

Max-Planck-Institut für Plasmaphysik

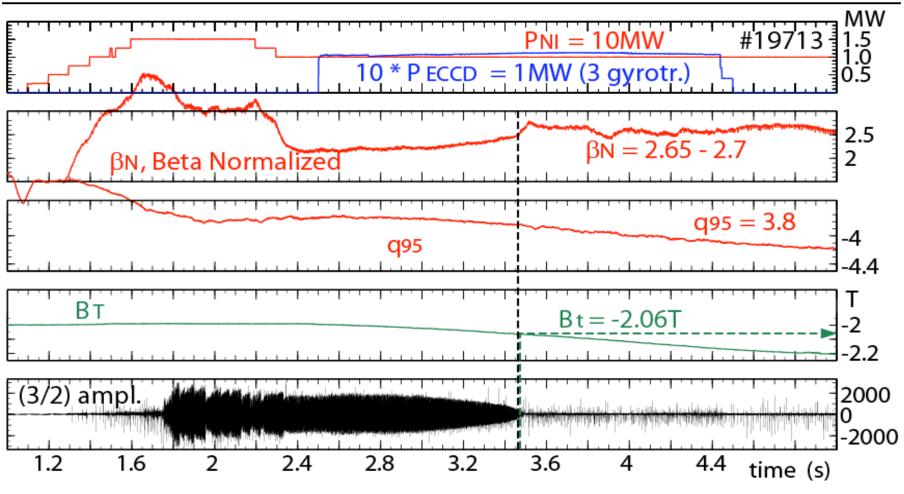
Modulated ECCD control of NTMs in ASDEX Upgrade

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- introduction
- role of deposition width
- modulated broad ECCD
- power requirements
- persepctive and requirements for ITER
- summary

(3/2)-NTM stabilisation with narrow ECCD at low q_{95}

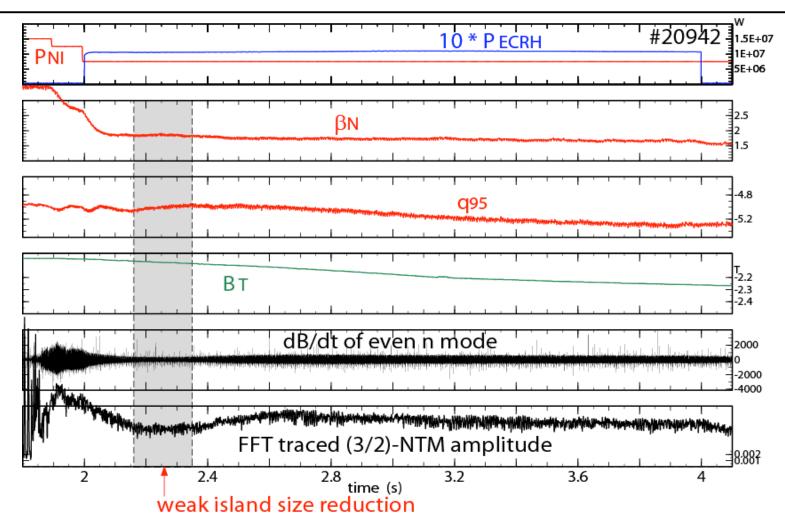
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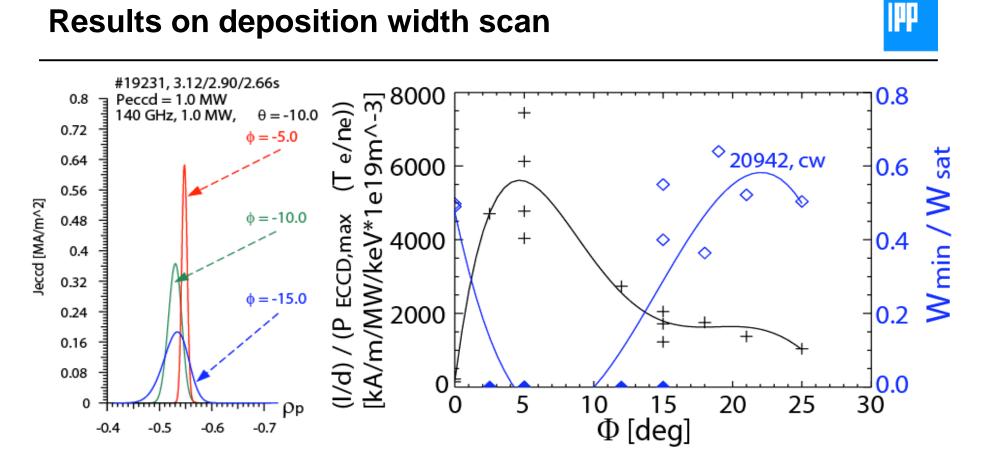
- complete stabilisation of smooth NTM at β_N =2.7 near ITER q_{95} =3.8 with only 1MW P_{ECCD} , $\beta_N/(P_{ECCD}/P_{NBI}) = 27$
- β_N increase with more PNBI possible (reexcitation, Shafranov shift)

No stabilisation possible for broad ECCD deposition

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- even with reduced NBI power :
- full ECCD (1.1MW) can no longer stabilise the (3/2)-NTM



NTM stabilisation predicted to be most efficient at high I_{ECCD}/d

- mode stabilised by helical current within island
 - \rightarrow *d* should be smaller than *W*
- possible to stabilise NTMs with half the total current, if better localised and narrow

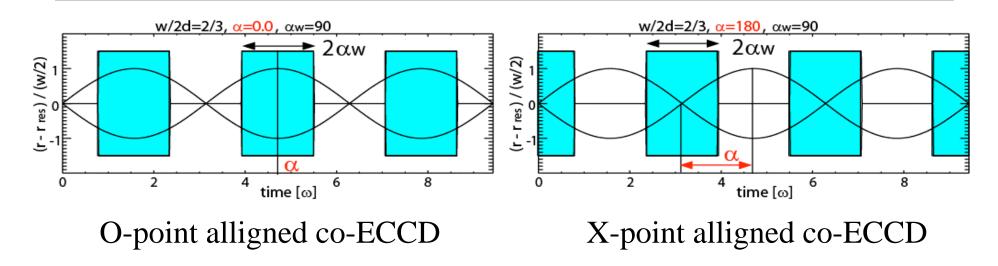


$$\frac{\tau_{res}}{r_s}\frac{dW}{dt} = a_{bs}r_s\beta_p\frac{1}{W} - r_s\Delta'_{stab} - c_j\frac{L_qr_s}{d^2}\frac{I_{ECCD}}{I_p(r_s)}\left(a_{mn}\eta_{mn}\left(\frac{W}{d}\right)\frac{d^2}{W^2} + a_{00}\right)$$

- helical (m,n) current in the island, a_{mn} works on nonlinear stability (suppression of existing mode)
- modification of equilibrium (0,0) current profile, a₀₀ also linear stability (prevention of mode)
- $\eta_{mn}=~j_{ECCD}~/~j_{bs}$, efficiency with which a helical component is created by island flux surface averaging
- c_i accounts for derivation from cylindrical large aspect ratio calculations
- misalignment of ECCD deposition not included

Phase locked modulated ECCD

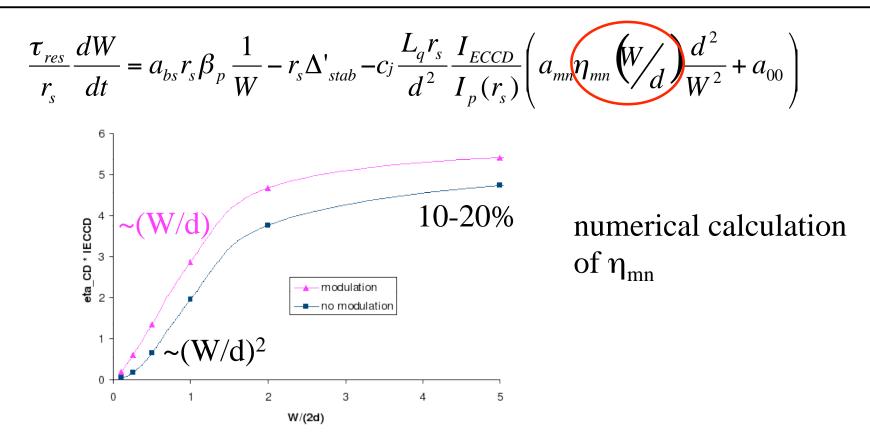




- O-point alligned modulated ECCD : the current is driven helically within the island , high η_{mn}
- X-point alligned modulated ECCD should give destabilising effect (wrong phase !) + more sensitive Δ ' effect

Efficiency for driving a helical current





η _{mn}	W > 2d	W < 2d
unmod.	const	$\sim (W/d)^2$
mod.	const, 10-20%	~ W/d

Predictions for the limits



$$\frac{\tau_{res}}{r_s}\frac{dW}{dt} = a_{bs}r_s\beta_p\frac{1}{W} - r_s\Delta'_{stab} - c_j\frac{L_qr_s}{d^2}\frac{I_{ECCD}}{I_p(r_s)}\left(a_{mn}\eta_{mn}\left(\frac{W}{d}\right)\frac{d^2}{W^2} + a_{00}\right)$$

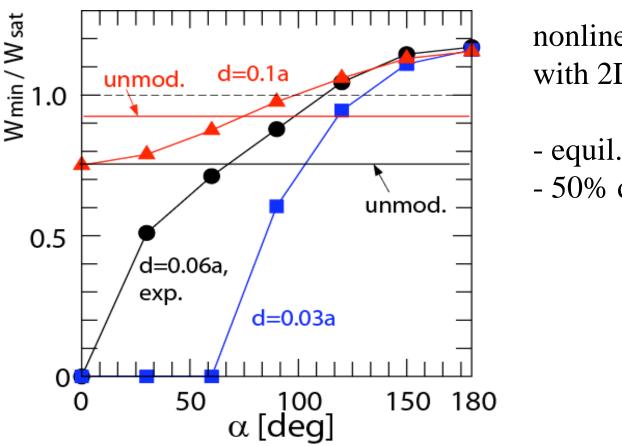
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unmod.	const	$\sim (W/d)^2$
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a _{mn} -term	W > 2d	W < 2d
unmod.	$ \sim I / W^2$	$\sim I/d^2$
mod.	$\sim I / W^2$	$\sim I/(dW)$

 a_{00} – term always ~ I / d²

- $W > d : \eta_{mn} \sim const.$ and I_{ECCD} counts; modulation has little advantage
- $W < d : \eta_{mn} \sim (W/d)^2$ without modulation and efficiency is small
- $W < d : \eta_{mn} \sim W/d$ with modulation and efficiency is better than with cw-ECCD
- \Rightarrow deposition should be well localized and modulated for W < d

Prediction of stabilisation for modulated ECCD



nonlinear MHD code with 2D transport:

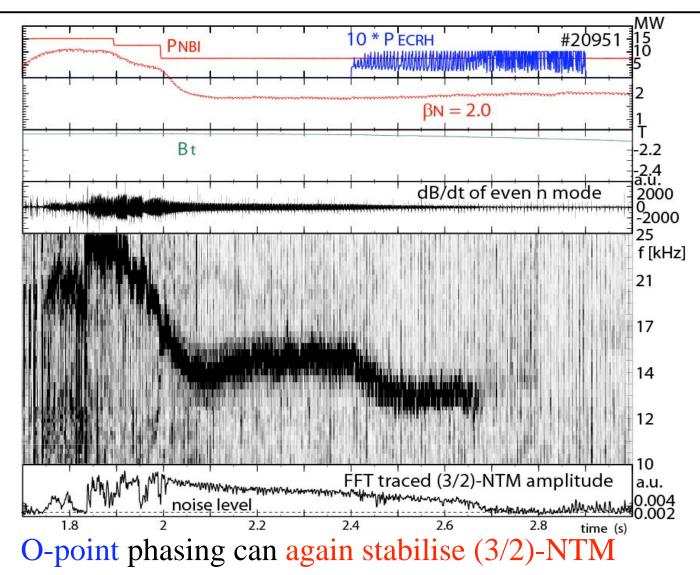
- equil. current profile taken
- 50% duty cycle

(Q.Yu, IAEA 2006 M.Maraschek, subm. to PRL)

- very broad deposition \rightarrow no stabilisation possible at all
- intermediate width → stabilisation can be regained for O-point modulation
- X-point modulation is predicted to be destabilising



NTM - stabilisation with modulated broad ECCD

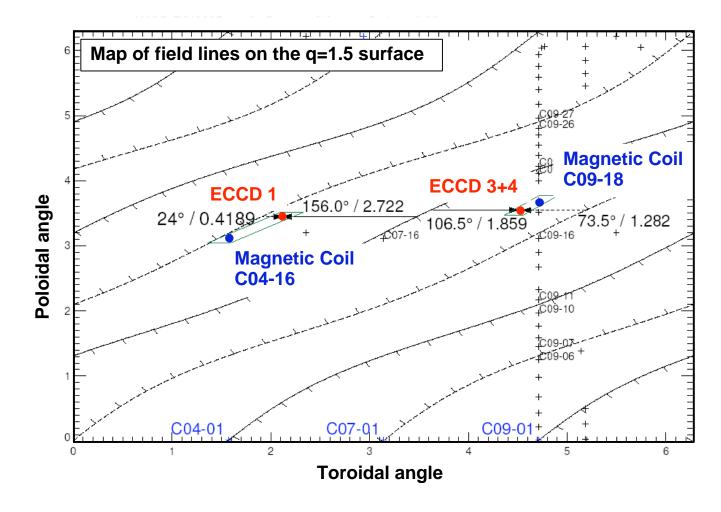


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• possible for smooth (3/2)-NTM with installed P_{ECCD}

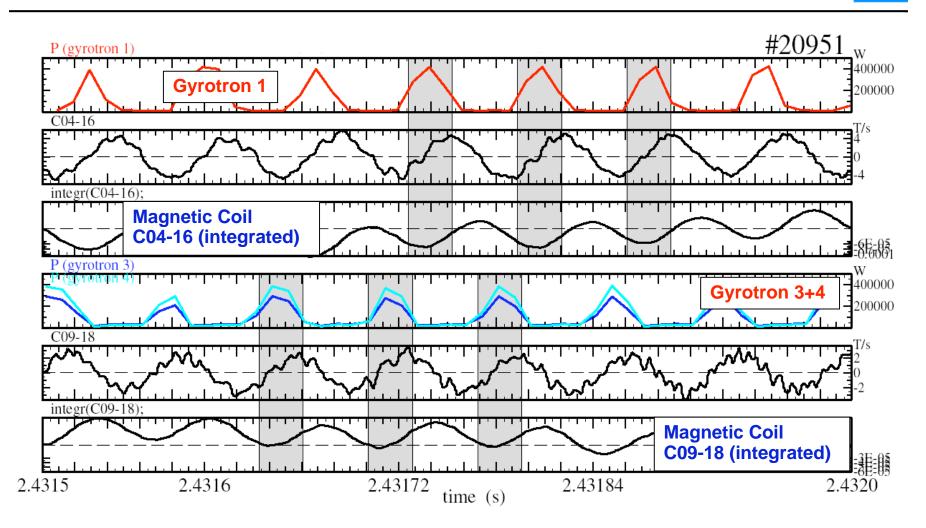
Mapping of the magnetic field lines required





- need to synchronise three gyrotrons at different positions with island
- requires mapping along field lines (magnetic coil as sensor for island)

Control of the phase with nearby control coils

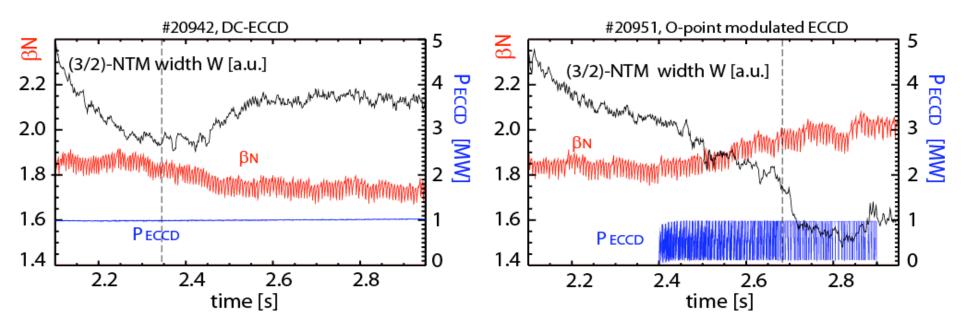


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Comparison between cw-ECCD and modulated ECCD

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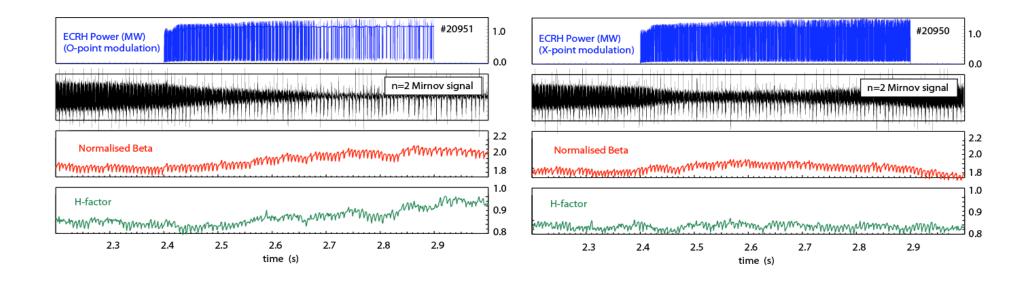


ASDEX Upgrade: NTMs rotate past ECCD antennae (plasma rotation)

- need to modulate gyrotron with island frequency of 30kHz
- 30 kHz modulation problematic, but ITER has lower rotation (2-3kHz)
 ⇒ however, narrow deposition highly desirable
- regain of stabilisation with O-point modulation compared to cw-ECCD

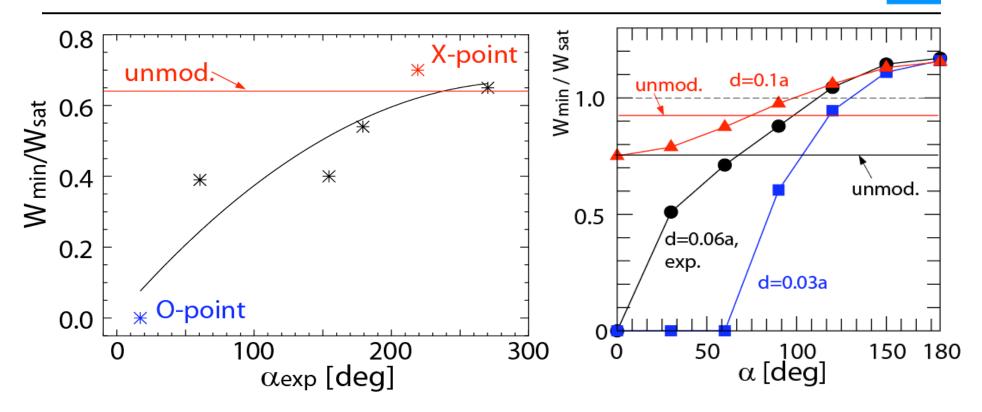
Comparison between O and X-point modulated ECCD

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- weak, but still stabilising effect of ECCD in X-point
- clearly increased β_N and H-factor for O-point case compared to X-point case

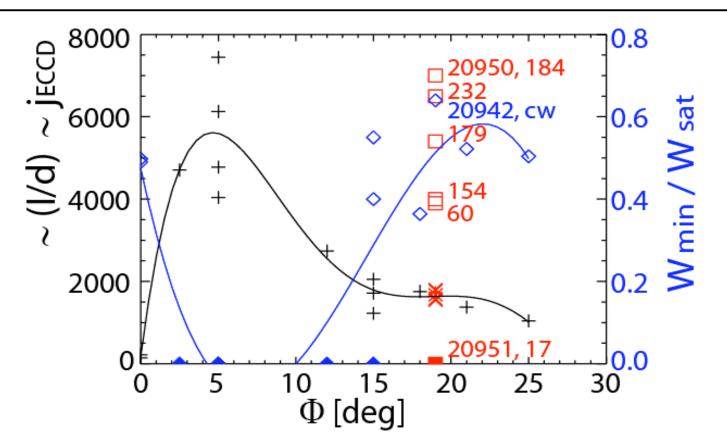
Experimental ECCD phase dependence of island size



- dependence of island size reduction could be perfectly reproduced
- X-point phasing worse than non-modulated, remaing Δ ' effect,
- current profile not adjusted to experiment

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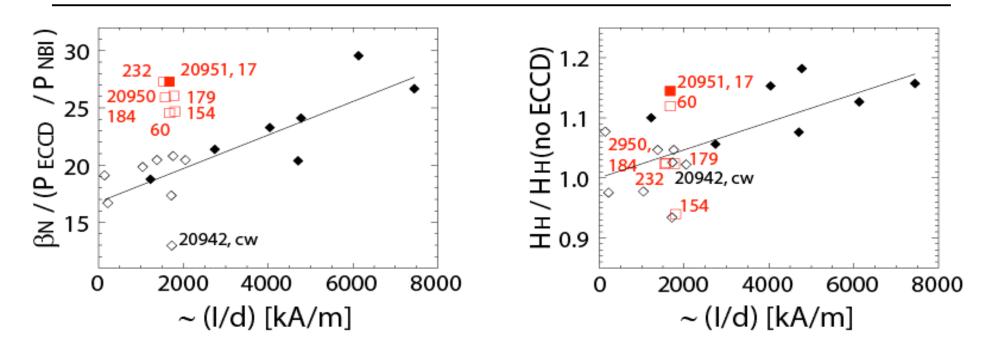
Stabilisation capability with modulated ECCD



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- Variation of stabilisation as function of phase angle and resulting current peaking $I_{ECCD}/d \sim j_{ECCD}$
- O-point: regain of stabilisation
- X-point: worse than DC ECCD





- increase in β_N and H_H / H_H (no ECCD) with increased current peaking $I_{ECCD}/d \sim j_{ECCD}$ for unmodulated cases
- additional improvement for O-point modulated cases for both figures of merit for given current peaking

Summary and conclusions

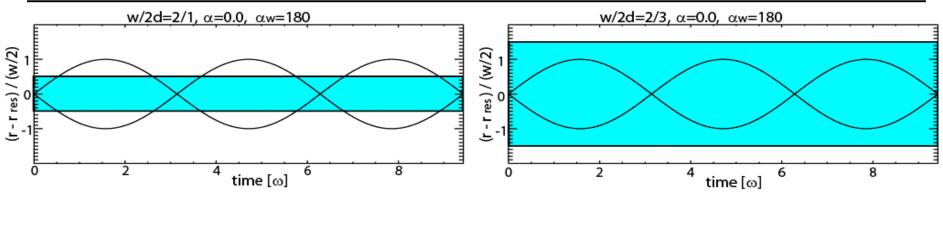


- extended depositon width scan verifies previous results
- O-point modulation of ECCD regains stabilisation
- X-point modulation worse than cw-ECCD
- phase scan consistent with theory
- improvement in achievable pressure and confinement
- modulation highly desirable for ITER ECRH in addition to optimisation of the deposition width

END



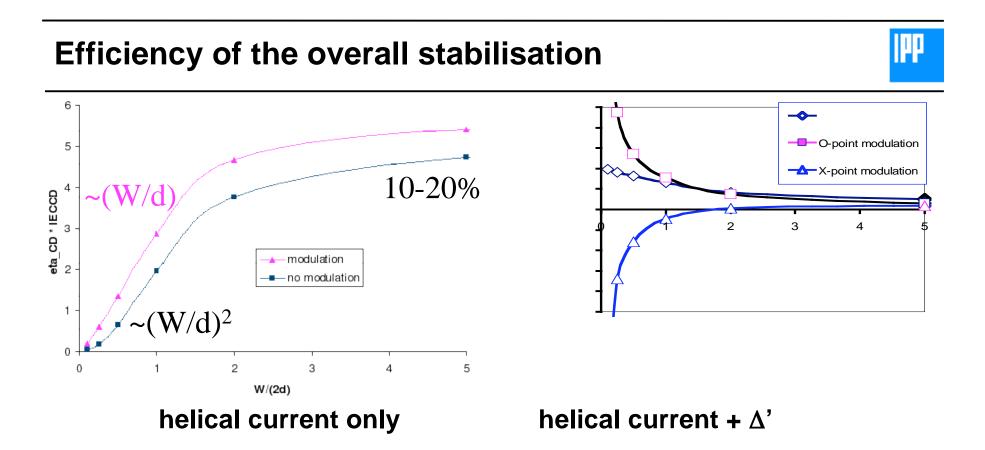
Status and perspective of constant ECCD



present experiments: 2d < W_{marg}

ITER, large exp. due to Lamor radius: $2d > W_{marg}$

- in ITER / any larger experiment $2d > W_{marg}$ is likely:
 - launcher geometry (technics),
 - device independent marginal island size ~ ρ_{pi} (physics)
- driving helical current within the island is relevant
 ⇒ O-point modulation of co-ECCD

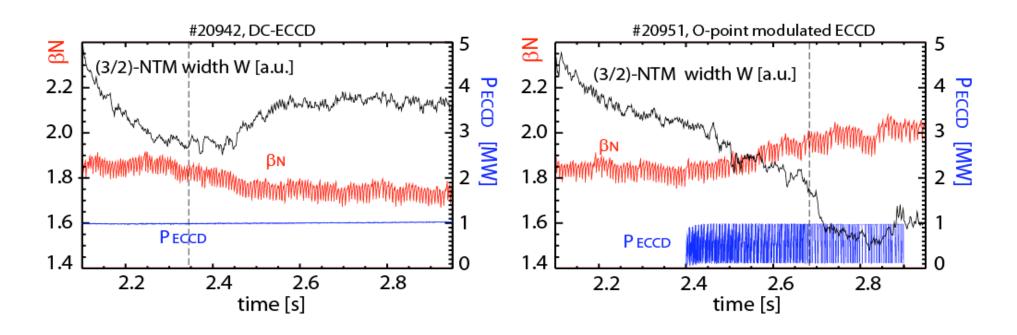


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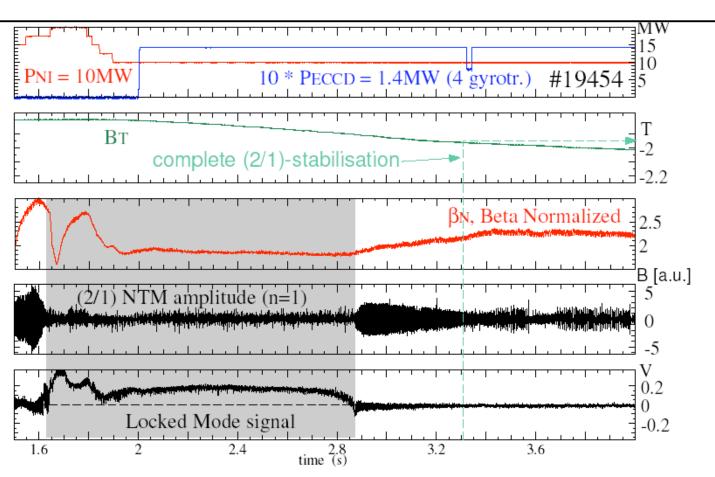
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 ⇒ narrow deposition highly desirable

Recent progress in validating physics requirements



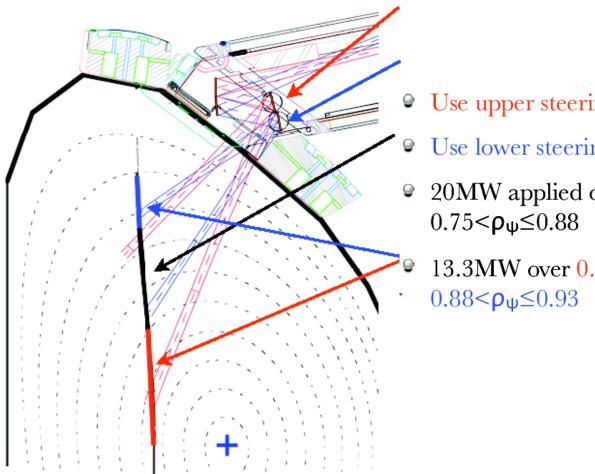
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Narrow deposition allows (2,1) stabilisation at higher β_N than before

- full stabilisation at β_N = 2.3 with 1.4 MW (β_N = 1.9/1.9 MW for broad dep.)
- but: for (2,1) stabilisation, still power limited (should do this at $\beta_N = 3!$)

Present Lines of Optimisation: FS Upper Launcher





- Use upper steering row to access inner surfaces
- Use lower steering row to access outer surfaces
- 20MW applied over principle NTM region 0.75<ρ_ψ≤0.88
- 13.3MW over $0.38 < \rho_{\psi} \le 0.75$ and $0.88 < \rho_{\psi} \le 0.93$

Beam penetration not prependicular to flux surfaces:

- \bullet j_{ECCD} is more than sufficient, but still relatively broad deposition
- major redesign of ITER ECRH required to gain smaller Z_{launcher} !

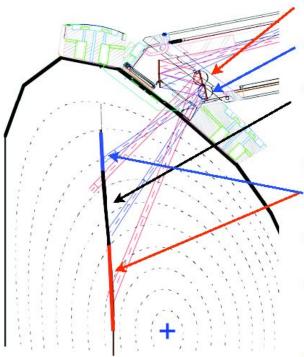
Present Lines of Optimisation: FS Upper Launcher



- 4 Ports or 32 entries for 24 gyrotrons
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- Decreases opening in first wall
- Decreases overall rotation (fatigue) of steering mechanism
- ♀ Maintains $η_{NTM}$ =1.2 (with ≤13MW)

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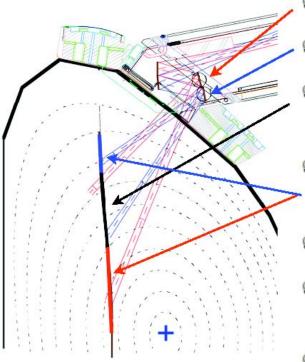
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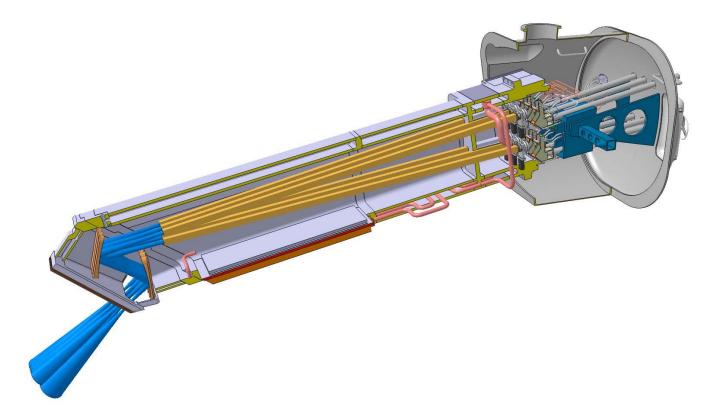
Possibilities to enhance FS Upper Launcher performance:

- \bullet since j_{ECCD} is more than sufficient, steering range can be expanded
- partitioning of power in the different rows can enhance flexibility



The present system design: Upper Launcher





Alternative design based on remote steering

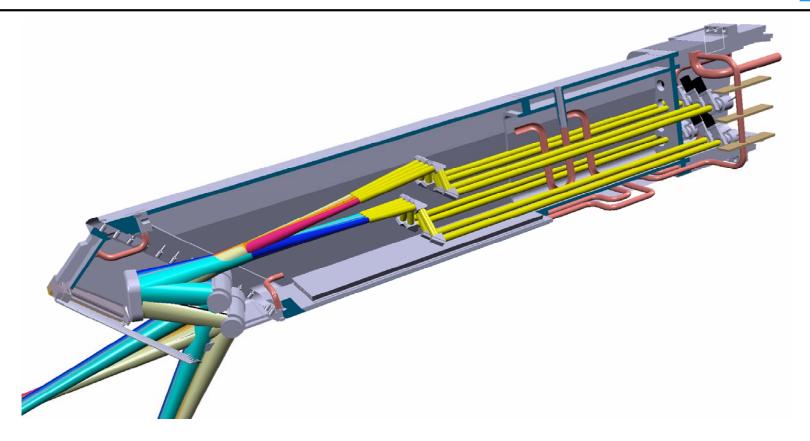
• no moving parts close to plasma

but: spot size in plasma much bigger than for front steering

 \Rightarrow physics perfromance reduced w.r.t. that of front steering solution

The present system design: Upper Launcher





Reference design(s) based on front steering

 upper launcher: poloidal (remote) steering range ±8-10° at front mirror launched from 3 ports in 2 rows of 4 beams per row biggest challenge: engineering of moving parts at front end