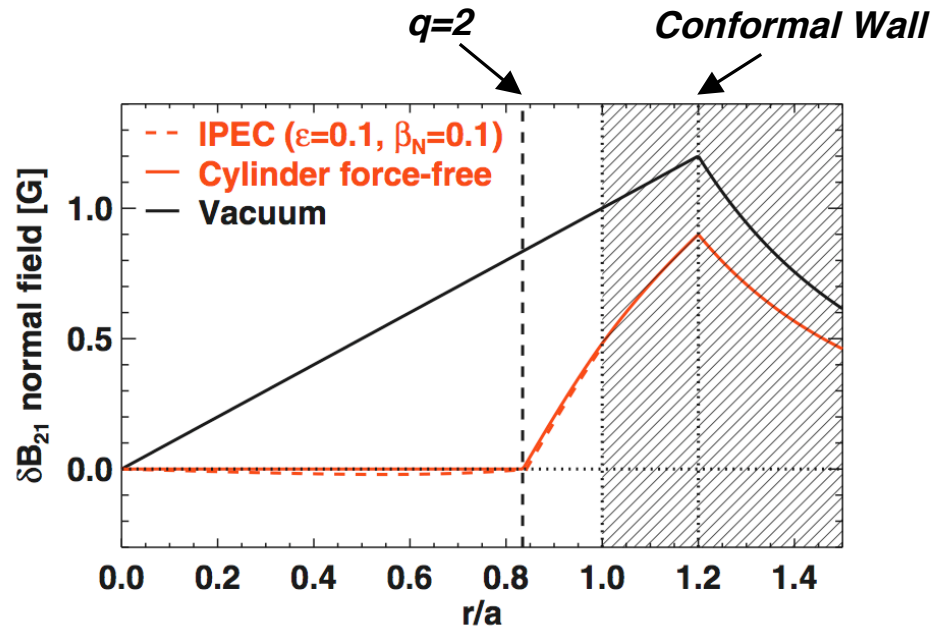
- Impose constraint that resonant magnetic perturbation vanishes at each surface.
  - Shielding currents are calculated to eliminate islands.
  - Relevant description of perturbed plasmas before the onset of error field penetration.
- Externally imposed non-axisymmetric fields represented by equivalent surface currents at the boundary.
- Shielding current determines **locking** properties.
- Lagrangian variation of the field strength determines **NTV transport**

[1] J. Park et al, *Phys. Plasmas* **14**, 052110 (2007)

## Example: Force-Free Cylinder

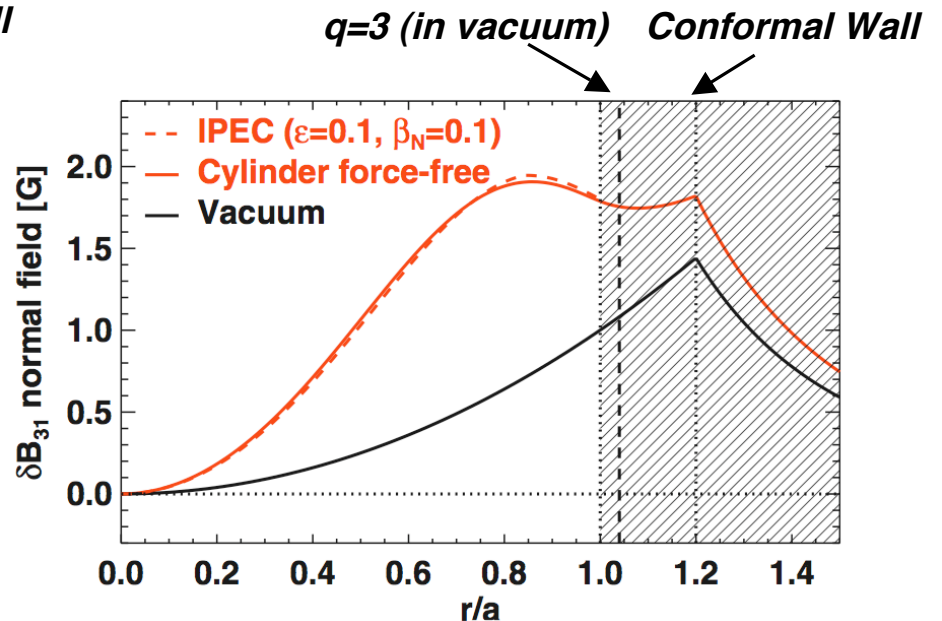
- Compare low- $\beta$ , high-A IPEC solution to force-free cylindrical solution.
- Non-axisymmetric fields applied by currents in the conformal wall.

*$m=2, n=1$  perturbation in a cylinder*



Shielding currents at  $q=2$  drive the resonant perturbation to zero.

*$m=3, n=1$  perturbation in a cylinder*



Easy to perturb the plasma.  
Large amplification of the applied fields

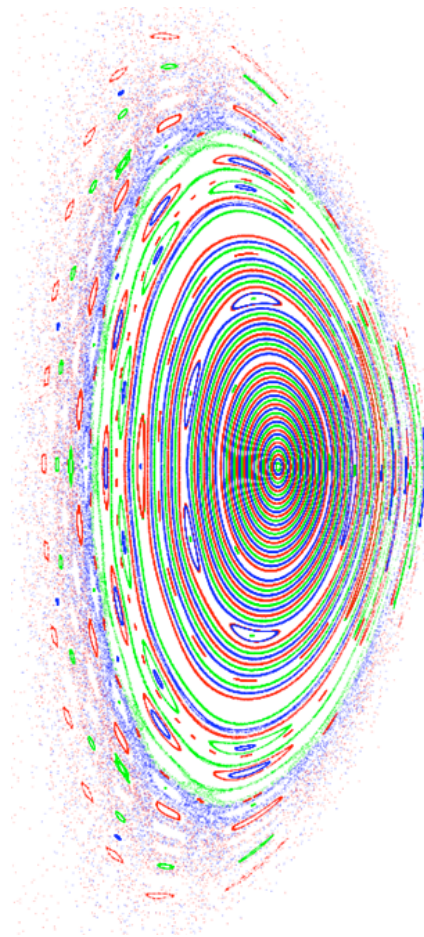


# Example: High- $\beta$ NSTX Plasma (Finite Beta, Toroidicity)

## *Application of $n=3$ Fields to a High- $\beta_N$ Equilibrium*

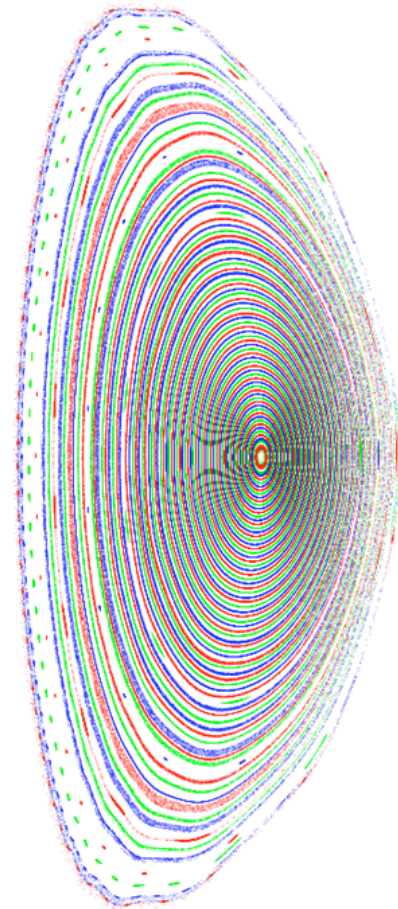
### *Vacuum Superposition*

Islands and Stochastic Regions



### *IPEC*

Surface Deformation



[1] J. Park et al, *Phys. Plasmas* **14**, 052110 (2007)

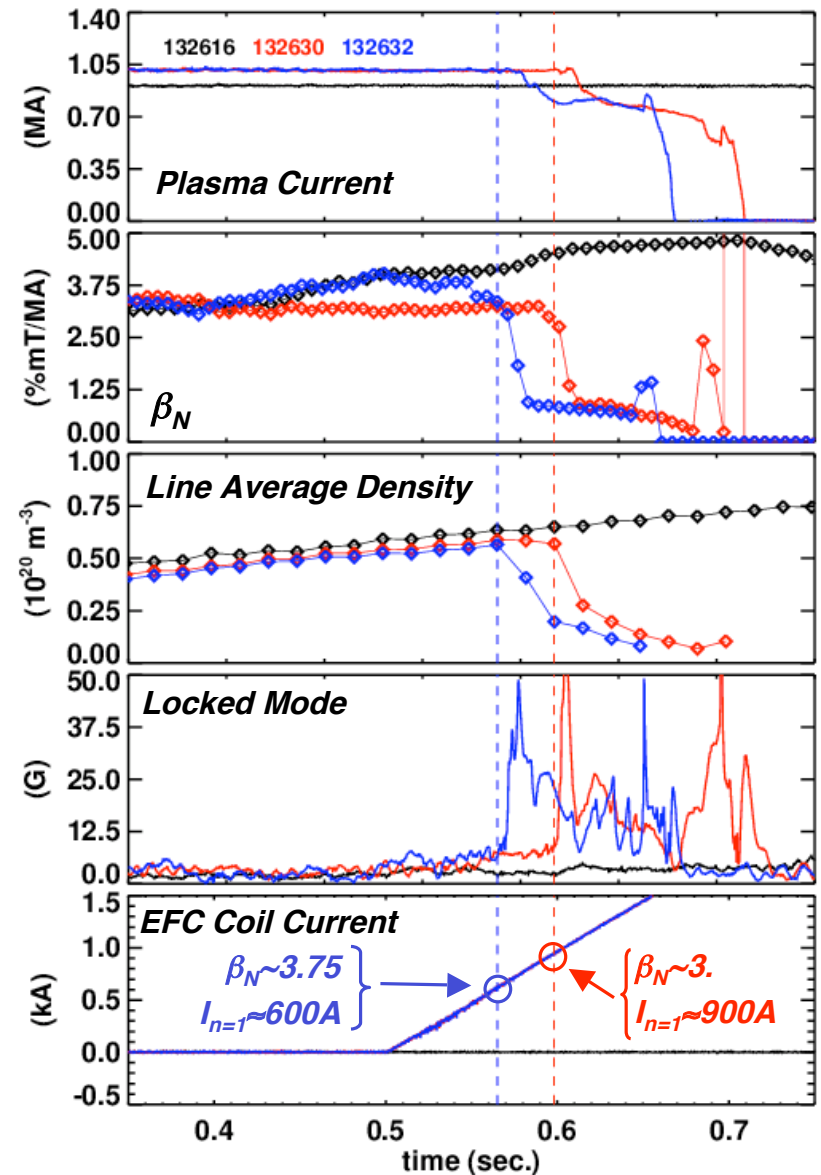
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# Increased $n=1$ Error-Field Sensitivity Observed at Higher- $\beta_N$

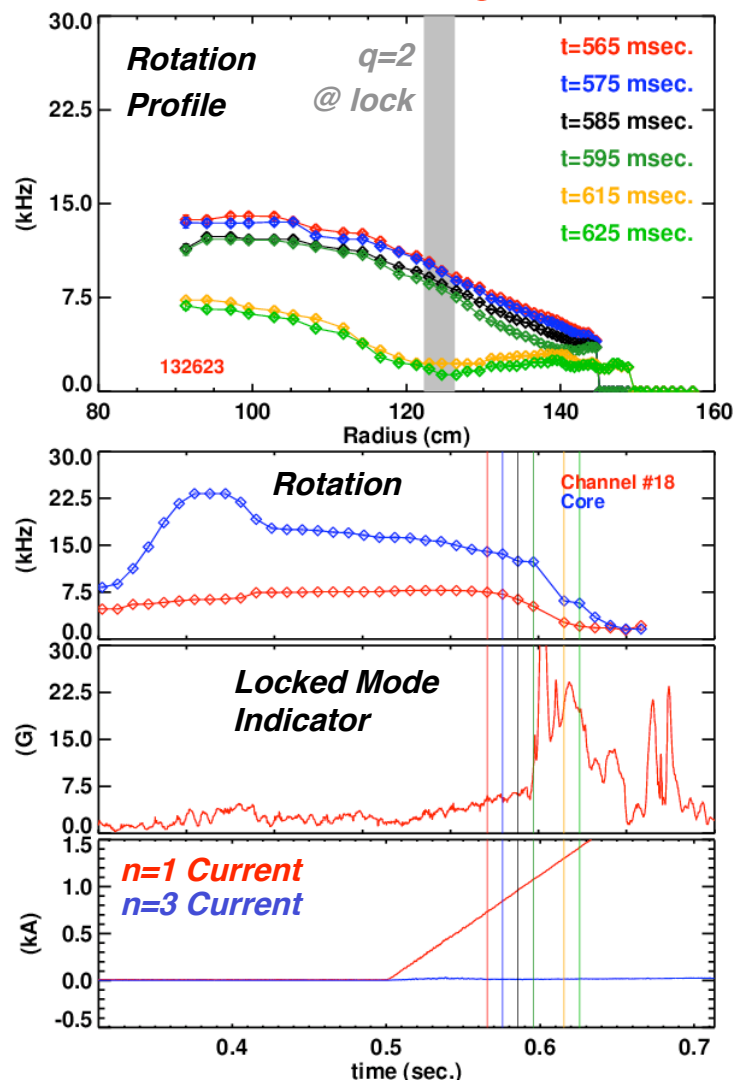
- Varied  $P_{inj}$ ,  $I_P$ , and  $B_T$  to achieve different  $\beta_N$  at fixed  $q_{95}$ .
- Apply ramping  $n=1$  field using the EFC coils.
- Measure the time of mode penetration using the internal resistive wall mode sensors.
- $\beta$ -dependence of locking at near identical density.
  - Higher- $\beta_N$  locks with ~600 Amps of  $n=1$  current
  - Low- $\beta_N$  locks with ~900 Amps of  $n=1$  current
  - Both cases beneath the no-wall limit.

*Increased error-field sensitivity at higher- $\beta_N$ !*

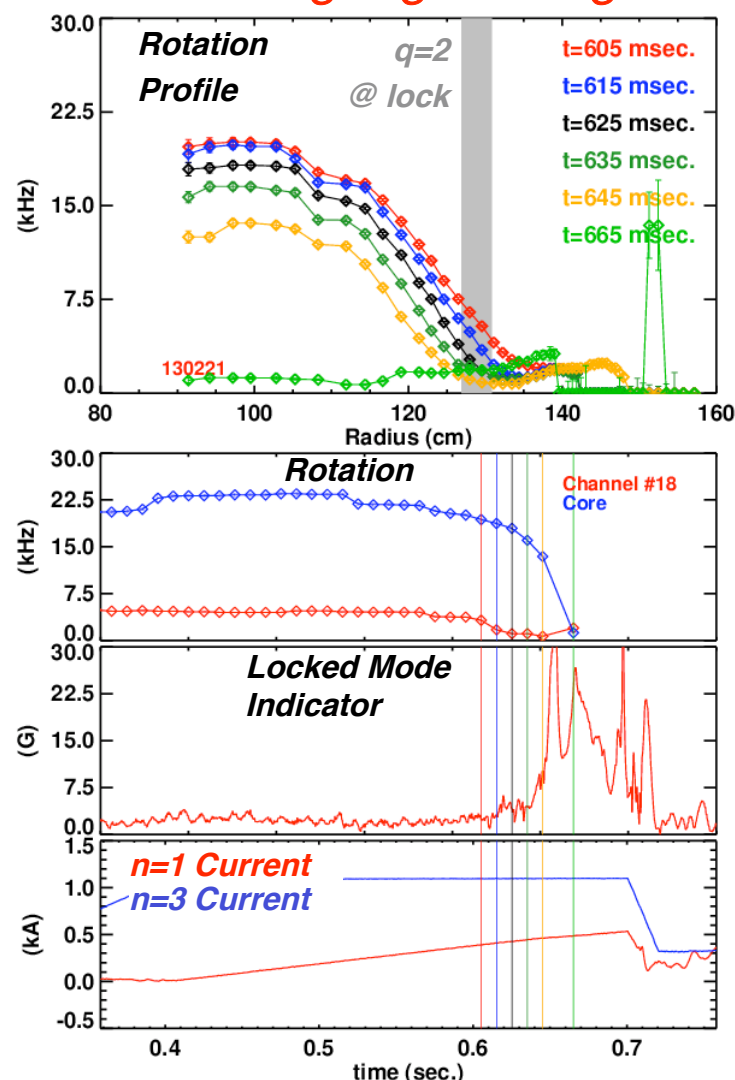


# Different Magnetic Braking Profiles Observed Before EF Penetration

*Large  $n=1$ , Small  $n=3$   
Broad Braking Profile*

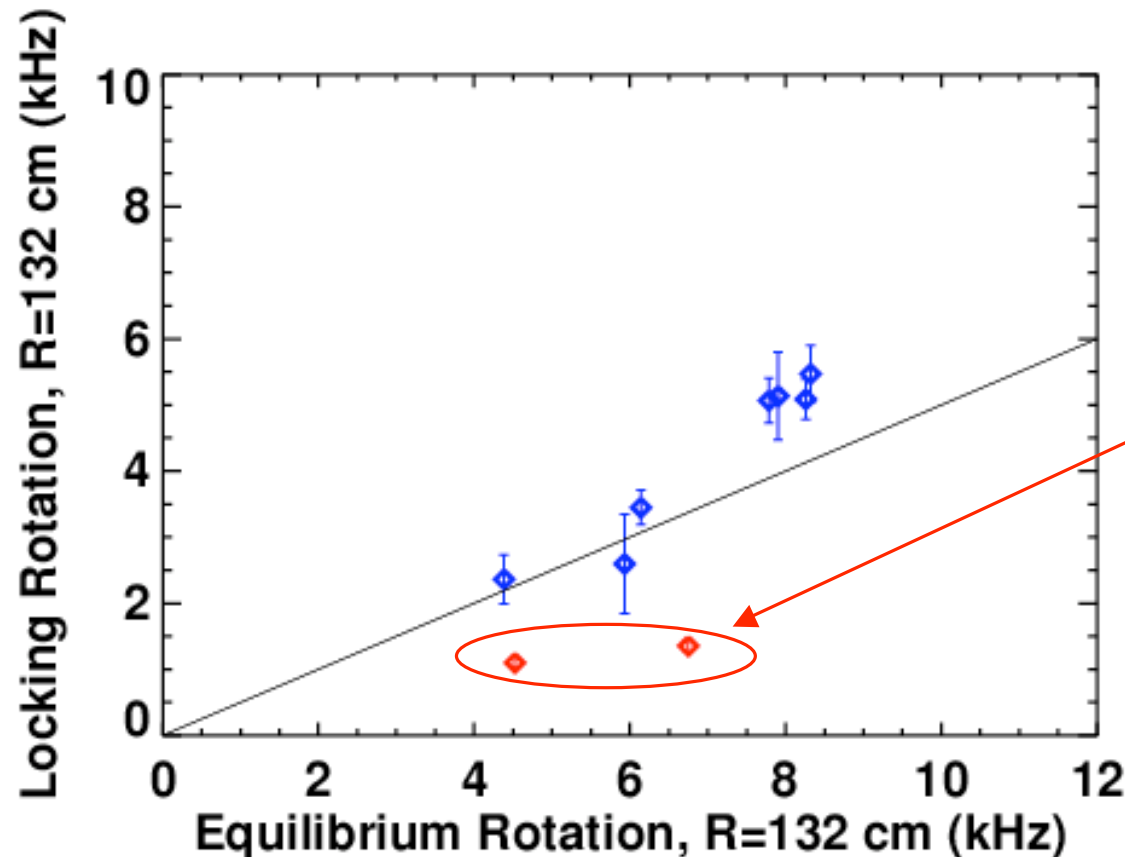


*Large  $n=3$ , Small  $n=1$   
Strong Edge Braking*



## EF Penetration Generally Occurs When Rotation is Reduced by Half

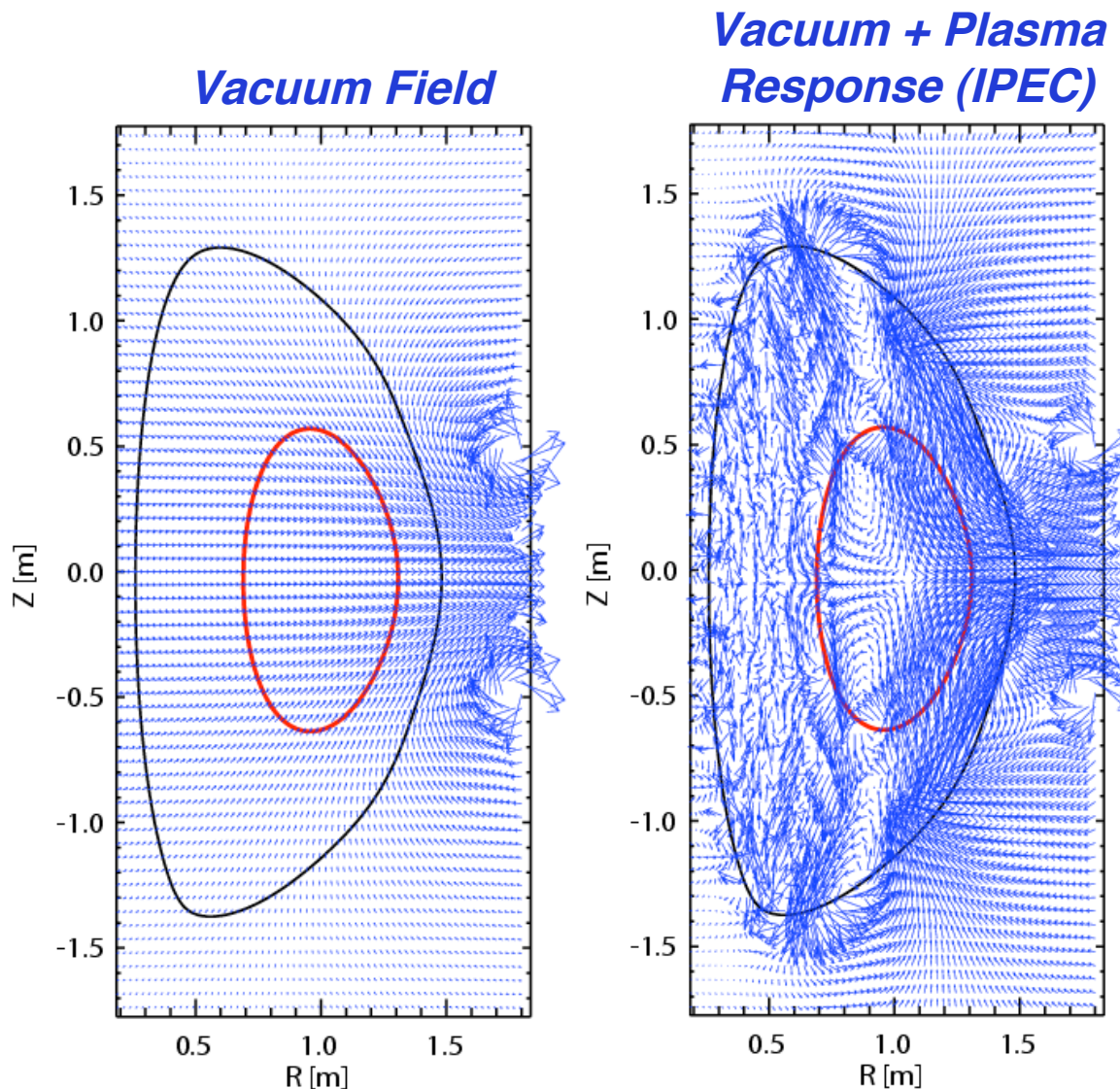
- Use CHERS rotation at R=132 cm, at the typical  $q=2$  radius
- Data only available for a limited # of discharges.
  - Most discharges have no CHERS data after EF penetration.



*Cases where  $I_{n=3} \gg I_{n=1}$   
Very strong edge braking  
proceeds EF penetration.  
Proper calculation of  $\omega_0$ ?*



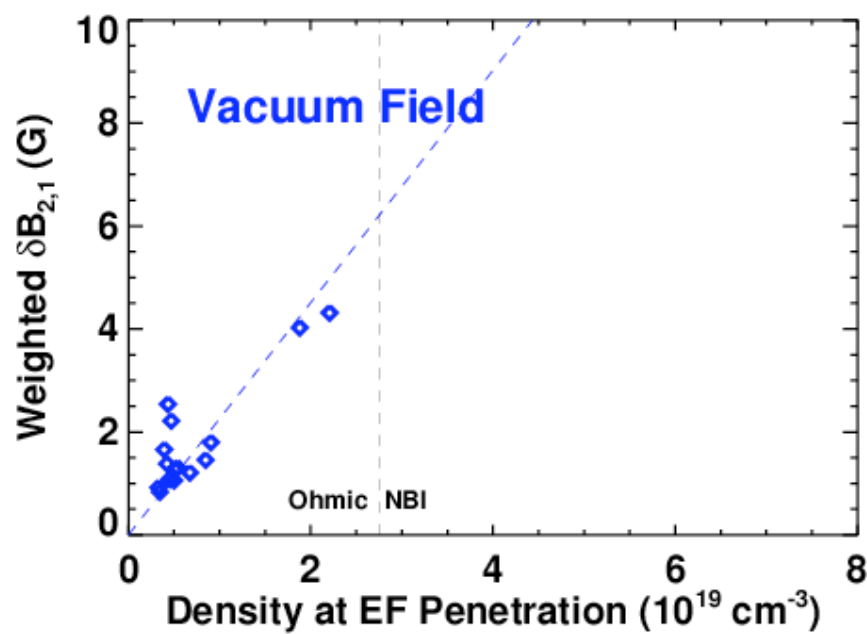
## $\beta$ -Dependence: The Resonant Field that Drives Islands can be Amplified by the Plasma at Higher $\beta_N$



- EFC coil currents and vacuum resonant fields are not the correct quantities when considering locking.
- The resonant field driving islands can be amplified at high- $\beta_N$ .

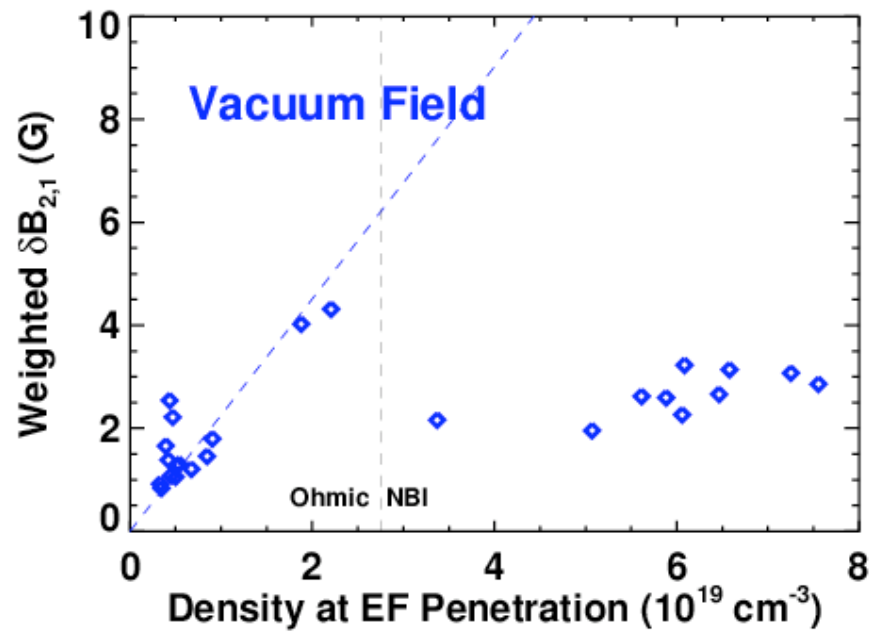
# Linear Correlation Between Resonant Field for Penetration and Density Can be Restored Using IPEC

- Wide variety of data in the scan:
  - Ohmic L-mode plasma at low density



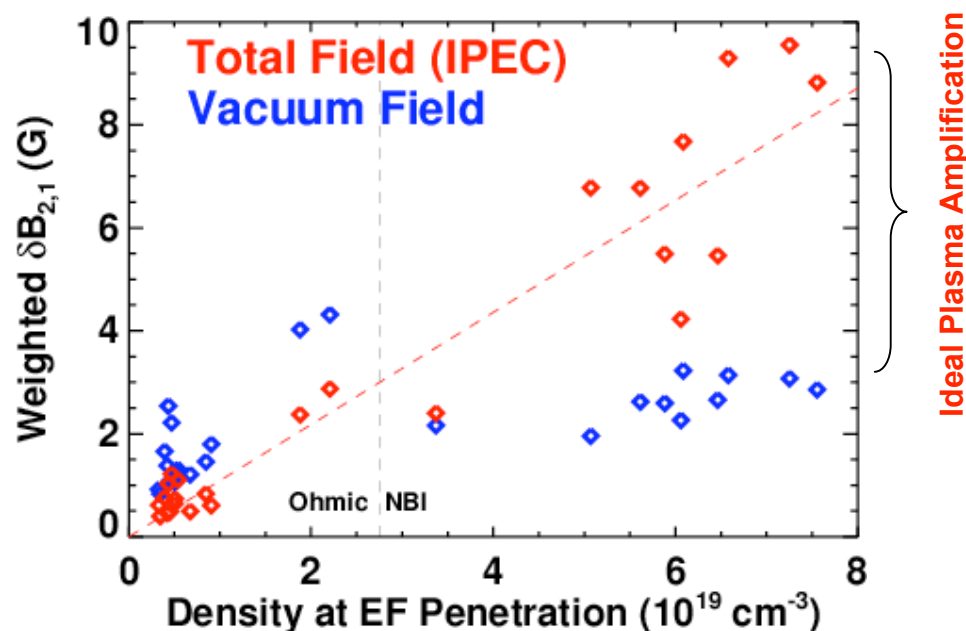
# Linear Correlation Between Resonant Field for Penetration and Density Can be Restored Using IPEC

- Wide variety of data in the scan:
  - Ohmic L-mode plasma at low density
  - NBI-heated H-mode at high density.
- IPEC results demonstrate importance of plasma response:
  - Vacuum: Linear scaling with density fails; error field penetration at high- $\beta$  seems anomalously easy.



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  - IPEC: Error field penetration threshold scales with density.



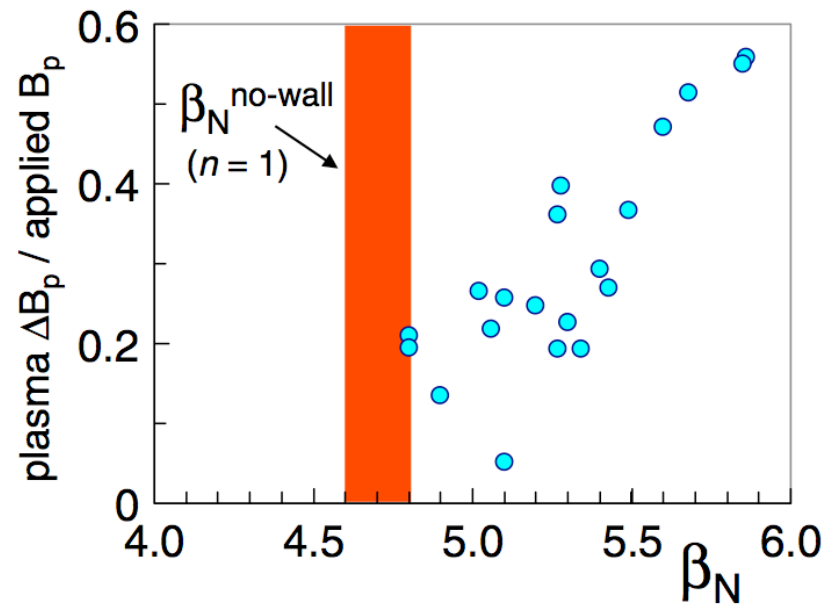
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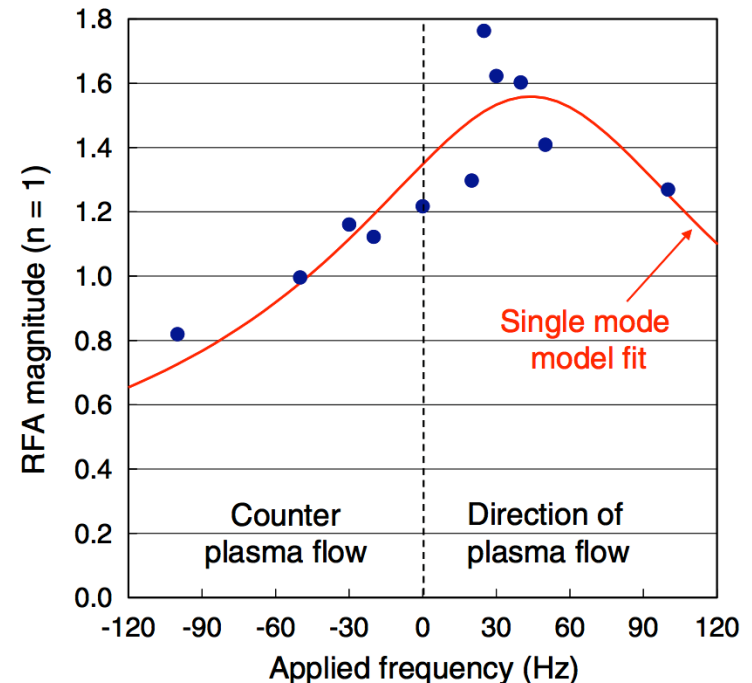
# Reminder: Plasma Amplification of Applied $n=1$ Fields is Well Documented in NSTX

## $\beta$ -Dependent RFA Response to a Standing Wave $n=1$ Field



*S. A. Sabbagh, et al., Nuclear Fusion 46, 635 (2006)*

## Frequency Dependent Response to a Traveling Wave $n=1$ Field



*A. C. Sontag, et al., Nuclear Fusion 47, 1005 (2007)*  
*Supportive of single-mode RFA model.*

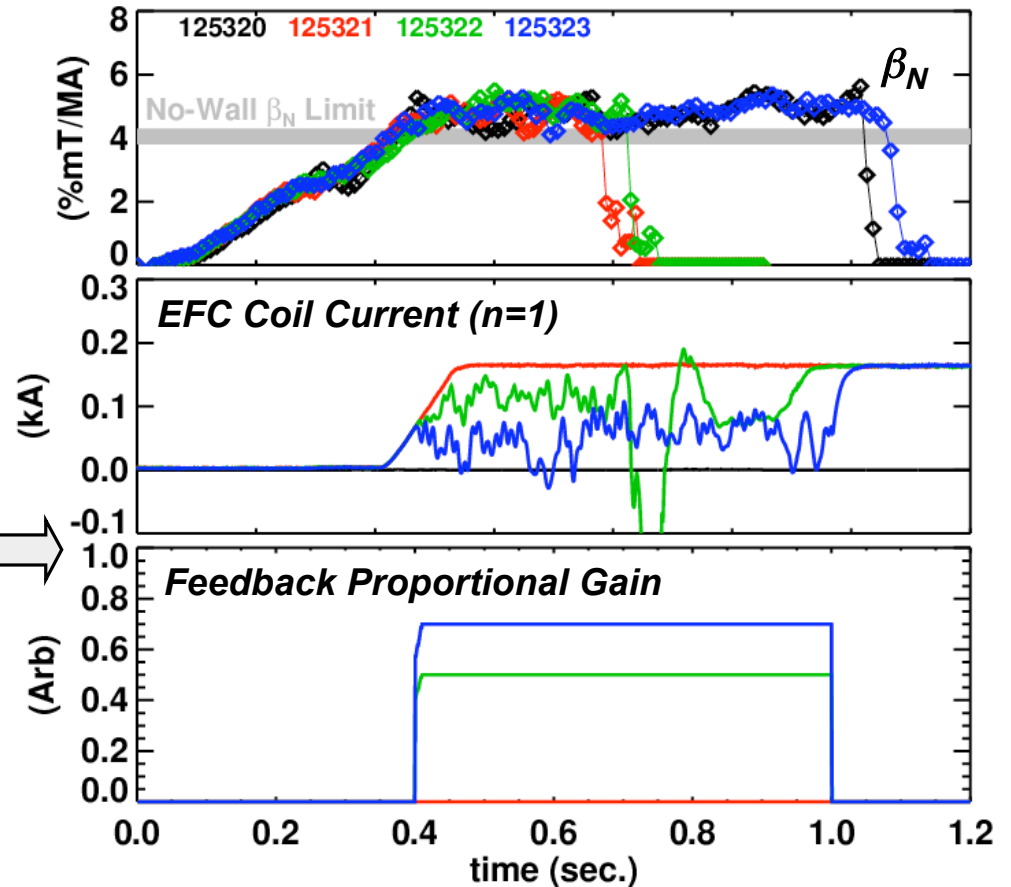
*We can use this physics for  $n=1$  EF correction.*

# Use Plasma Amplification of Error Fields as a Control Tool

- Pre-programmed  $n=1$  EF correction requires a priori estimate of intrinsic EF
- Detect plasma response → EF correction using only feedback on RFA

## RFA Suppression Algorithm

- Use discharge with rotationally stabilized RWM.
- **Deliberately apply  $n=1$  EF in order to reduce rotation, destabilize an RWM.**
- Find feedback phase that reduces the applied  $n=1$  currents ( $B_p$  sensors).
  - Direct coil-sensor pickup is removed.
- Increase the gain until currents are nearly nulled and plasma stability is restored.



→ Use same gain/phase settings to suppress RFA from intrinsic EF **and** any unstable RWMs

J.E. Menard et al., to be published in Phys. Plasmas

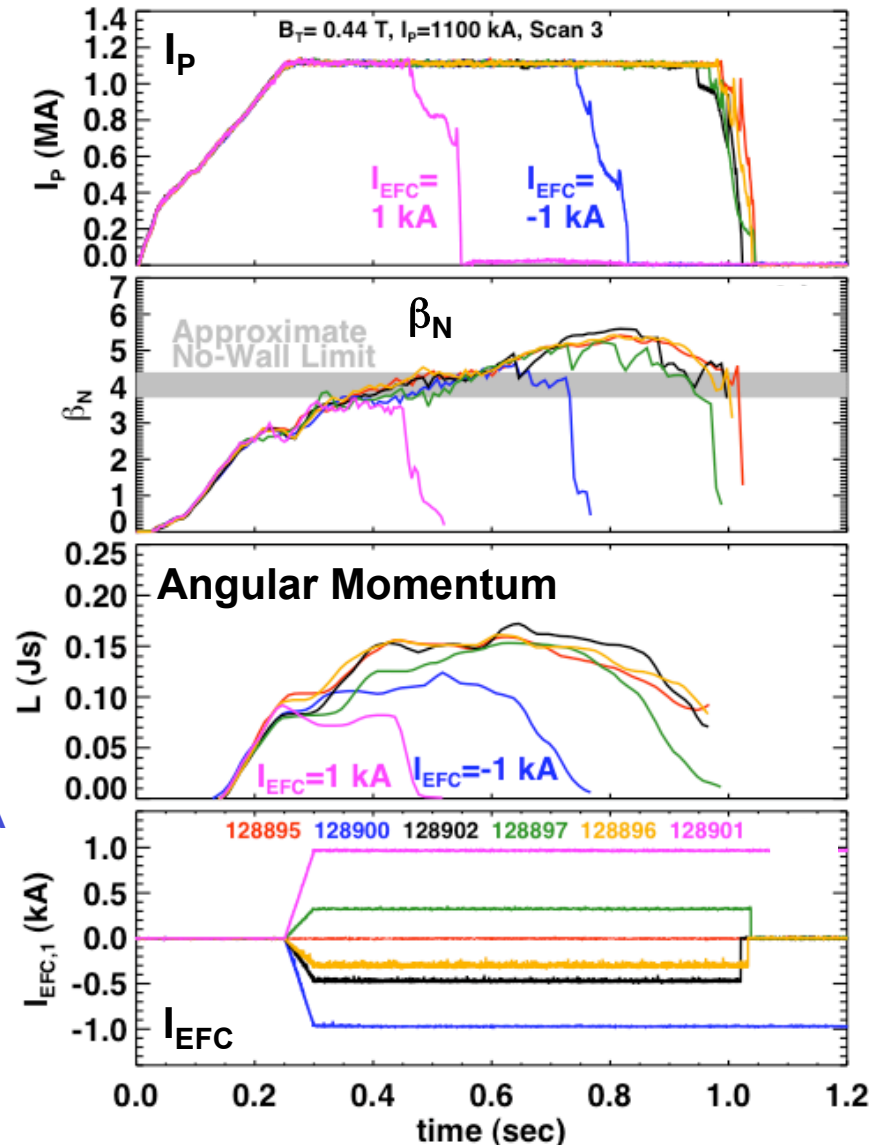
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# Non-Resonant ( $n>1$ ) Error Fields Observed in NSTX

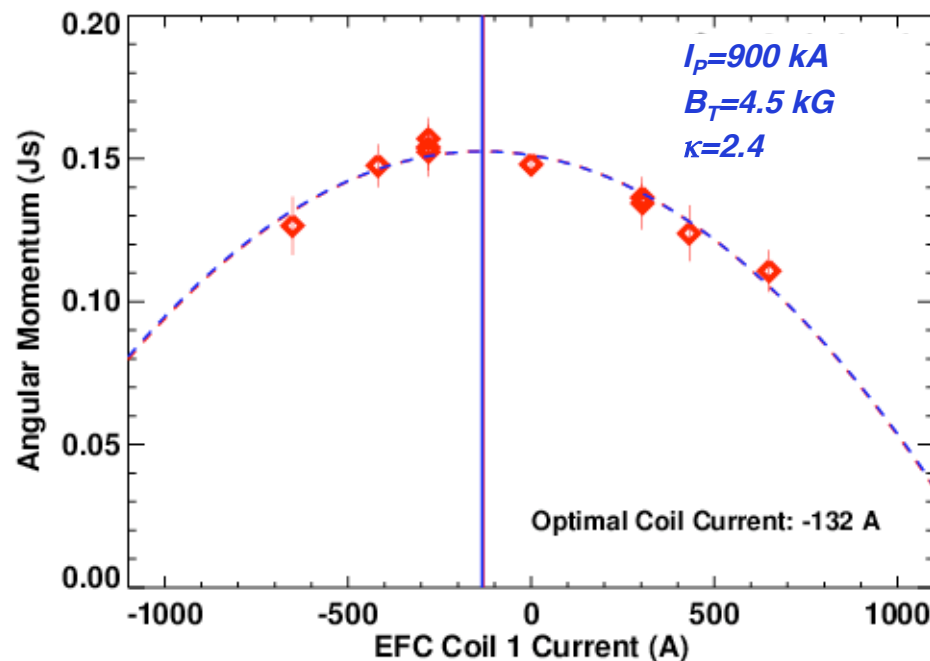
- Use a long-pulse high- $\beta_N$  discharge.
- Apply  $n=3$  fields of two different polarities, many amplitudes.
- Asymmetric response in the pulse length:
  - Discharge with 1000 A applied  $n=3$  field disrupts before that with -1000 A.
- Asymmetric response in the angular momentum:
  - Discharge with  $I_{n=3}=+1000$  A has less angular momentum before the disruption than that with  $I_{n=3}= -1000$  A

*There is an intrinsic  $n=3$  error field.*



# Determine Optimal Correction by Maximizing the Angular Momentum

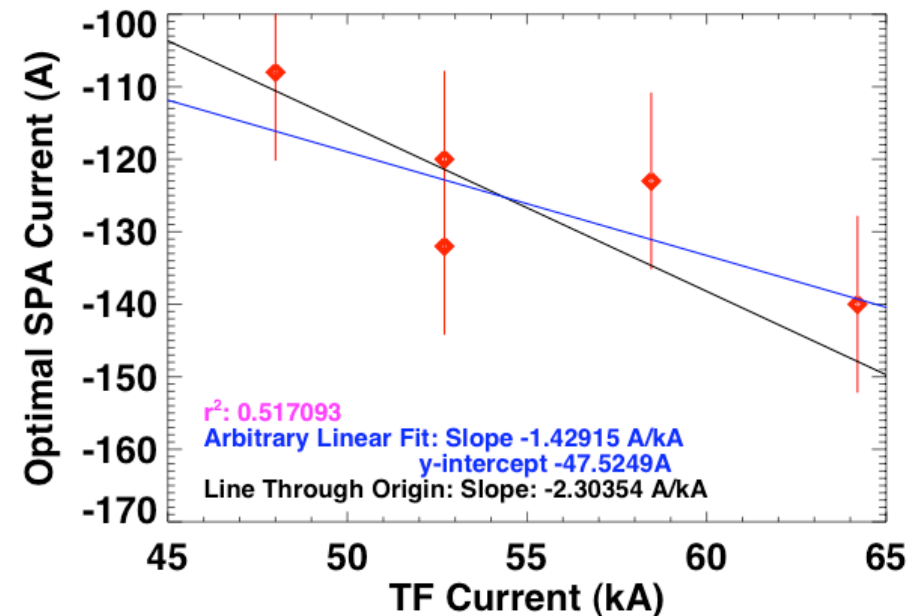
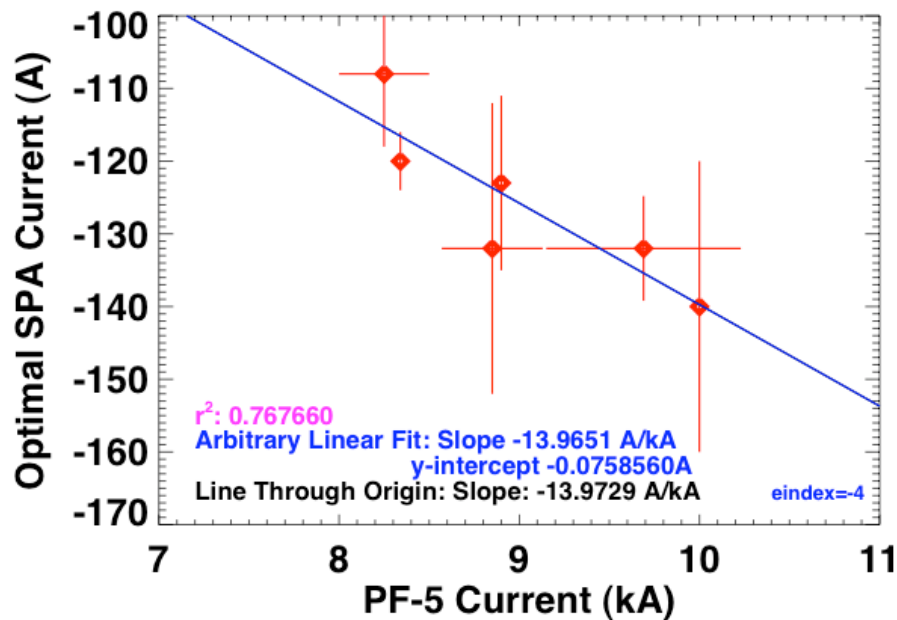
- Likely sources of the EF:
  - Vertical Field Coil  $\rightarrow$  Function of  $I_p$
  - Radial Field Coil  $\rightarrow$  Function of  $I_p, \kappa$
  - Toroidal Field Coil  $\rightarrow$  Function of  $B_T$
- Choose 6 configurations with different values of  $I_p$ ,  $B_T$ , and  $\kappa$ .
- For each configuration, determine the optimal  $n=3$  correction





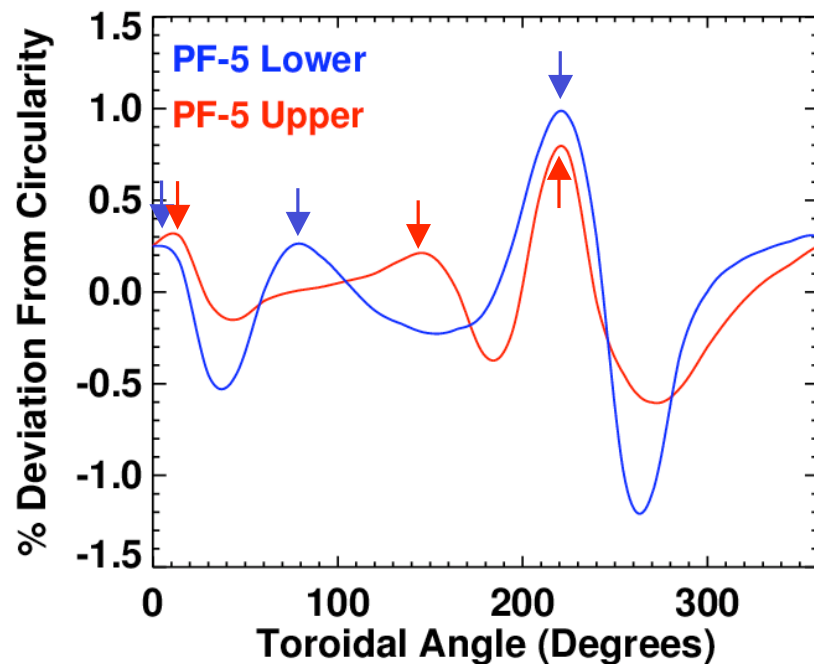
# Experiments Indicate that the Vertical Field Coils are the Source of the Error Field

- Optimal correction correlates well with the current in the vertical field coil.
- Correlation not as strong with the other coils:
  - Correlation coefficients not as large.
  - Best fit lines do not extrapolate to zero correction at zero coil current.
  - Changing the TF direction did NOT change the sign of the correction.



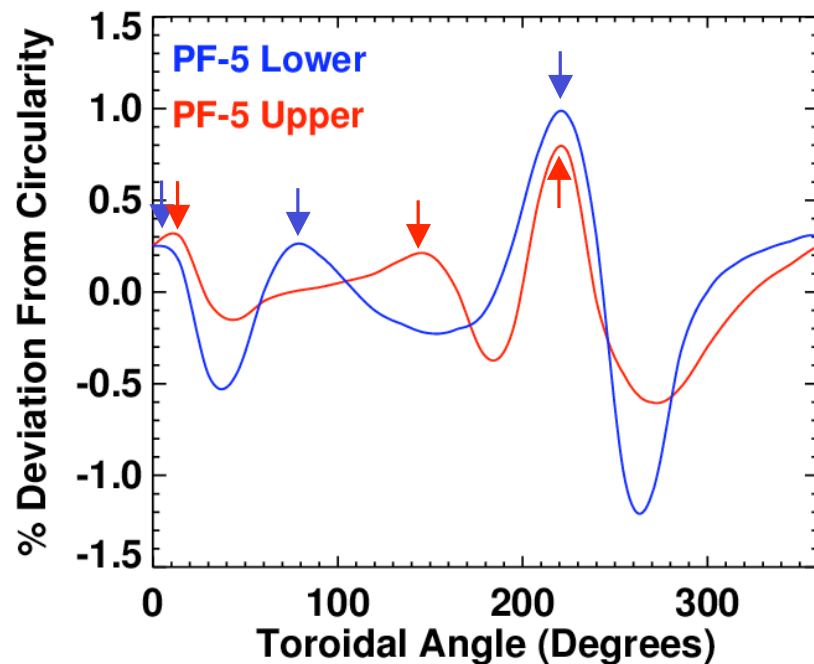
# Correction is Consistent With Known Out-of-Round Vertical Field Coil

- Vertical field coils have a dominantly  $n=3$  radial variation.
  - Makes an  $n=3$  error field.
- Experimental correction  $I_{EFC}/I_{PF-5} \approx 14 \text{ A/kA}$
- Optimal vacuum correction:  $I_{EFC}/I_{PF-5} \approx 18.5 \text{ A/kA}$

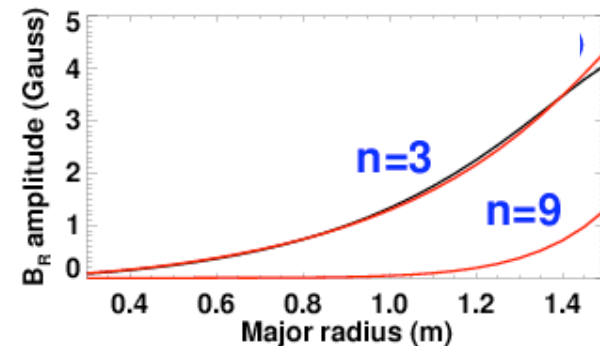
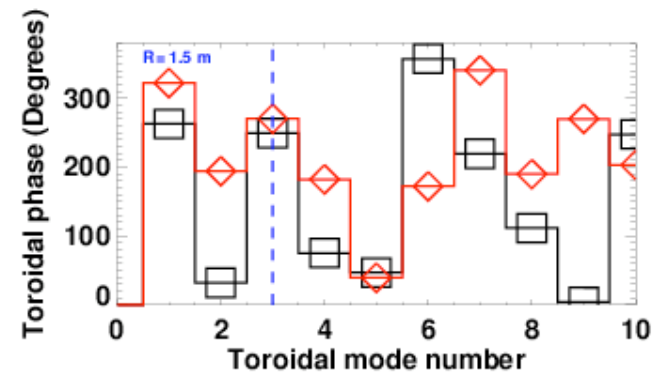
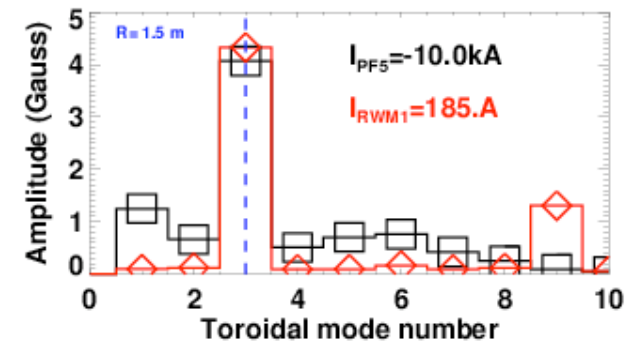


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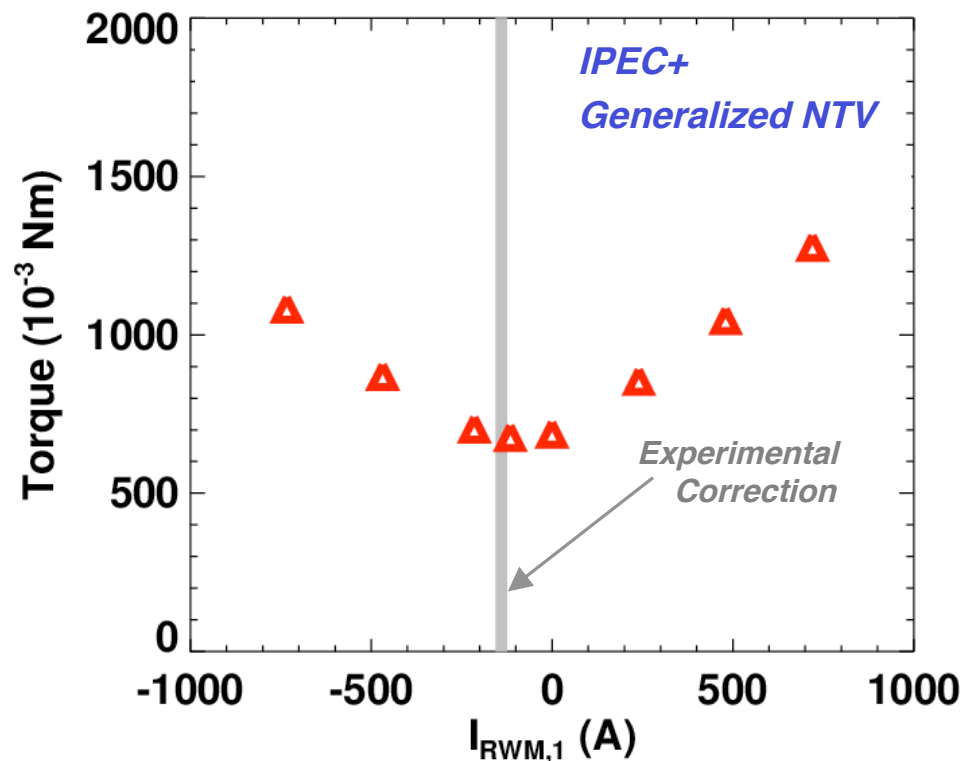


## Error Field Correcting Field



# Minimal NTV Torque at the Experimentally Determined Correction

- Computed the expected braking torque using the IPEC total field and generalized NTV theory.<sup>1</sup>
- Include the intrinsic EF, scan the EFC coil current.
- Predicts minimum total torque at  $I_{EFC}/I_{PF-5} \approx 15 \text{ A/kA}$



**Torque magnitude is consistent with experiment**

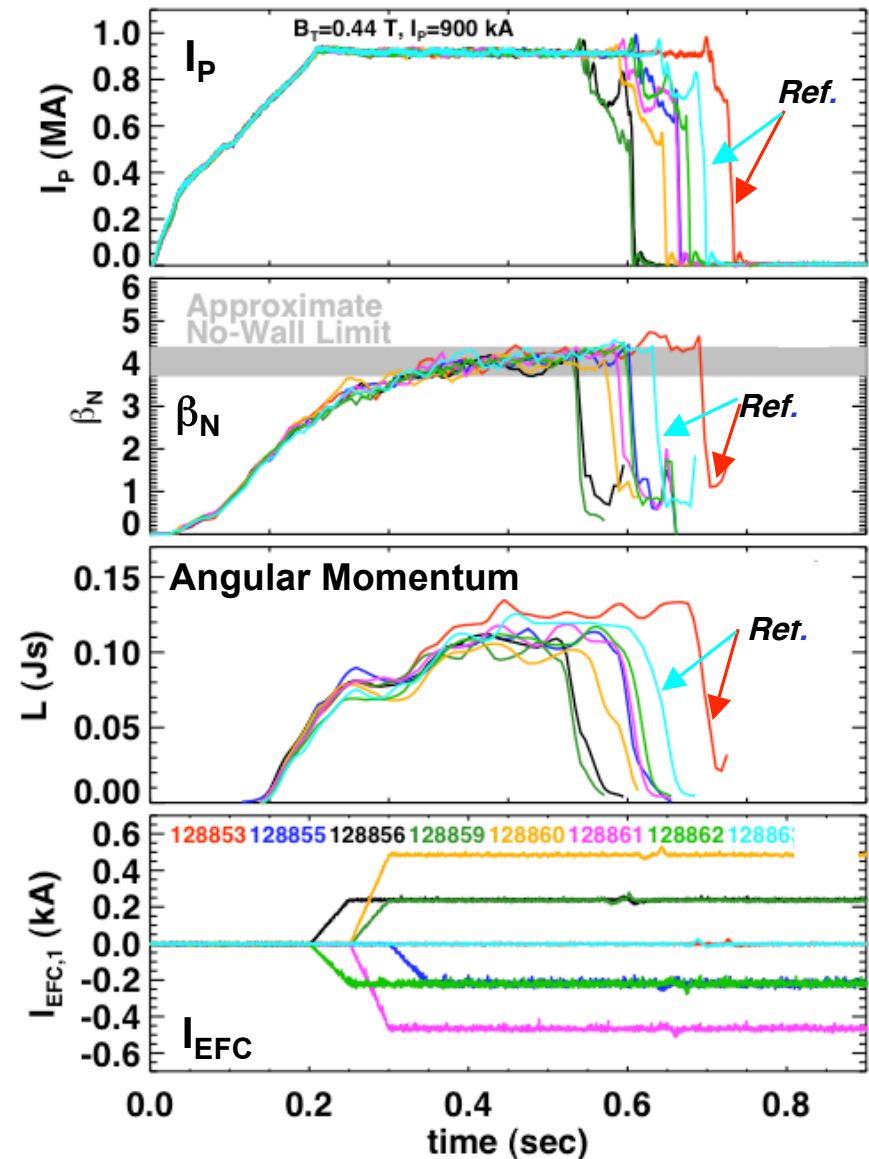
• Angular momentum changes by 0.05 J·s in ~100 msec

• Implied torque of .05/0.1=0.5 N·m is consistent with the change in calculated torque.

[1] J. Park et al, Phys. Rev. Lett. **102**, 065002 (2009)

## Error Fields with $n=2$ Appear to be Beneath the Detection Threshold

- Utilize a long-pulse, high- $\beta$  discharge.
  - Reference shots suffer an RWM followed by disruption at  $t \sim 0.7$  sec.
- Apply  $n=2$  fields of various phases.
- $n=2$  fields always cause the disruption to occur earlier.
- Momentum damping occurs in all cases with applied fields.
- Infer that  $n=2$  error fields, if present, are small.
  - Consistent with modeled shape of the VF coil producing a dominantly  $n=3$  EF.





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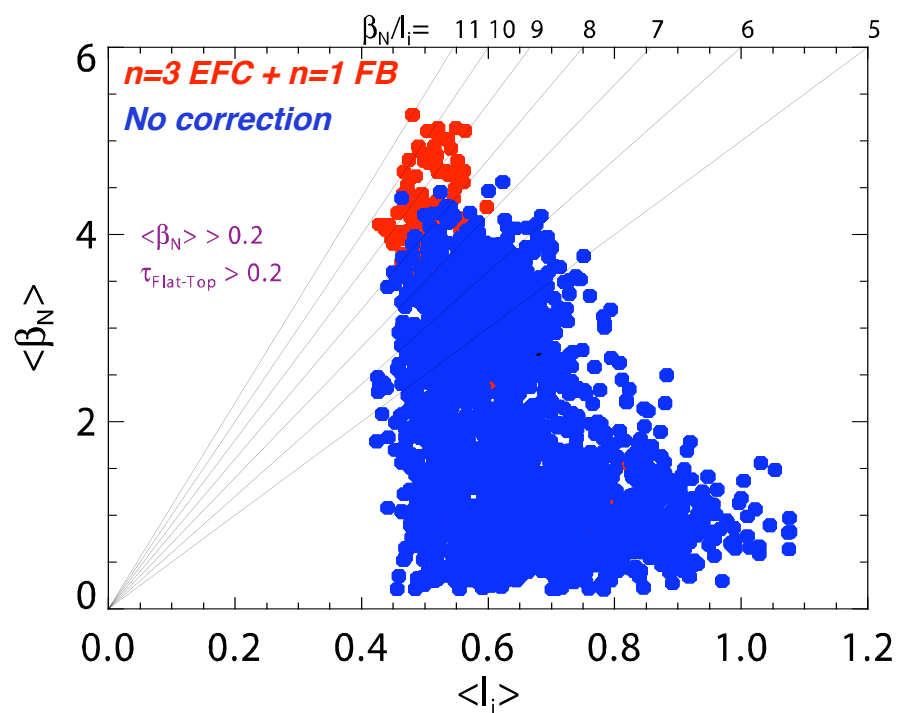
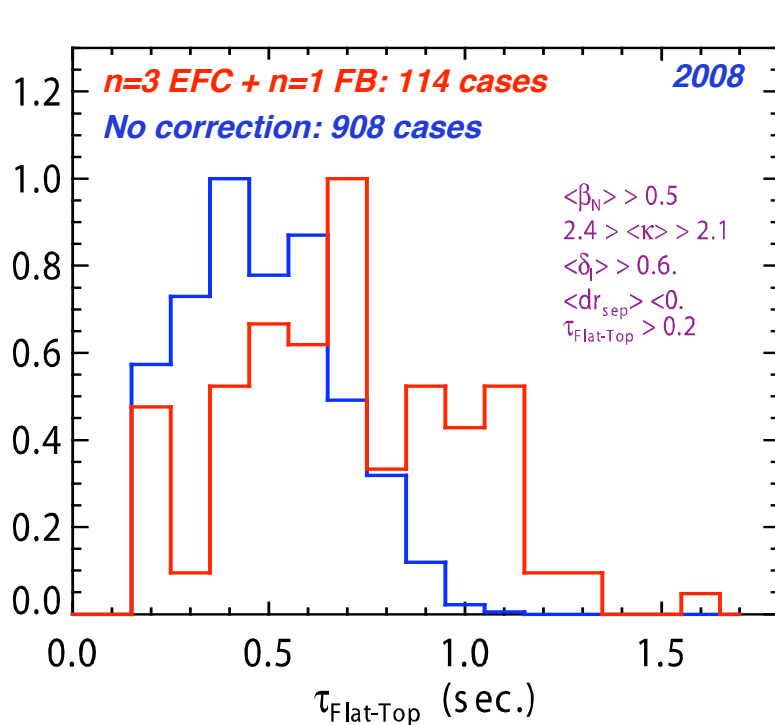
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# Combined n=1 Feedback and n=3 Correction Yield Improved Performance

*n=1 DEFC (slow) and RWM (fast) feedback combined in these cases*

Probability of a long-pulse shot increases with n=3 correction and n=1 feedback.

High average values of  $\beta_N/I_i$  achievable with n=3 correction and n=1 feedback.



*No equivalent data for 2009 run...DEFC used on nearly every NB heated shot starting from the beginning of the run.*

[1] S. Sabbagh et al., submitted to Nuclear Fusion.

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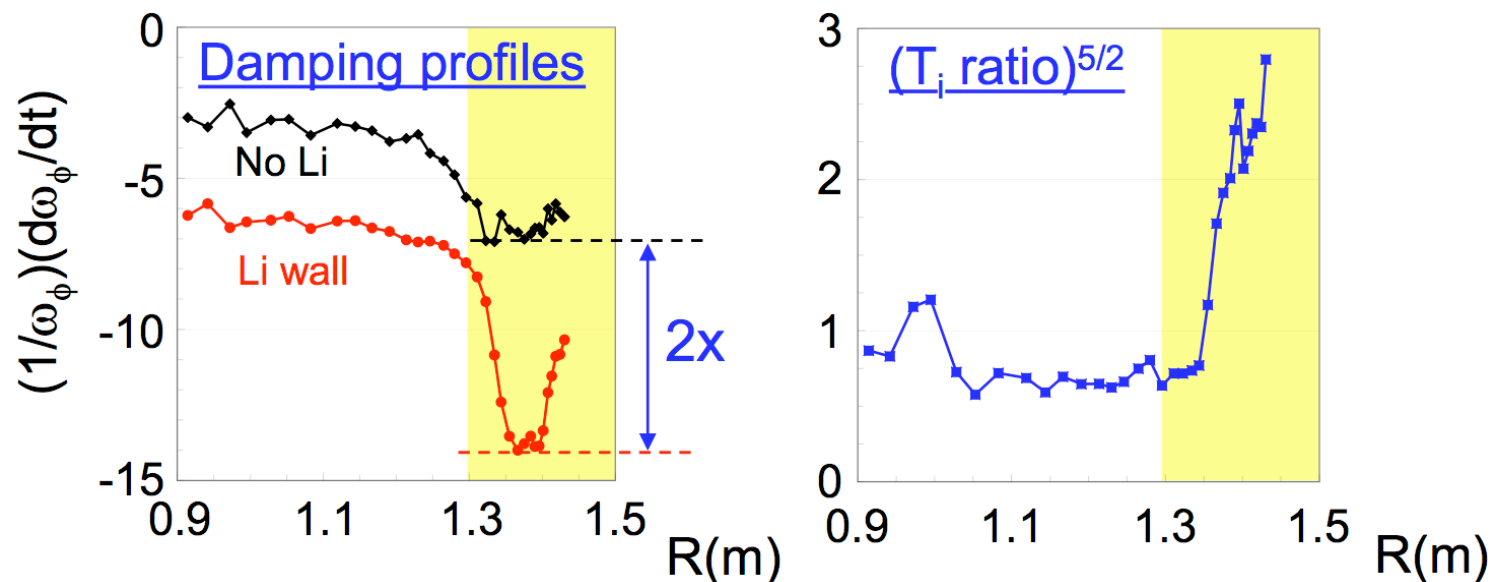
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# Stronger Non-Resonant Braking at Increased $T_i$

- Neoclassical toroidal viscosity (NTV) is the damping of toroidal flows due to the loss of toroidal symmetry.
  - Neoclassical fluxes lose their intrinsic ambipolarity.
  - Results in a braking torque.
- Expression in “1/ $\nu$ ” regime,  $q\omega_E < \nu_i/\varepsilon < \sqrt{\varepsilon}\omega_{ti}$

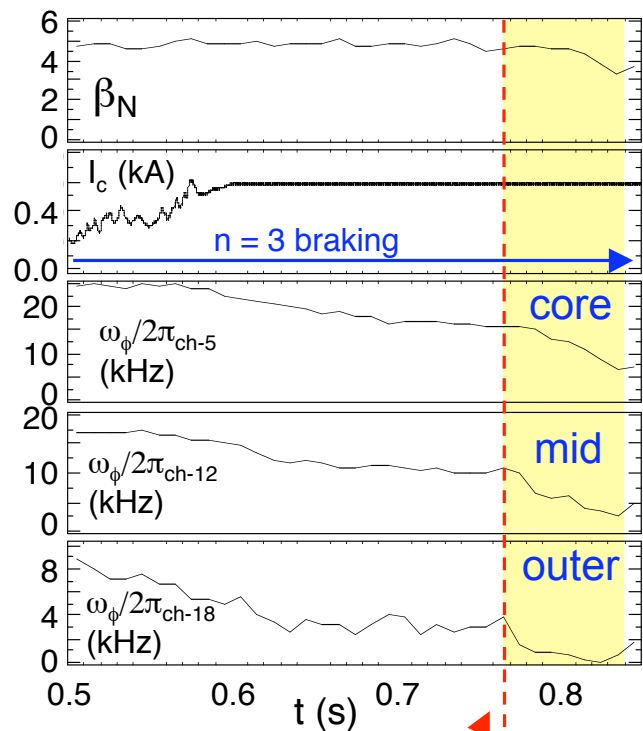
$$\left\langle \hat{e}_t \cdot \vec{\nabla} \cdot \vec{\Pi} \right\rangle_{(1/\nu)} = B_t R \left\langle \frac{1}{B_t} \right\rangle \left\langle \frac{1}{R^2} \right\rangle \frac{\lambda_{1i} p_i}{\pi^{3/2} \nu_i} \varepsilon^{3/2} (\Omega_\phi - \Omega_{NC}) I_\lambda \quad \longrightarrow \quad \frac{d\Omega}{dt} \propto \frac{p_i}{\nu_i} \Omega \quad \longrightarrow \quad \frac{1}{\Omega} \frac{d\Omega}{dt} \propto T_i^{5/2}$$

- Use Lithium evaporation to increase  $T_i$ , apply n=2 fields and measure rotation damping.
- Verified  $T_i^{5/2}$  dependence of NTV torque in region of max braking.



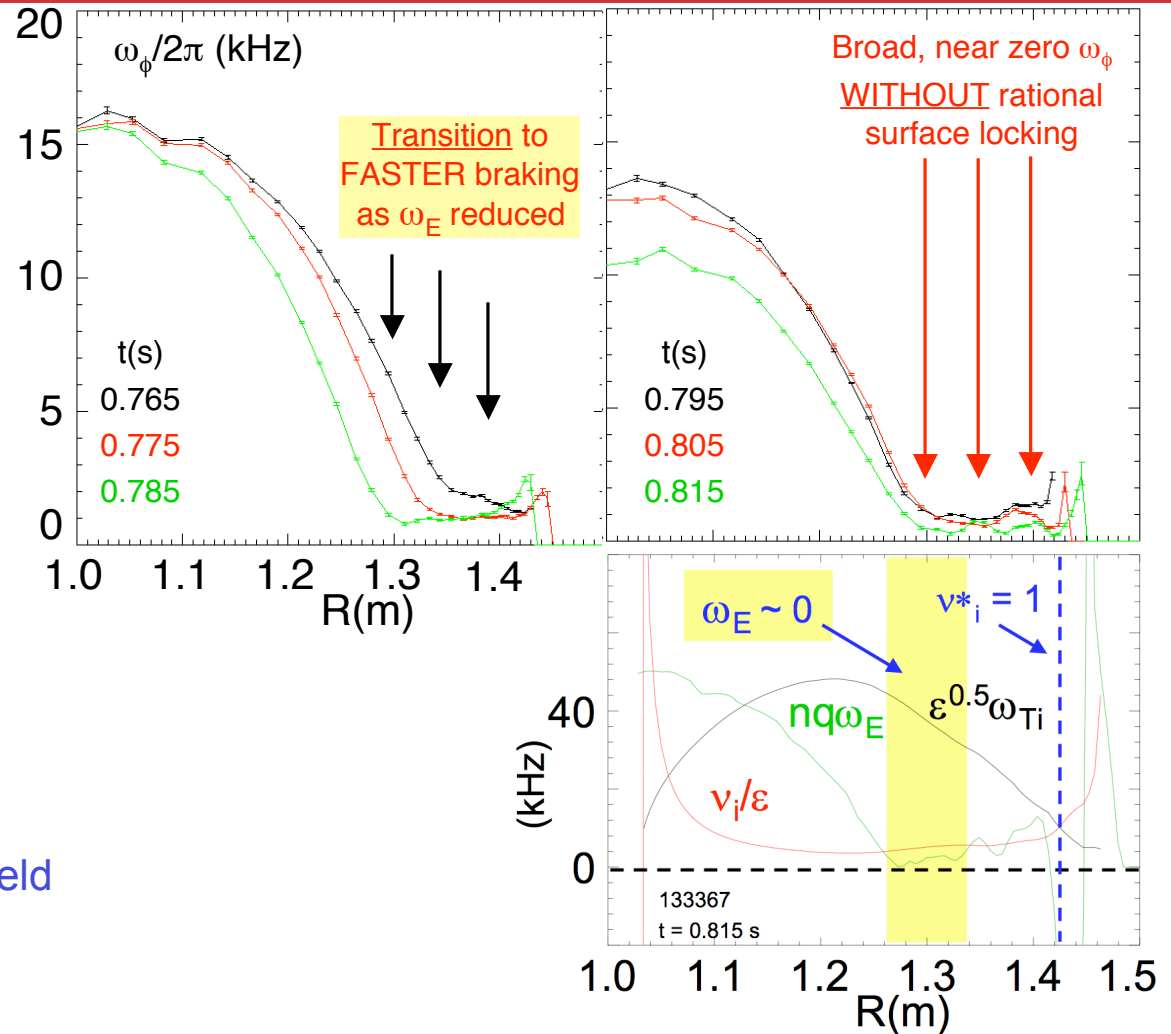
S. Sabbagh, et al, 2008 IAEA FEC & submitted to Nuclear Fusion

# Stronger Braking With **Constant $n = 3$ Applied Field** as $\omega_E$ Reduced – Accessing Superbanana Plateau NTV Regime



- **Faster braking with**
  - Constant  $\beta_N$ , applied  $n = 3$  field
  - No mode activity

- Torque not  $\propto 1/\omega_\phi$  (non-resonant)
  - NTV in “ $1/\nu$  regime” ( $|\ln q \omega_E| < \nu_i/\epsilon$  and  $\nu_i^* < 1$ )
  - Stronger braking expected when  $\omega_E \sim 0$  (superbanana plateau)
  - Important for predicting NTV effects in low-rotation plasmas  $\rightarrow$  RWMs, NTMs, rotation control, RMP effects



**S.A. Sabbagh (CU)**

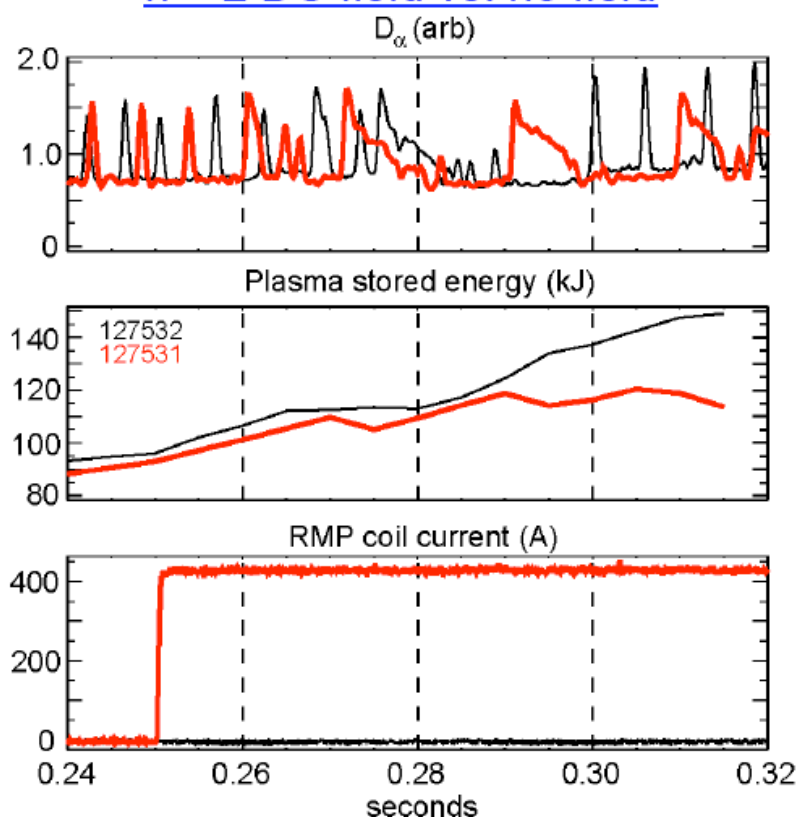
# Introduction and Preview

- Background:
  - The NSTX Device
  - The IPEC code
- Effects of 3-D fields on the core plasma
  - Studies of  $n=1$  error field penetration in high- $\beta$  plasmas.
  - Dynamic correction of  $n=1$  error fields.
  - Non-resonant error field detection and correction.
  - Improved NSTX performance with resonant and non-resonant EF correction.
  - Neoclassical toroidal viscosity (NTV).
- Effects of 3-D fields on the edge plasmas
  - Magnetic ELM pacing
  - 3-D effects on the divertor footprint.
- Summary

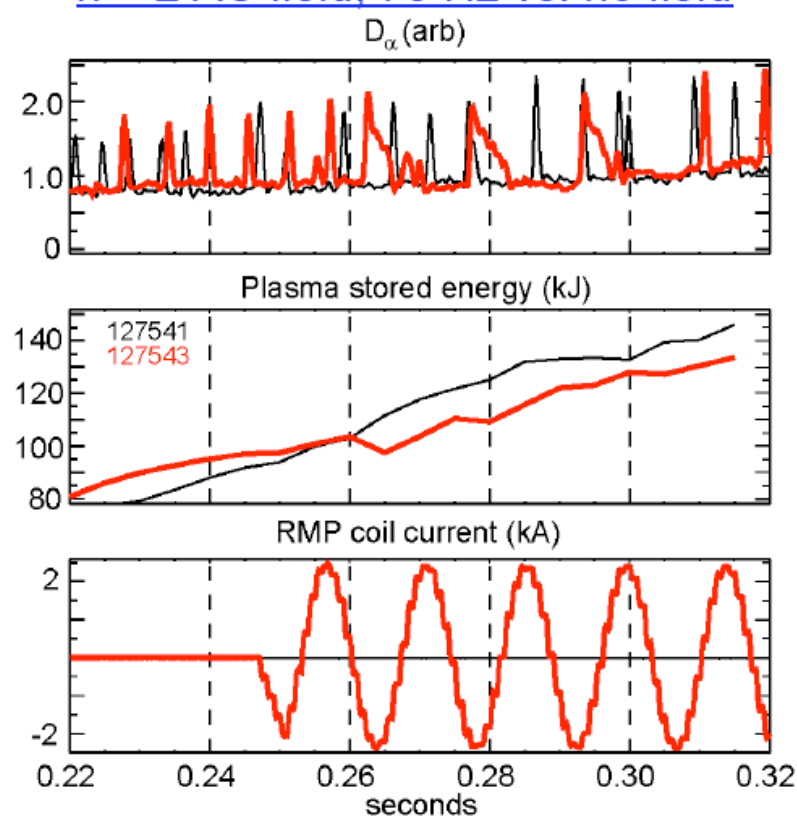
# 3-D Field Modified ELMs in NSTX, But Suppression Not Observed

- Satisfy the standard vacuum Chirikov  $> 1$  outside of  $\psi_N > \sim 0.85$ .
- Broadly similar results for  $n=3$ ,  $n=2+3$  fields.
- See S. Sabbagh's GA RMP workshop talk.

$n = 2$  DC field vs. no field



$n = 2$  AC field, 70 Hz vs. no field

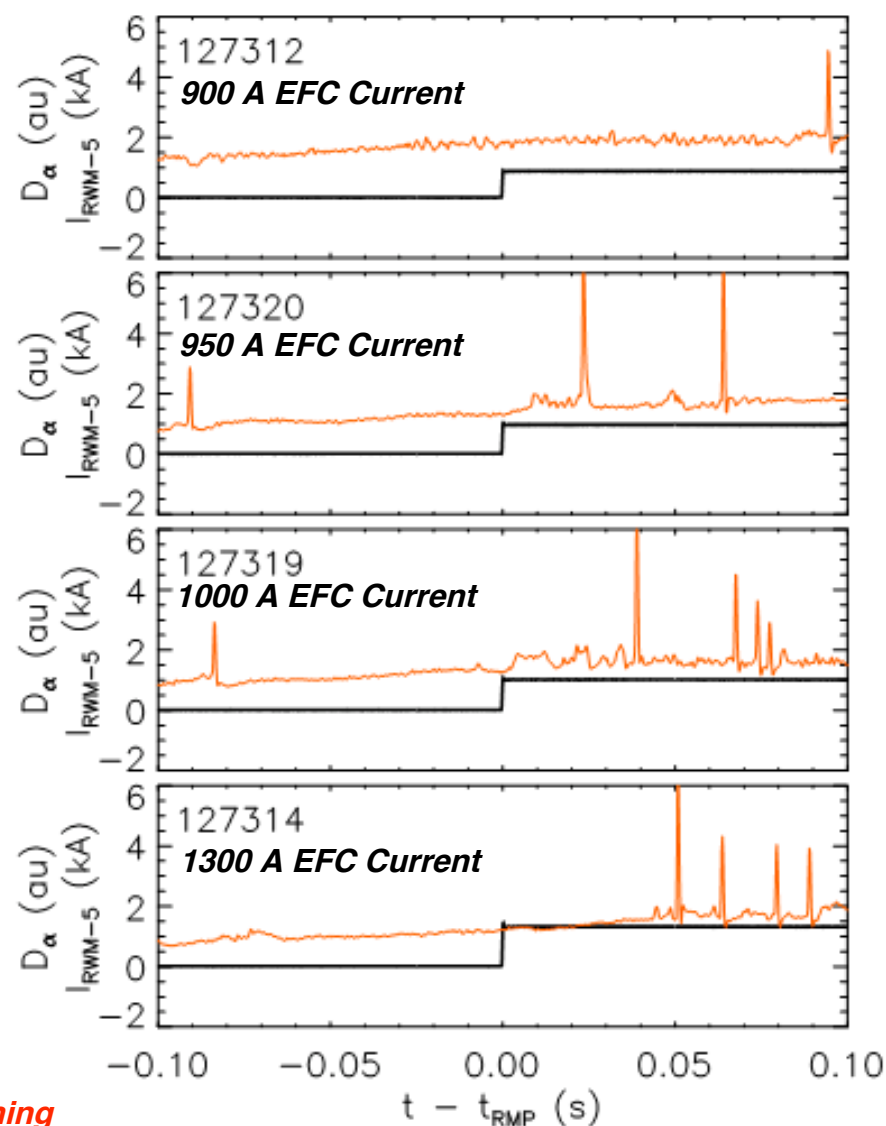


***If 3-D Fields Don't Suppress ELMs in NSTX, then use them for ELM triggering!***



# EFC Current Scan Shows Threshold for ELM Destabilization with 3-D Fields

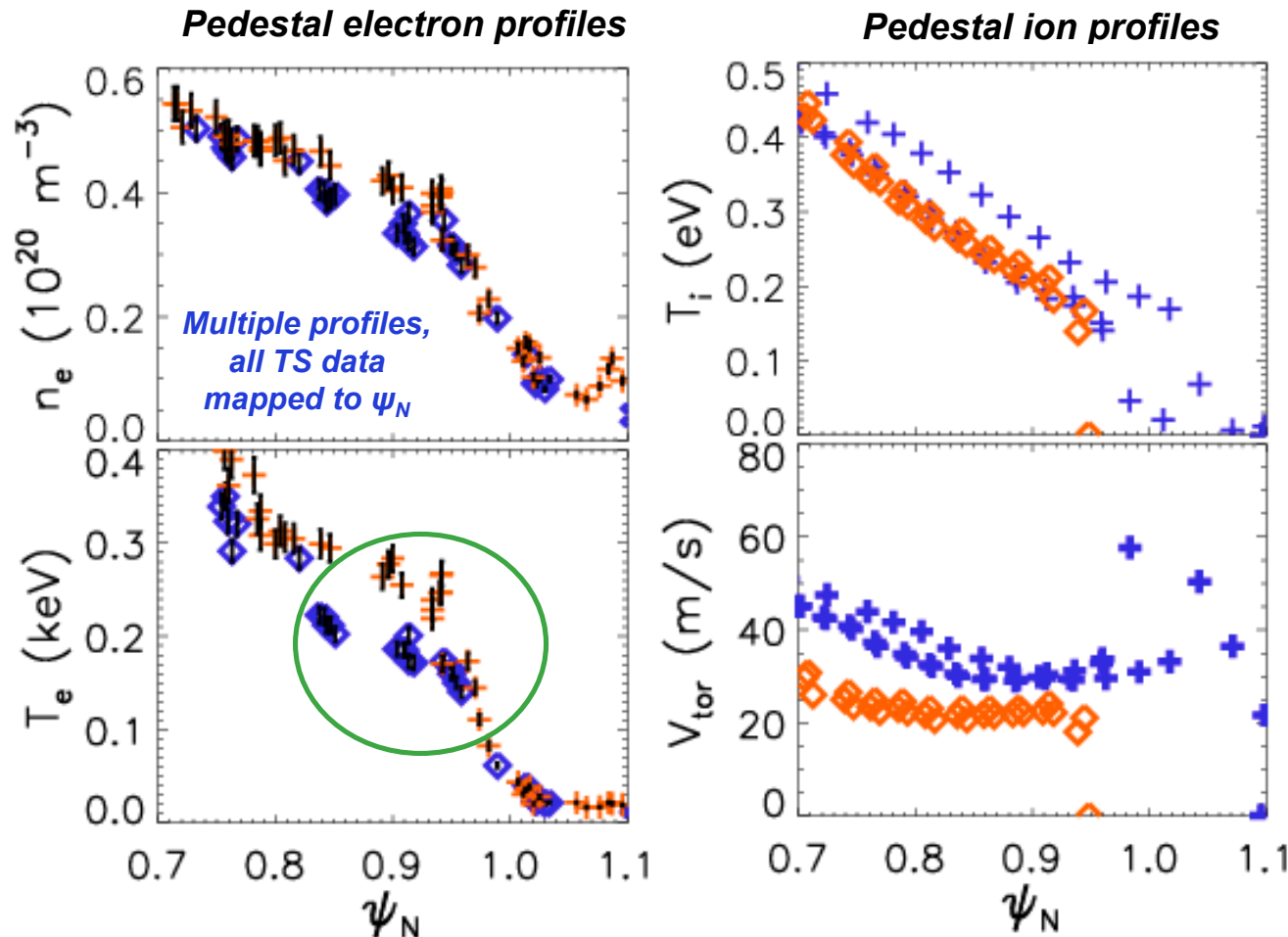
- Threshold coil current for ELM-triggering is  $\sim 950$  A  
- $\rightarrow \Delta B/B = 6 \times 10^{-3}$ 
  - No triggering at 900 A (natural ELMs start at  $\sim 0.5$ s in control discharge)
  - Intermittent ELMs at 950 and 1000 A
- ELM frequency appears to increase with  $n=3$  field magnitude
  - ELMs become more regular
  - Trend clouded by tendency of plasma to lock high currents-too much braking



# $T_e^{\text{ped}}$ Increases When n=3 Fields are Applied

- Blue profiles: no n=3 applied
- Red profiles: 20 ms after n=3 applied (before ELMs)

**No lithium coatings in these shots**

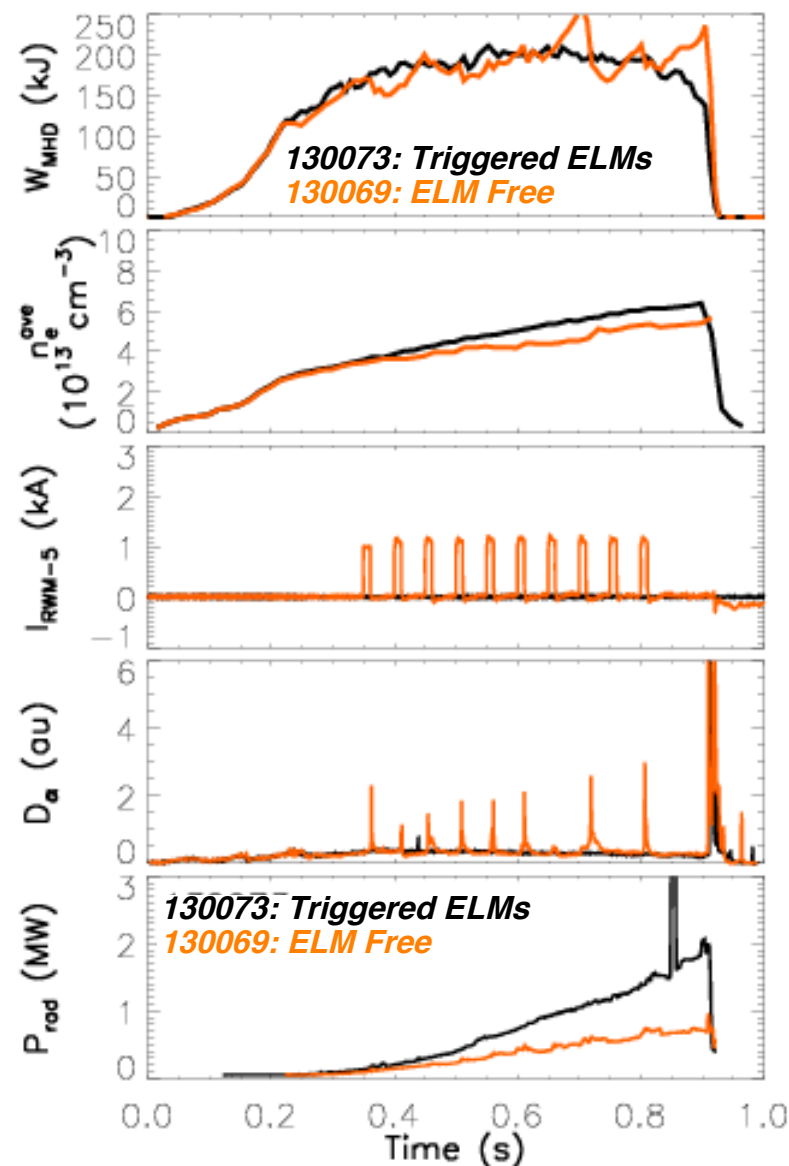


- No density pumpout is observed
- $T_e$ , pressure gradient increases after n=3 field is applied
  - Tanh fitting gives ~30% increase in peak pressure gradient
- PEST shows edge unstable after n=3 application

J.M. Canik, A.C. Sontag, R. Maingi (ORNL)

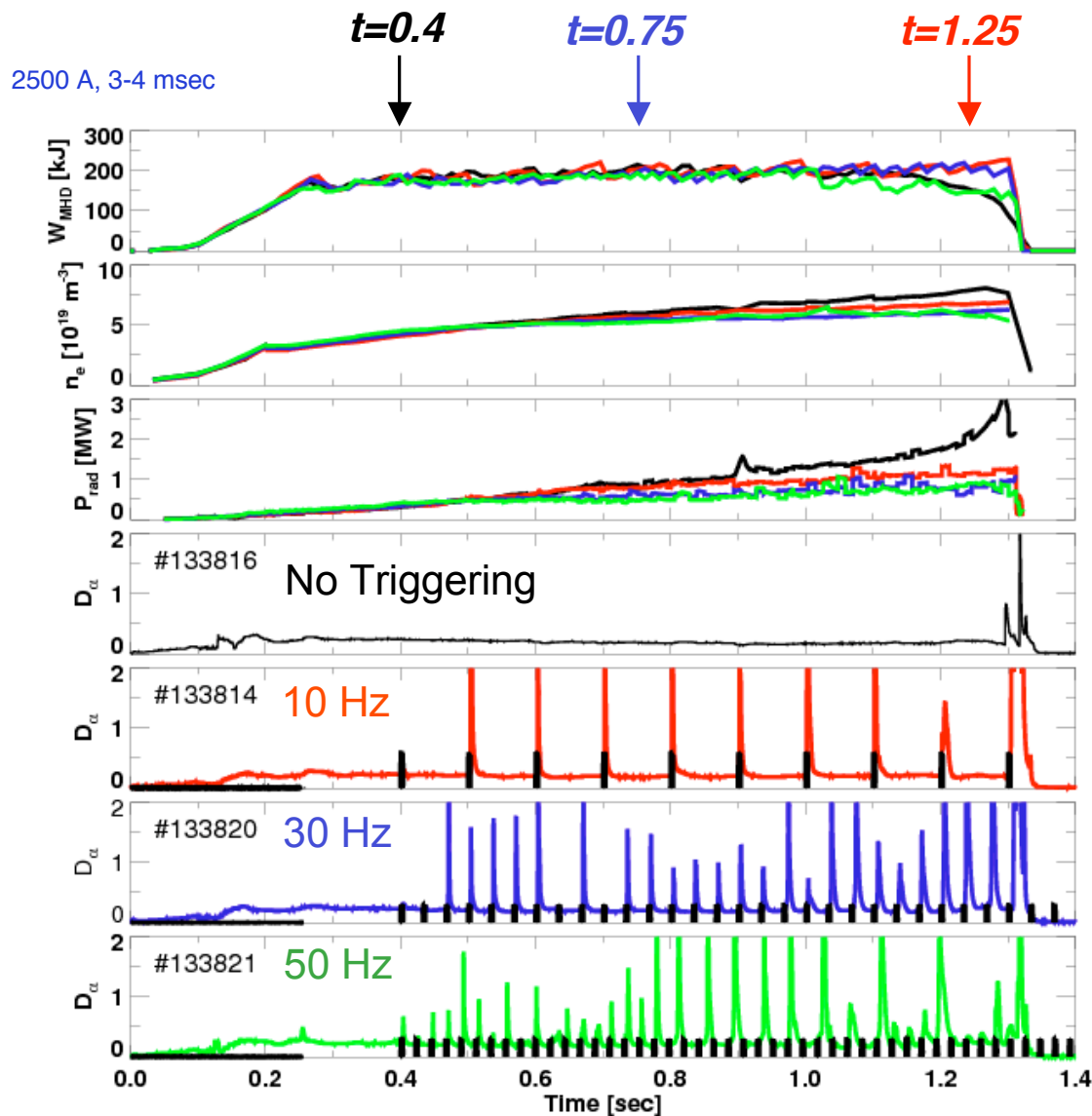
# Magnetic ELM Triggering has been Applied to Lithium Enhanced ELM-free H-modes

- Typical behavior with Li wall conditioning:
  - ELMs suppressed
  - $P_{\text{rad}}$  ramps to  $\sim 2$  MW, for  $P_{\text{NBI}}=3$  MW
- Steady  $n=3$  fields not optimal for ELM triggering.
  - Too much rotation braking.
- Square-wave  $n=3$  fields applied to trigger ELMs.
  - 10 ms pulses,  $f=20$  Hz, 1.2 kA
  - ELMs triggered on 8 of 10 pulses
  - Radiated power reduced by a factor of 2.
  - Density ramp rate reduced
  - Stored energy relatively unaffected.

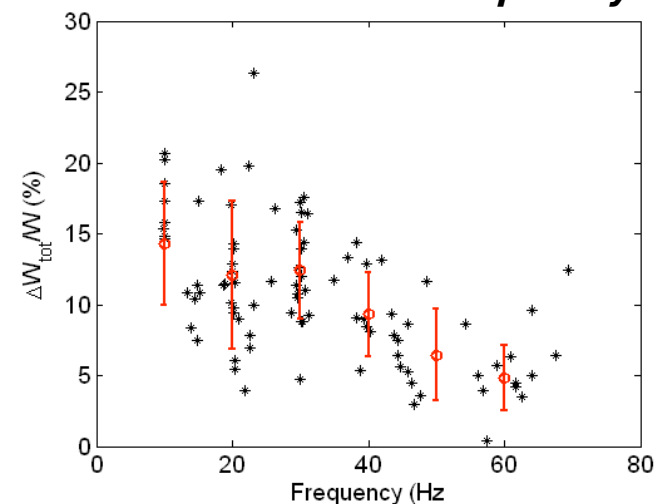


J.M. Canik, A.C. Sontag, R. Maingi (ORNL)

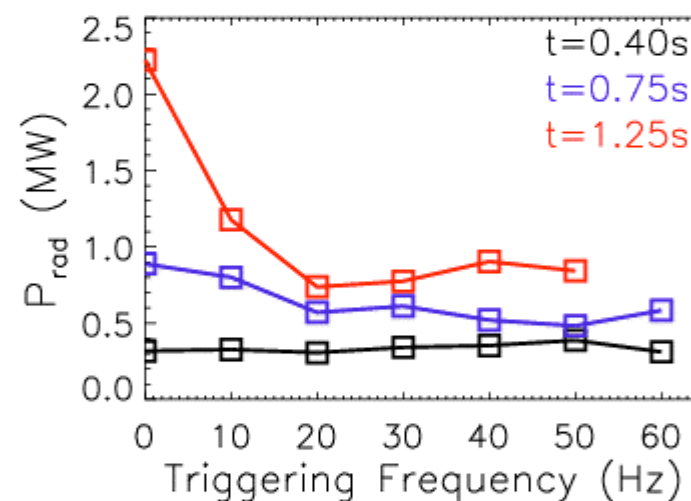
# Maximizing the n=3 Pulse Amplitude Allows Rapid Frequency Triggering with Very High Reliability



## ELM Size vs. Frequency



## $P_{Rad}$ vs. Triggering Frequency



J.M. Canik, A.C. Sontag, R. Maingi (ORNL)

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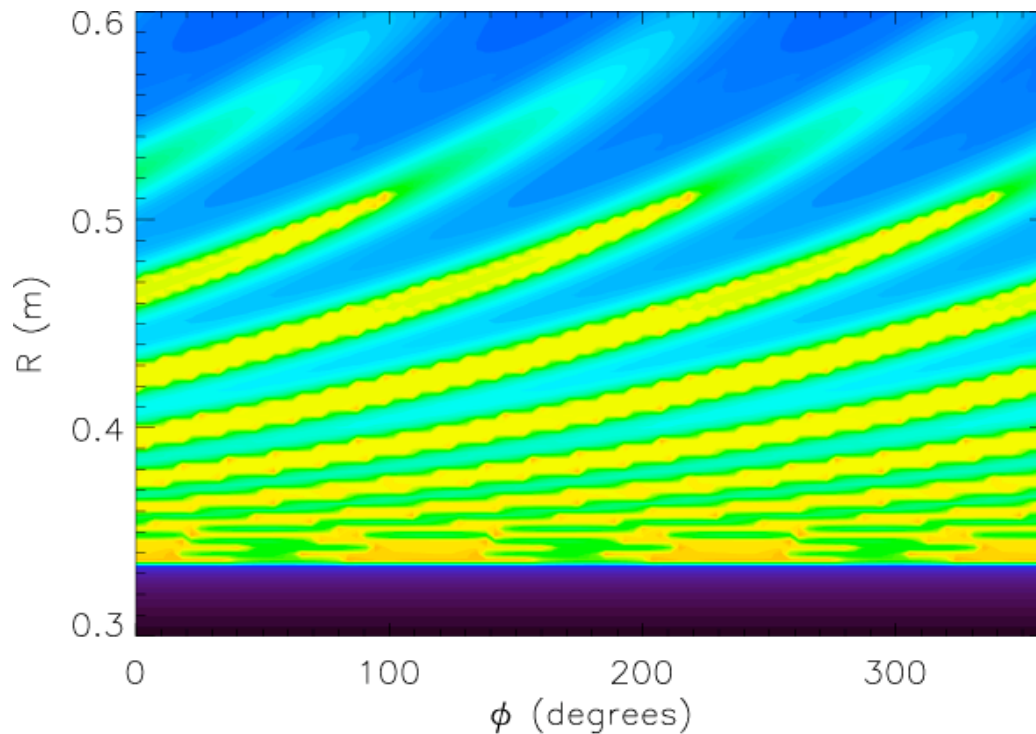
# Strike-Point Splitting Predicted...

- Effects of axisymmetric fields (flux expansion) on divertor heat loading has been characterized in NSTX.<sup>1,2</sup>
- What happens when 3-D fields are applied?

[1] D. Gates et al, Phys. Plasmas **13**, 056112 (2006)

[2] V. Soukhanovskii, et al., "High flux expansion divertor studies in NSTX", 36th EPS Conference on Plasma Physics

*Connection Length for Field Lines  
Launched From Divertor Floor ( $n=3$ )*



**J-W. Ahn, J.M. Canik (ORNL)**



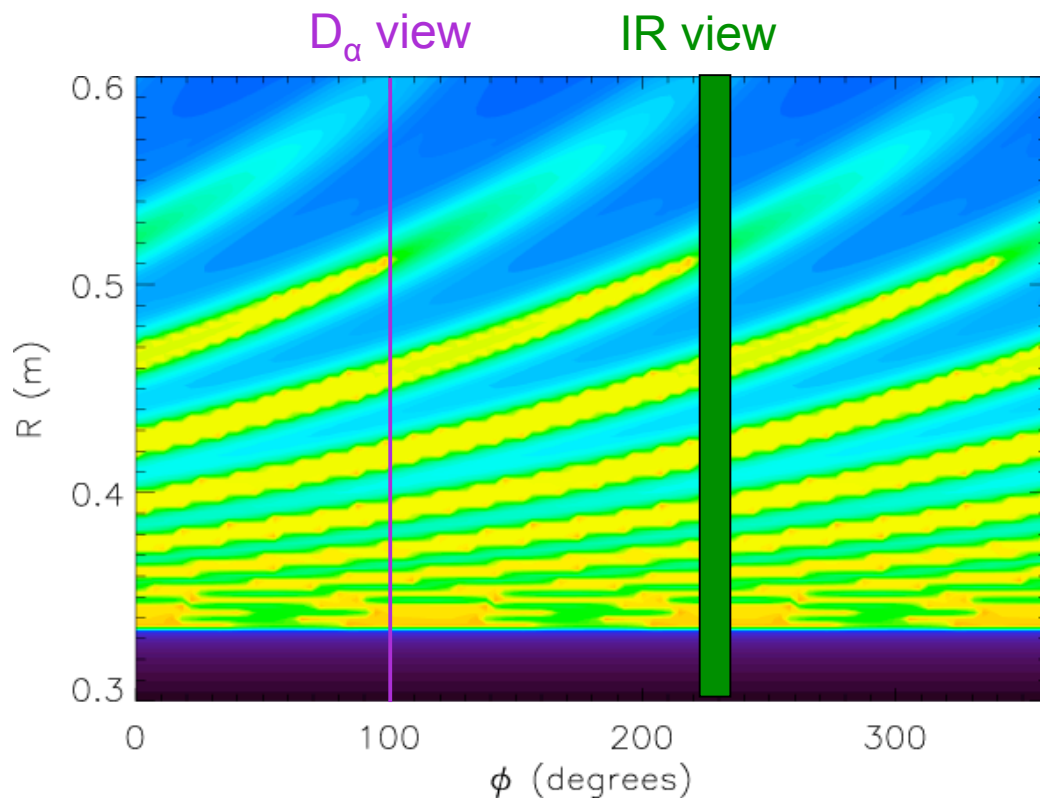
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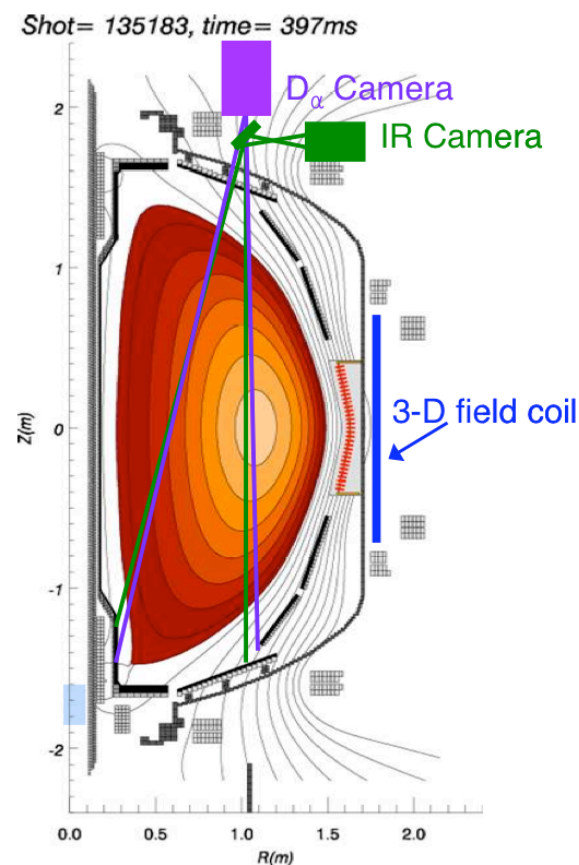
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*Connection Length for Field Lines  
Launched From Divertor Floor*



J-W. Ahn, J.M. Canik (ORNL)

*NSTX Diagnostics For Viewing This Splitting*

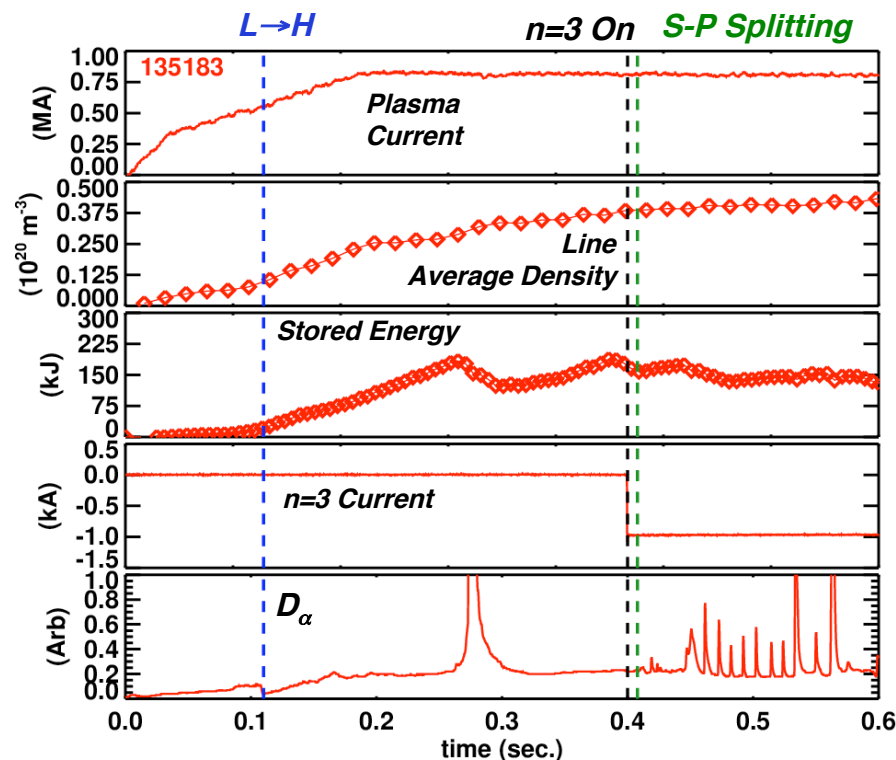




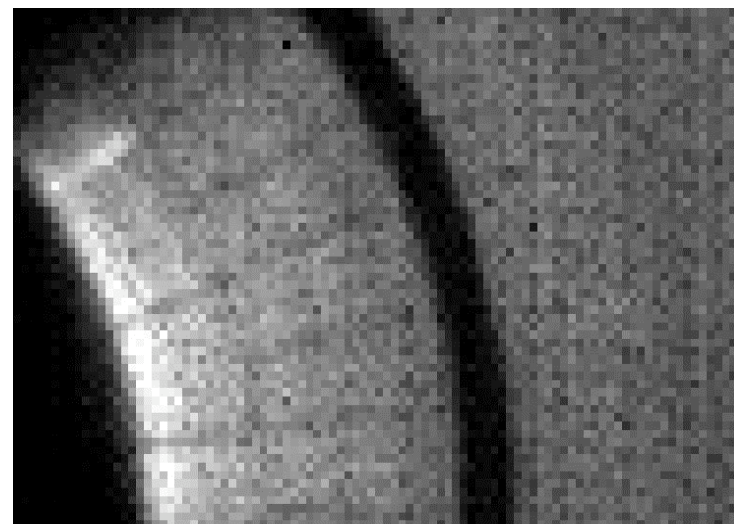
# Strike-Point Splitting Predicted... ...And Observed

- H-mode discharge with 4 MW NBI.
- Steady  $n=3$  fields used to trigger ELMs.
- S-P. splitting clearly observed in period before ELMs.<sup>1</sup>

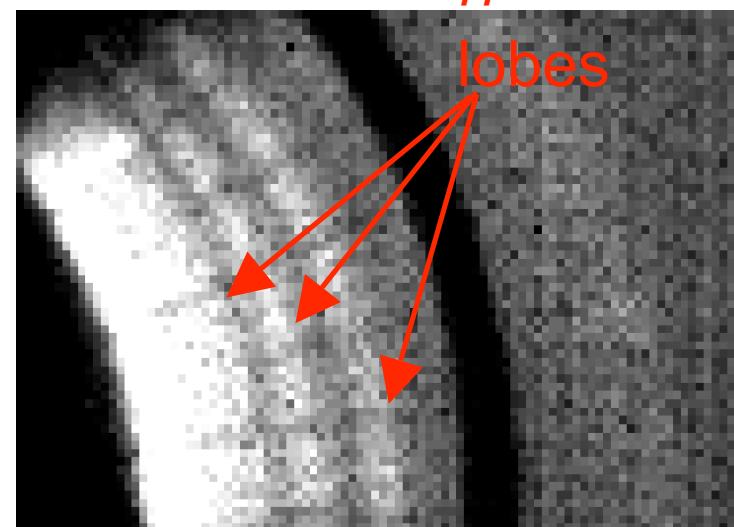
[1] J-W. Ahn et al, submitted to Nuclear Fusion (2009)



*Before 3-D Field Application*

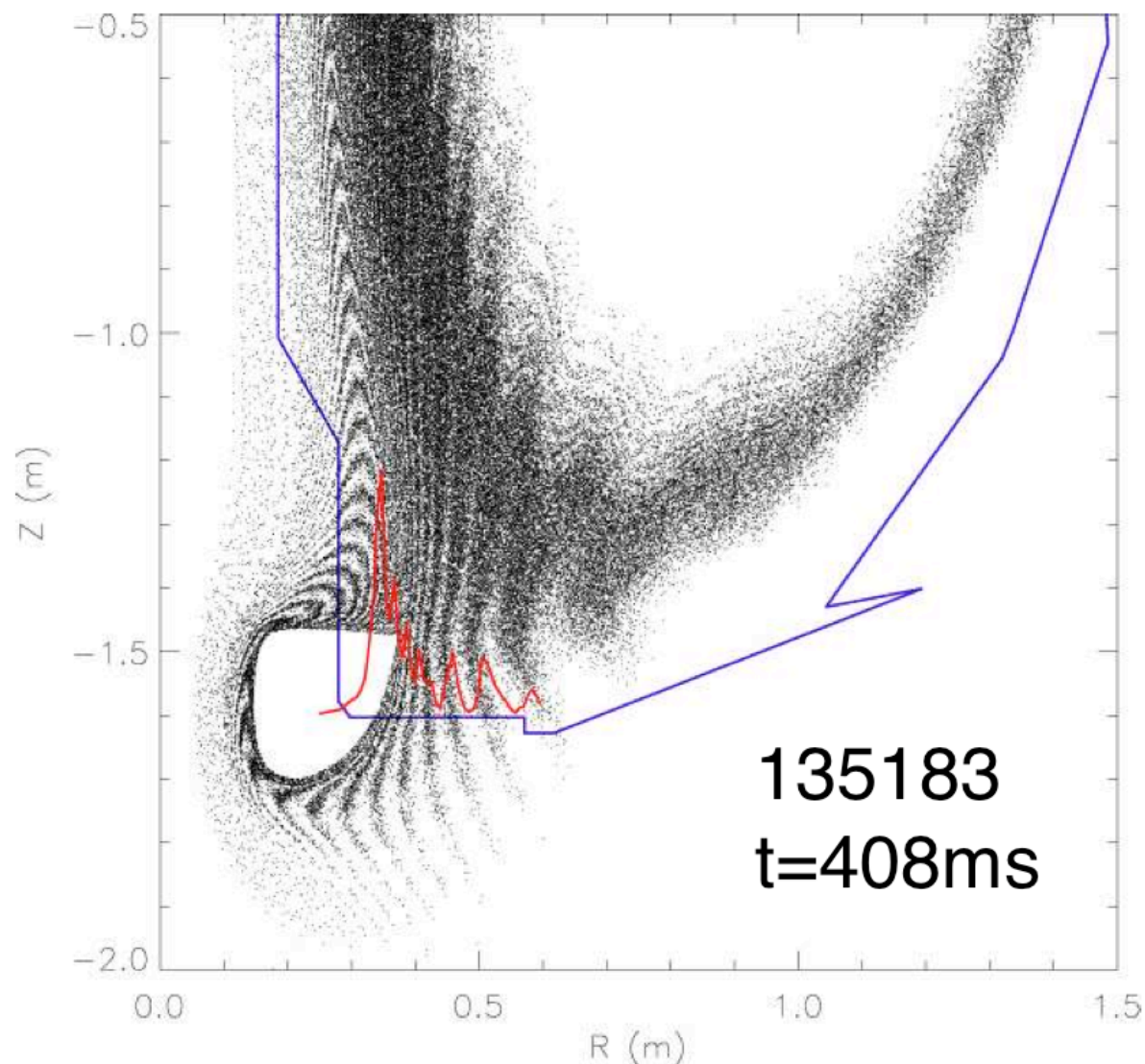


*After 3-D Field Application*



J-W. Ahn, J.M. Canik (ORNL)

# Heat Flux Profile Has Spatial Structure Similar to that From Vacuum Field Line Following

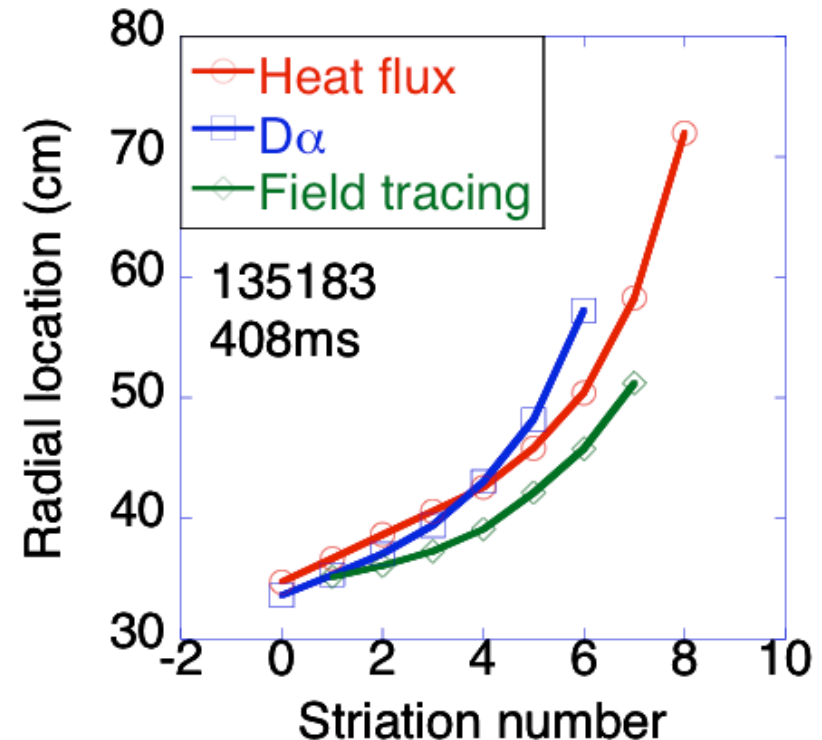
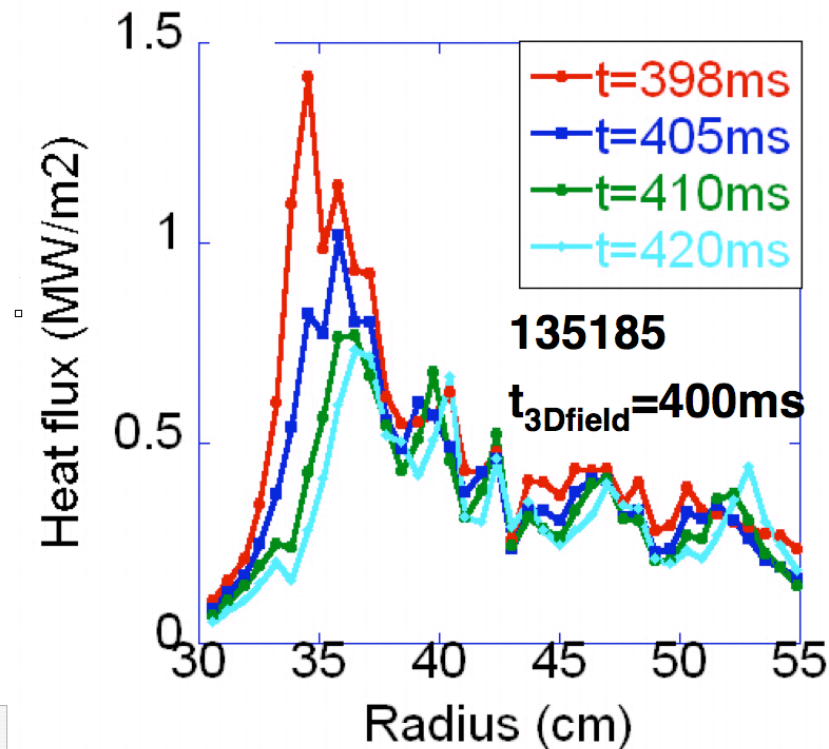


*J-W. Ahn, J.M. Canik (ORNL)*

# Prompt Development of Lobes With 3-D Fields; Lobe Locations Consistent With Vacuum Modeling



- Development of lobe-structure.
  - Rapid response after application of 3-D fields.
    - Time scale appears to be set by 3-D field penetration through vessel.
  - Reduction of peak heat flux?
- Radial locations of  $D_\alpha$  and heat flux lobes qualitatively consistent with vacuum modeling.
  - Working on additional effects.
    - IPEC w/ various boundary truncations.
    - Intrinsic EFs



J-W. Ahn, J.M. Canik (ORNL)

## Summary: 3-D Fields Can be Both Dangerous and Advantageous

- Error fields, coupled to the plasma response, can degrade performance.
  - Damp beneficial rotation.
  - Make plasma more susceptible to RWMs and NTMs.
  - Drive static islands.
  - All this makes the plasma more disruptive.
- 3-D fields can be useful tools for tokamak control needs.
  - Plasma response can be used to detect and correct slowly varying error fields.
  - NTV+non-resonant fields can drive rotation, or possibly tailor the rotation profile.
    - See papers by A. Garofalo, W. Solomon.
  - Beneficial modifications to the ELM characteristics.
    - Either suppression or triggering.
  - Modify divertor heat loading.
  - Deconfine disruption runaways.
    - See talk by Lehnen on Monday.

***NSTX Research Forum is December 1<sup>st</sup>-3<sup>rd</sup>***

***Proposals for error field physics and control studies (and other MHD) are welcome.***

***See: <http://nstx.pppl.gov/index.html>***

# Backup

## Summary

- Error fields can penetrate and disrupt a high- $\beta$  plasma more easily than might be expected from the low- $\beta$  density scaling.
- Plasma response can be used as a powerful tool for controlling error fields.
- Non-resonant error fields are present in NSTX, and can be corrected.
- Combined  $n=1$  control and  $n=3$  correction have dramatically improved NSTX long-pulse high- $\beta$  reliability.
- Regime of enhanced NTV torque at low rotation, and  $1/\nu$ -like  $T_i$  dependence, have been observed.
- Applied 3-D fields can be used to trigger/pace ELMs.
- Modifications of divertor heat loading with 3-D fields has been observed.

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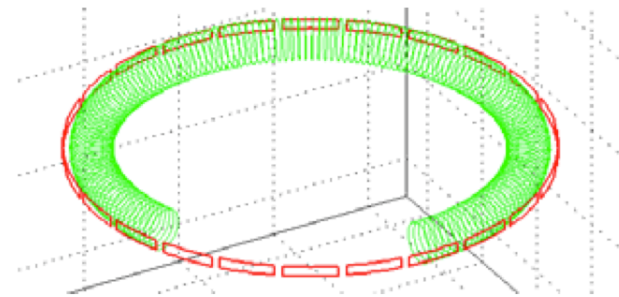
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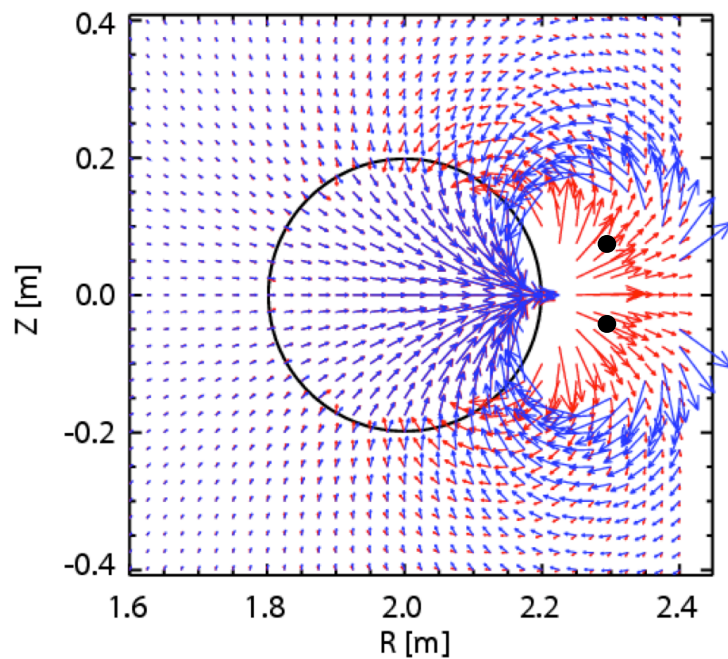
## Example: Force Free Cylinder

$$\delta B = \delta B^X + \delta B^P$$

*Total Perturbed Field*      *External Field*      *Plasma Response*

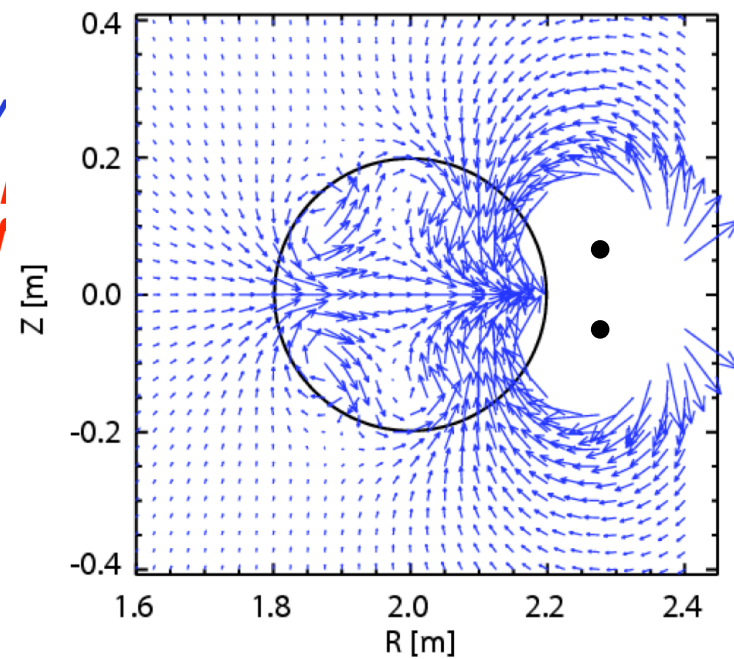


**Vacuum Fields**



*Actual v*  
*Field in ,*  
*to sur*

**Plasma + Vacuum**





## Continued Active Program in Error Field Physics

- RFA Suppression: Comparison of different sensor types for utility.
- Strike-Point splitting:
  - Role of  $q_{95}$ , toroidal mode number
  - Test of rotating perturbation for further heat flux mitigation.

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