

# SEPARATRIX LOCATION AND ERROR FIELD

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# MOTIVATION / BACKGROUND

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- **Enhanced confinement of H-mode discharges often leads to large edge density and temperature pedestals with very narrow widths ~ 1-4 cm**
- **Accurate determination of the edge separatrix location becomes crucial for proper interpretation and understanding of the physical processes governing H-mode discharges**
- **In DIII-D, a 1-4 cm difference is found between the separatrix vertical position determined from EFIT magnetic analysis and those inferred from Thomson  $T_e$  measurements**
  - **First seen in some quiescent H-mode (QH-mode) upper single-null discharges but since found also exist for other discharges**

# QUESTIONS TO ANSWER

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- Are the difference between the edge separatrix vertical positions as determined from EFIT and Thomson  $T_e$  due to real physical effects ?
- What physical processes may lead to this difference ?
- What are the effects of externally imposed toroidally asymmetric magnetic field or error magnetic field on the edge separatrix boundary ? The new I-coils ?
- How important are the plasma responses ?



12-coil internal set available for experiments 2003

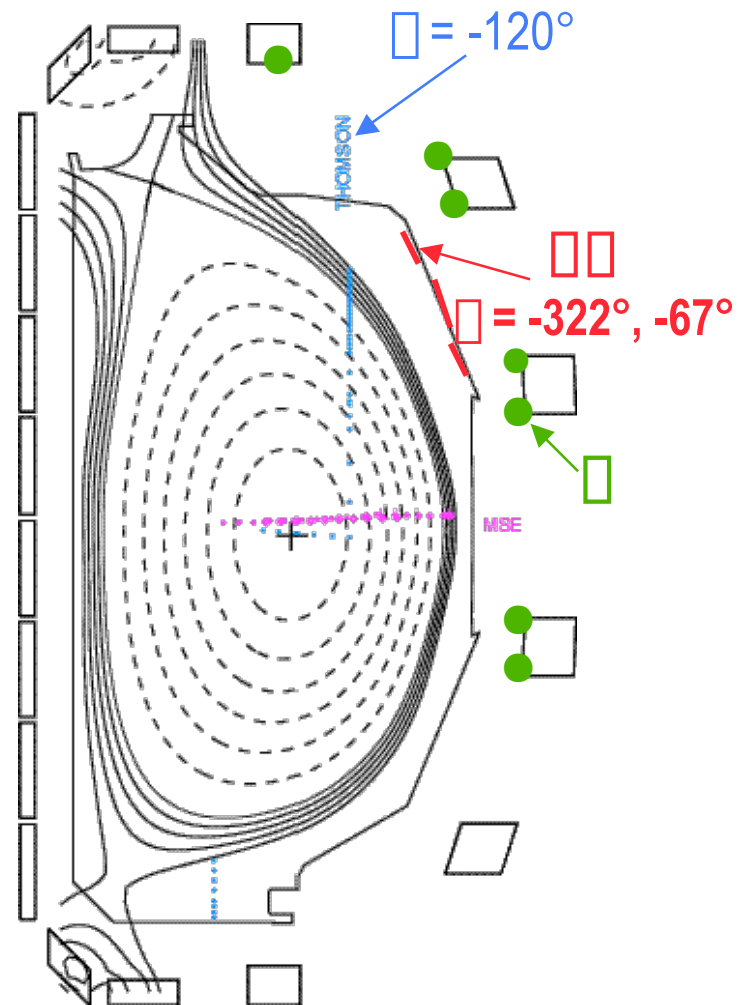
# OUTLINE / SUMMARY

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- Separatrix location can be conveniently obtained from EFIT magnetic analysis and inferred from Thomson  $T_e$  measurements in H-mode discharges
- Results from the two methods generally agree for lower single-null discharges
- There can be a 1-4 cm difference in the Thomson separatrix vertical position in upper single-null and double-null discharges when  $B_p$  is weaker near the Thomson viewing path
- Results of an heat flux analysis for a upper single-null discharge are consistent with a 1-2 cm difference seen between EFIT and Thomson
- A leading explanation for this difference is the toroidal asymmetry due to external magnetic field
- Perturbative 3-D calculations indicate a 1-2 cm radial shift in the external coil location can lead to formation of an edge ergodic layer with observable distortion of the plasma shape
  - Need to include plasma response

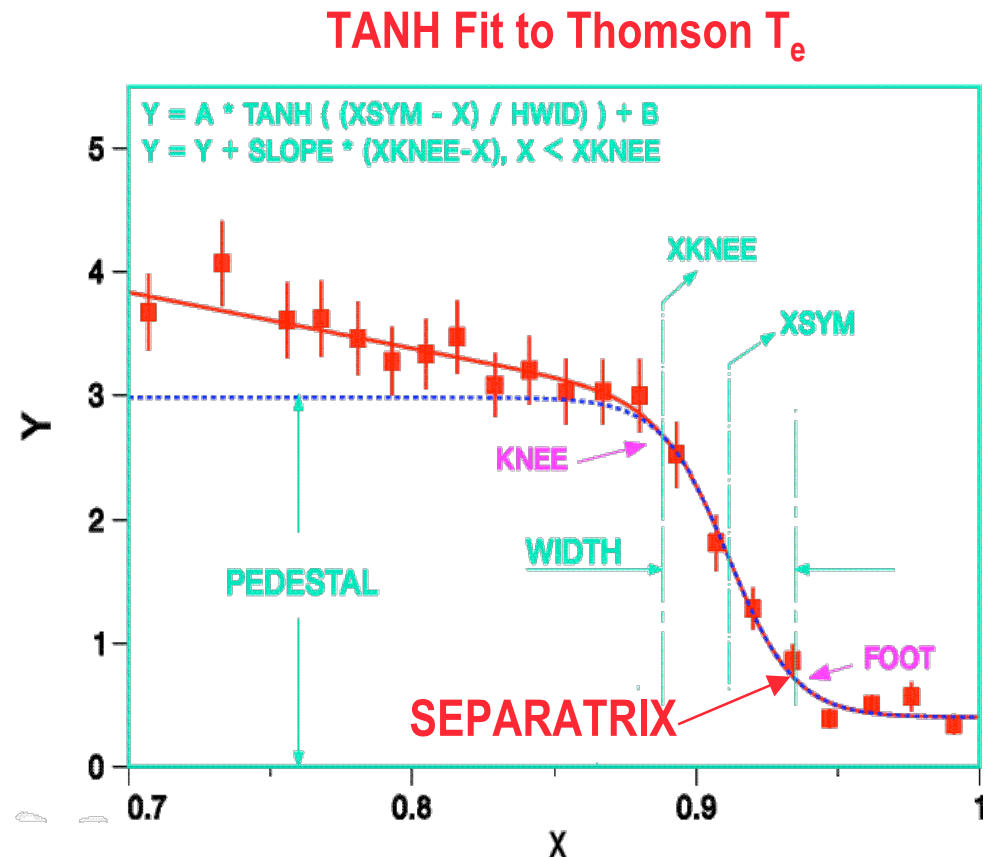
# SEPARATRIX LOCATION IS DETERMINED BY EXTRAPOLATING EXTERNAL MAGNETIC MEASUREMENTS INWARD

- EFIT extrapolates magnetic measurements inward assumed discharge in a 2-D equilibrium state
  - ~ 41 flux loops:  $\psi$ , ~ 73 magnetic probes:  $\theta$
  - Equilibrium relates 2nd derivatives to  $\psi$  and  $\theta$
  - Separatrix location defined by largest closed flux surface enclosed by limiter
  - More accurately determined if separatrix is closer to magnetic loops
- Main magnetic probes are at  $\theta = -322^\circ$ , some at  $-67^\circ$ 
  - Separatrix location largely represents magnetic topology at  $\theta = -322^\circ$
- Thomson measurements are at  $\theta = -120^\circ$



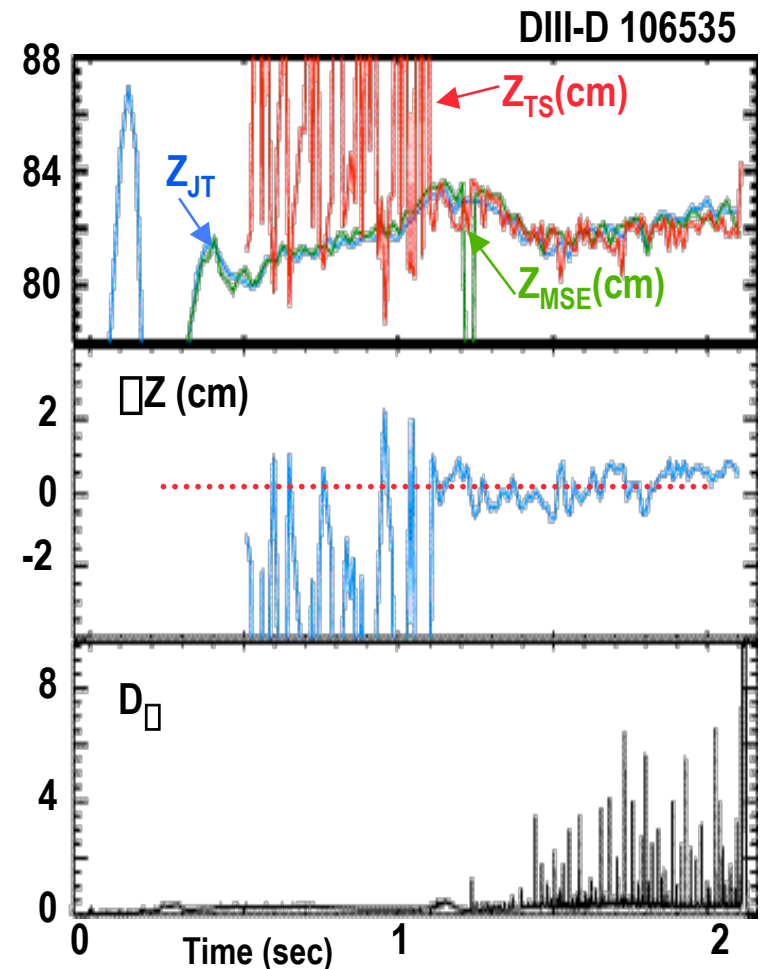
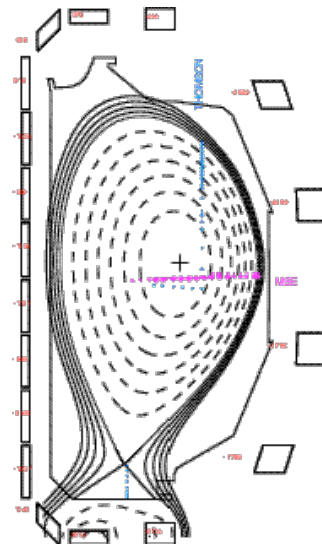
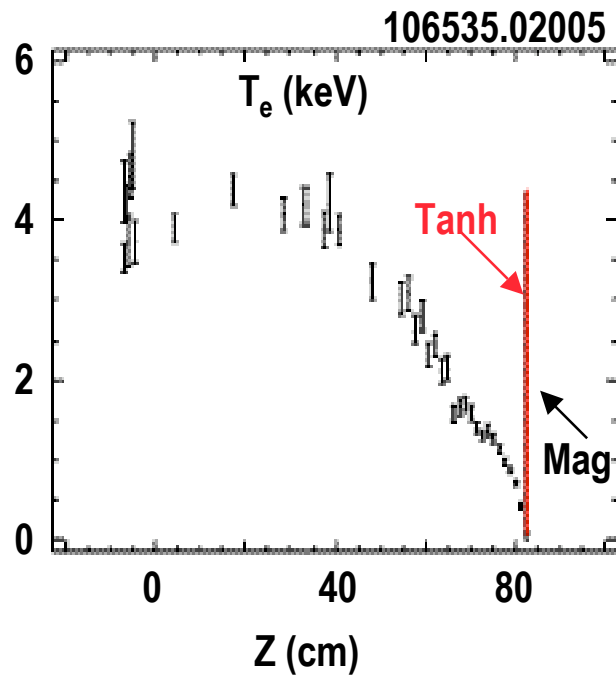
# SEPARATRIX LOCATION CAN ALSO BE INFERRED FROM HYPERBOLIC TANGENT FIT TO THOMSON $T_e$ PROFILE

- H-mode discharges only
- 3 parameters amplitude, radius, and width Tanh fit to  $T_e$
- $Z_{TS} = Z_{SYN} + 0.5 \square Z_{WIDTH}$
- Previous analyses indicate some consistency with UEDGE divertor heat flux solution with this approach [1]
- Thomson measurements are located at the poloidal plane  $\square = -120^\circ$



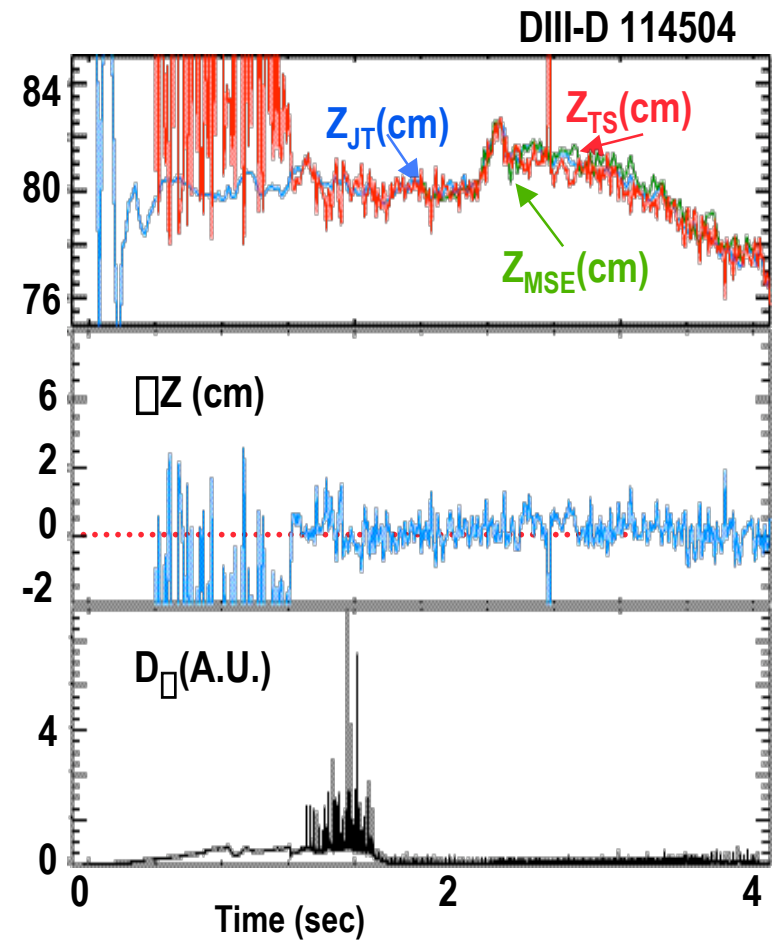
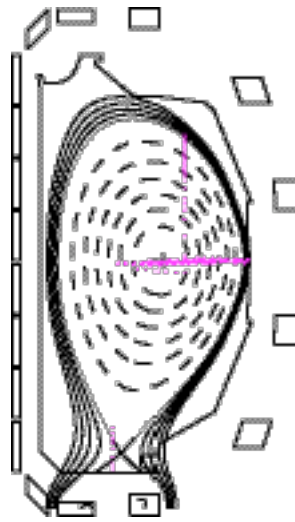
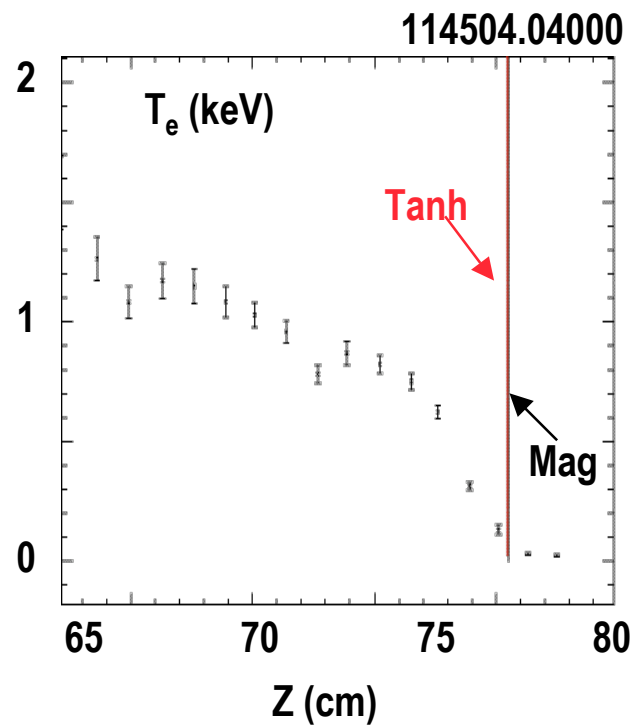
# SEPARATRIX LOCATIONS FROM EFIT AND THOMSON AGREE IN LOWER SINGLE-NULL DISCHARGES

- From 2001 resistive wall mode stabilization experiment



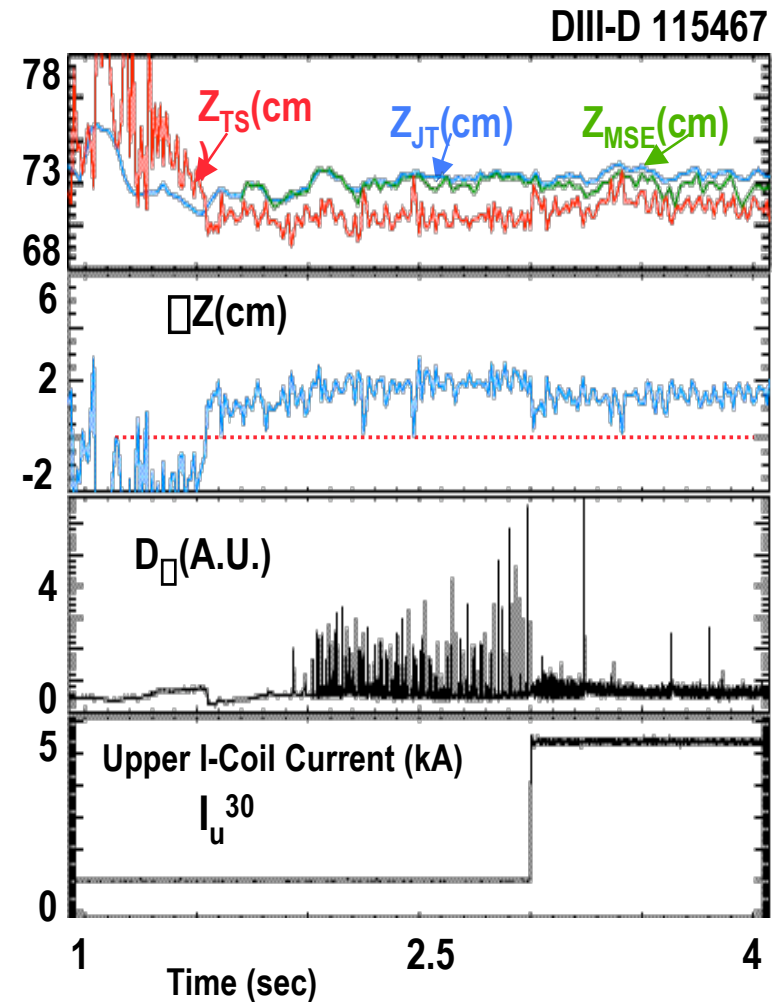
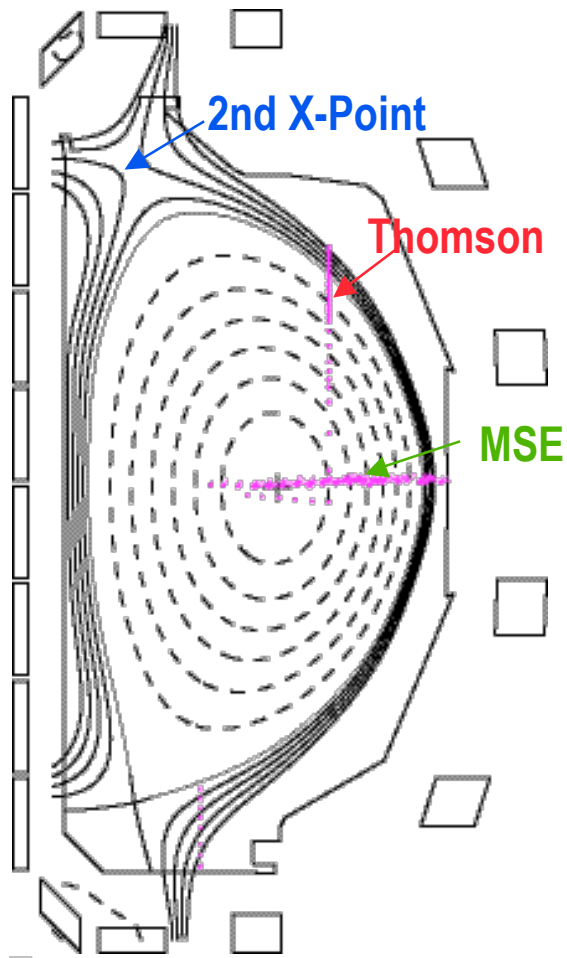
# SEPARATRIX LOCATIONS FROM EFIT AND THOMSON AGREE IN LOWER SINGLE-NULL DISCHARGES

- From 2003 NTM active tracking ECH stabilization experiment



# SEPARATRIX LOCATIONS FROM EFIT AND THOMSON CAN DIFFER WHEN EXTERNAL X-POINT IS CLOSER

- From 2003 ergodic edge ELM experiment **Evans**
- Similar  $\square Z$ 's with or without I-Coil

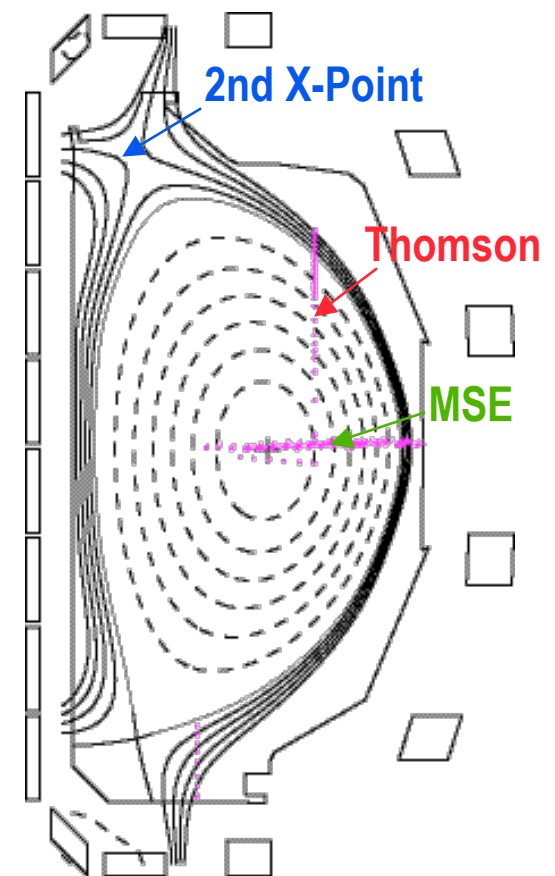


# HIGHER $\chi^2_{MAG}$ WHEN EQUILIBRIUM RECONSTRUCTED WITH THOMSON SEPARATRIX CONSTRAINT

- From 2003 ergodic edge ELM experiment **Evans**
- Reconstructions using for vertical position switch option improve convergence

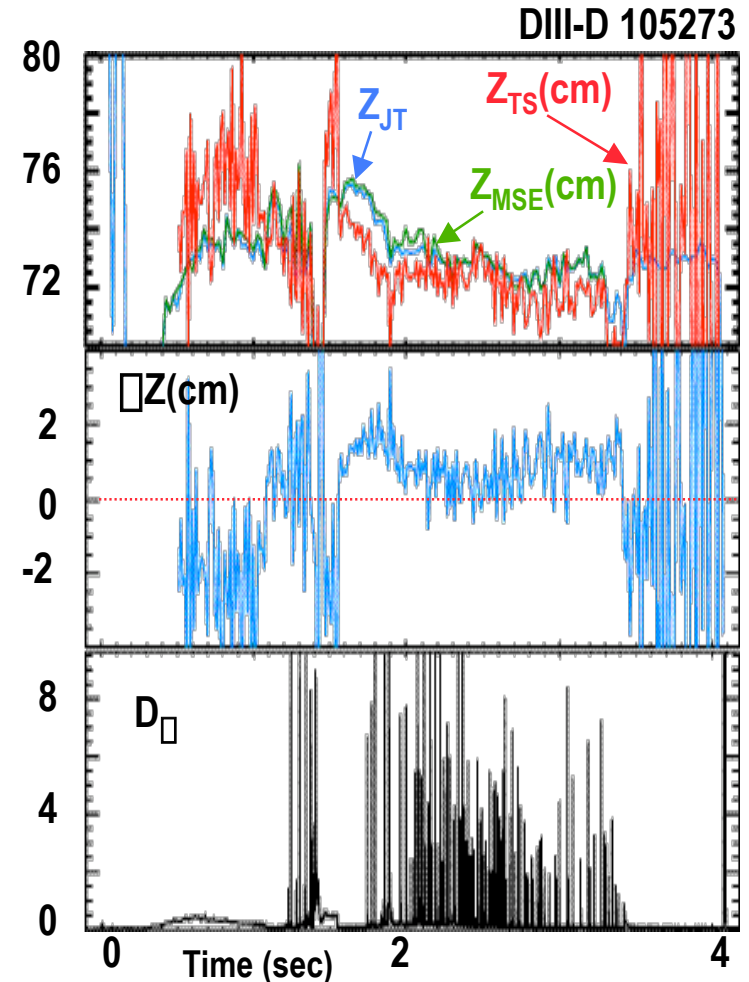
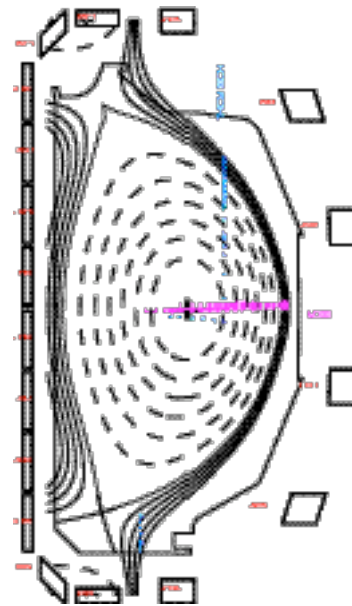
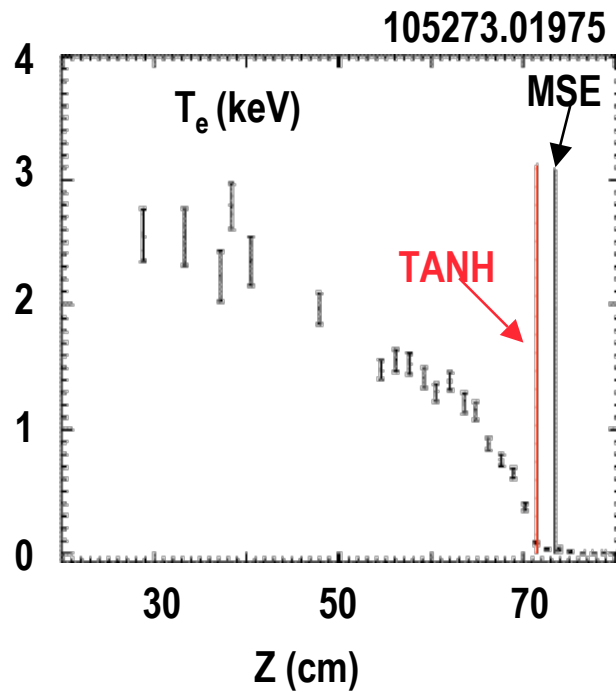
DIII-D 115467

	Iterations	Convergence Error	$\chi^2_{MAG}$	$Z_{JT}(cm)$
No I-Coil 2900 ms	42	$8.1 \times 10^{-5}$	9.2	72.5
No I-Coil Fit $\chi_Z$	10	$7.6 \times 10^{-5}$	9.1	72.5
No-Icoil Fit $Z_{TS}$	36	$9.4 \times 10^{-5}$	14.6	70.6
<b>I-Coil 3400 ms</b>	<b>28</b>	<b><math>9.3 \times 10^{-5}</math></b>	<b>9.5</b>	<b>72.9</b>
<b>No I-Coil Fit <math>\chi_Z</math></b>	<b>11</b>	<b><math>6.4 \times 10^{-5}</math></b>	<b>8.6</b>	<b>72.9</b>
<b>No-Icoil Fit <math>Z_{TS}</math></b>	<b>29</b>	<b><math>7.4 \times 10^{-5}</math></b>	<b>10.3</b>	<b>71.9</b>



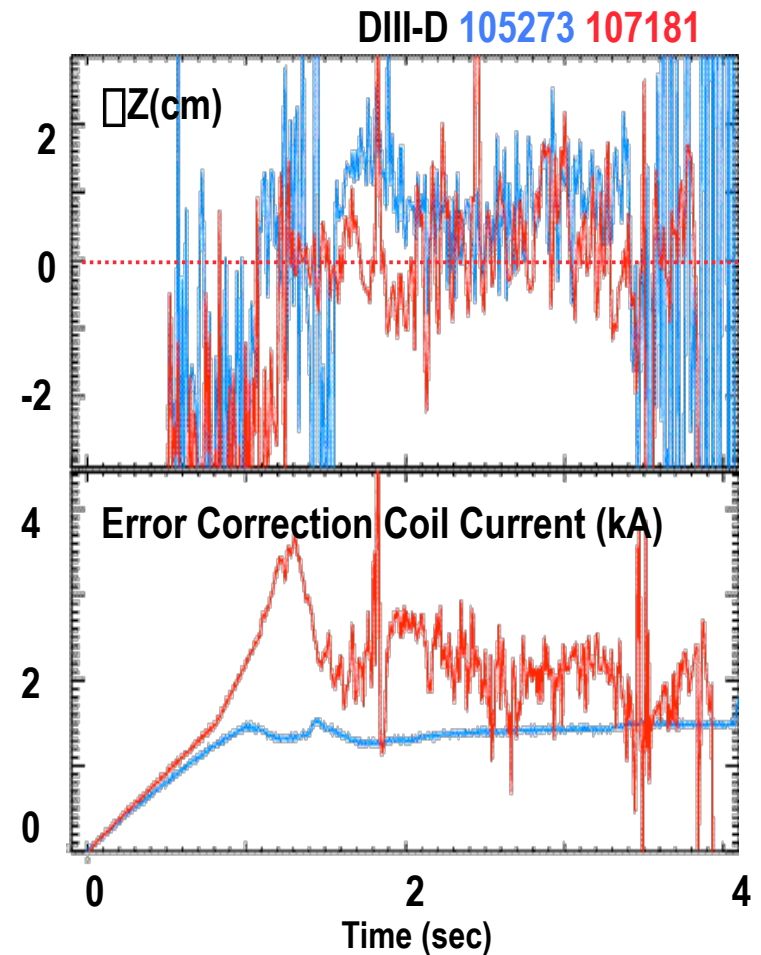
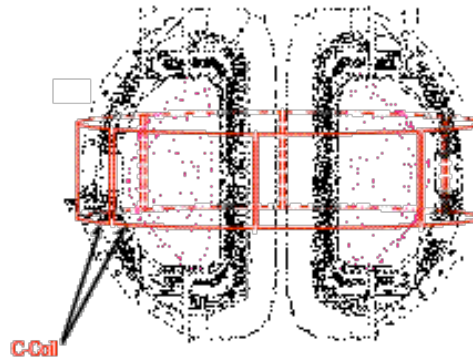
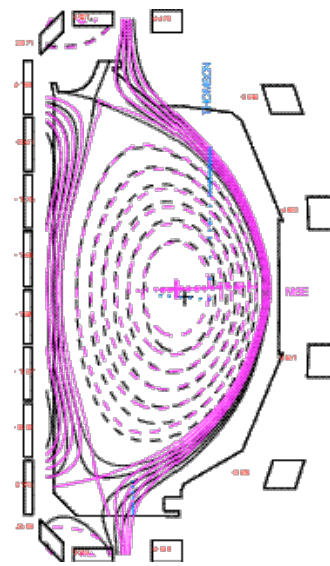
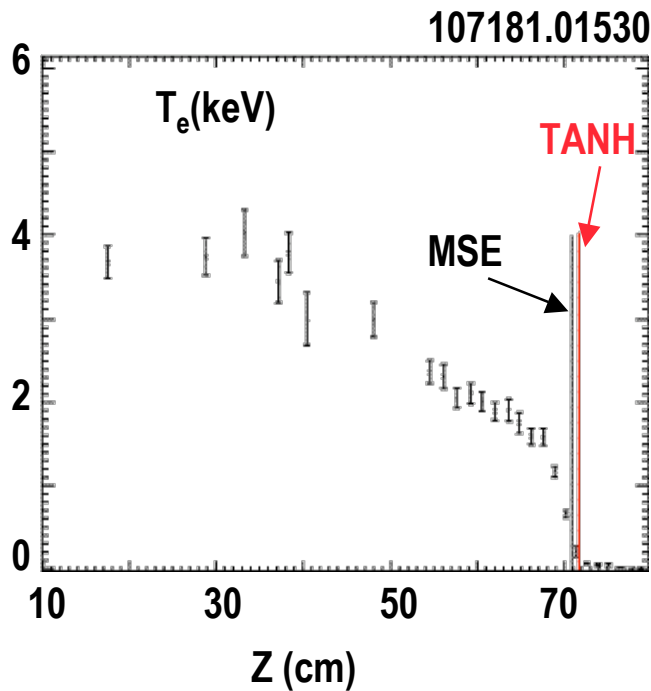
# SEPARATRIX LOCATIONS FROM EFIT AND THOMSON CAN DIFFER BY 1-2 cm IN DOUBLE-NUL DISCHARGES

- From AT stability experiment,  $\bar{n}_N \sim 4$ ,  $q_{95} \sim 4.2$



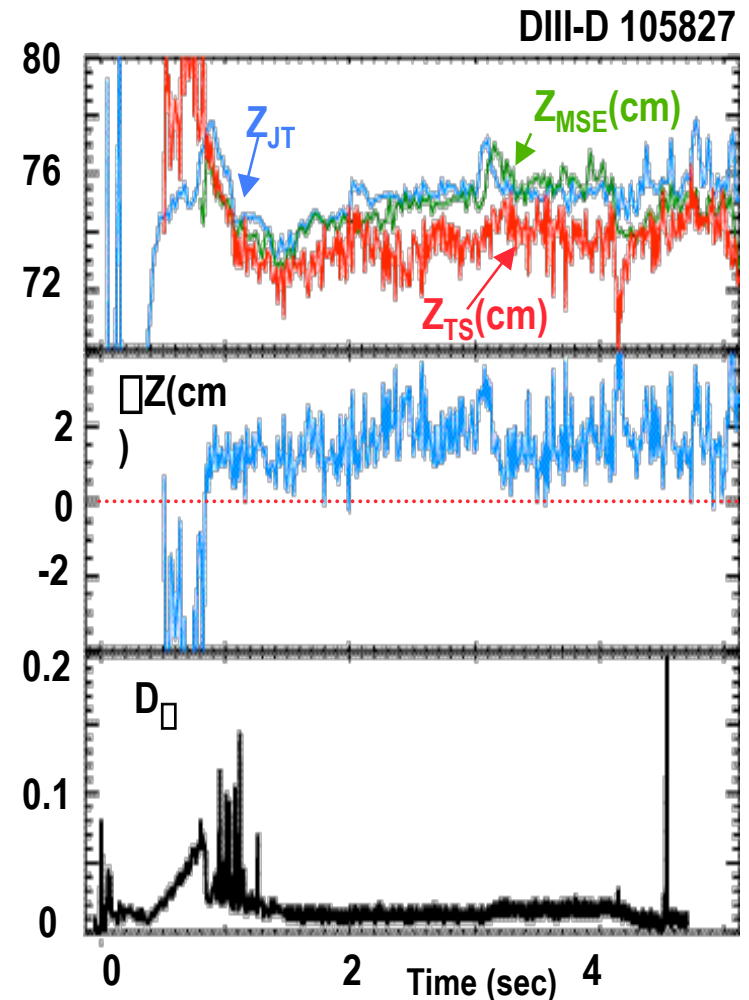
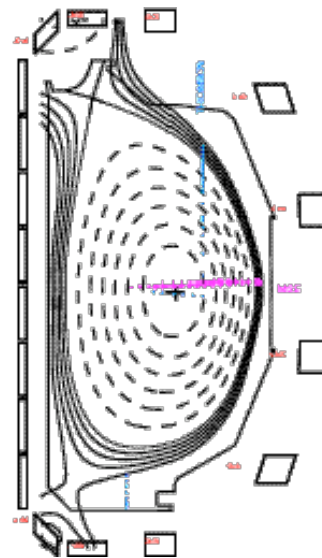
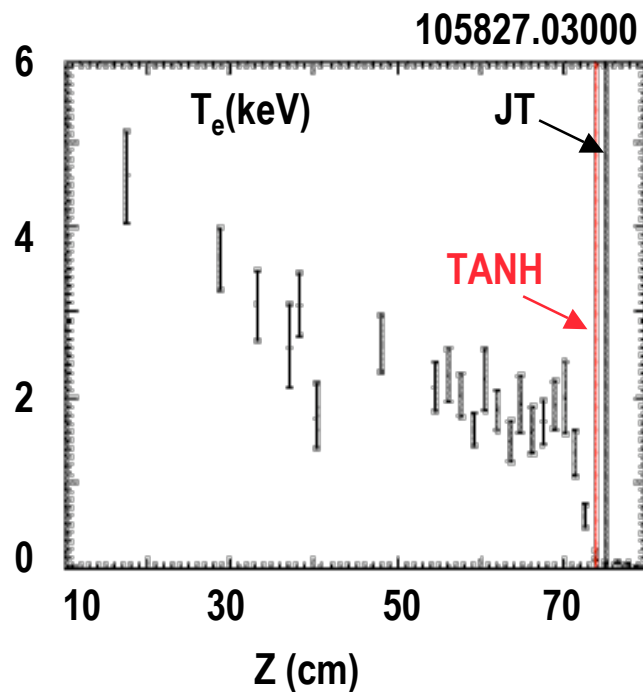
# ERROR FIELD CORRECTION COIL CURRENT APPEARS TO PLAY A ROLE IN THE EFIT-THOMSON SEPARATRIX LOCATION DIFFERENCE

- From AT stability experiment,  $\bar{n}_N \sim 3.8$ ,  $q_{95} \sim 5.1$ , higher  $C_{79}$  current



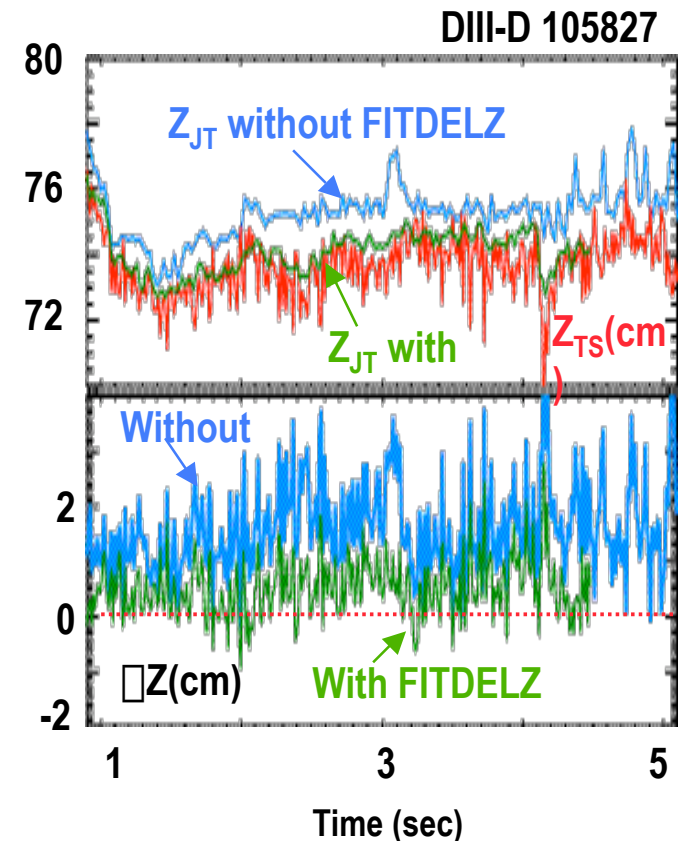
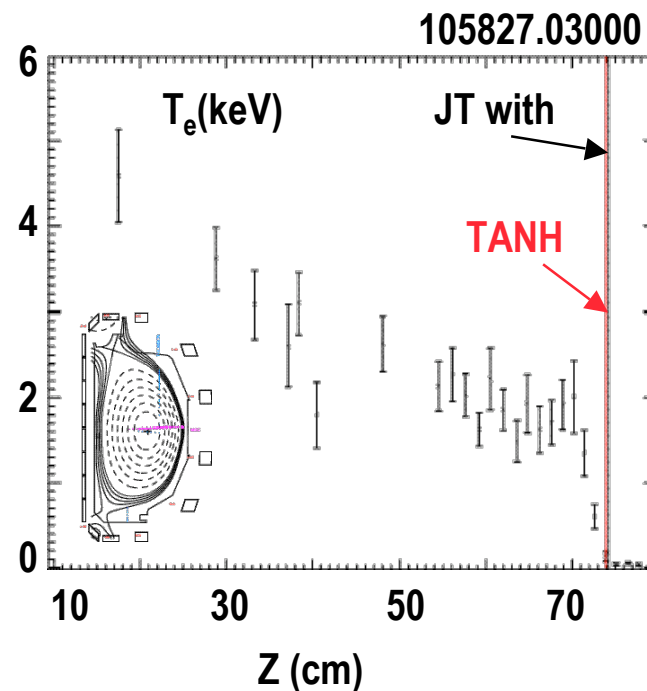
# SEPARATRIX LOCATIONS FROM EFIT AND THOMSON CAN DIFFER BY 1-4 cm IN UPPER SINGLE-NULL DISCHARGES

- From 2001 QH mode experiment using 1.3 MA shape control panel



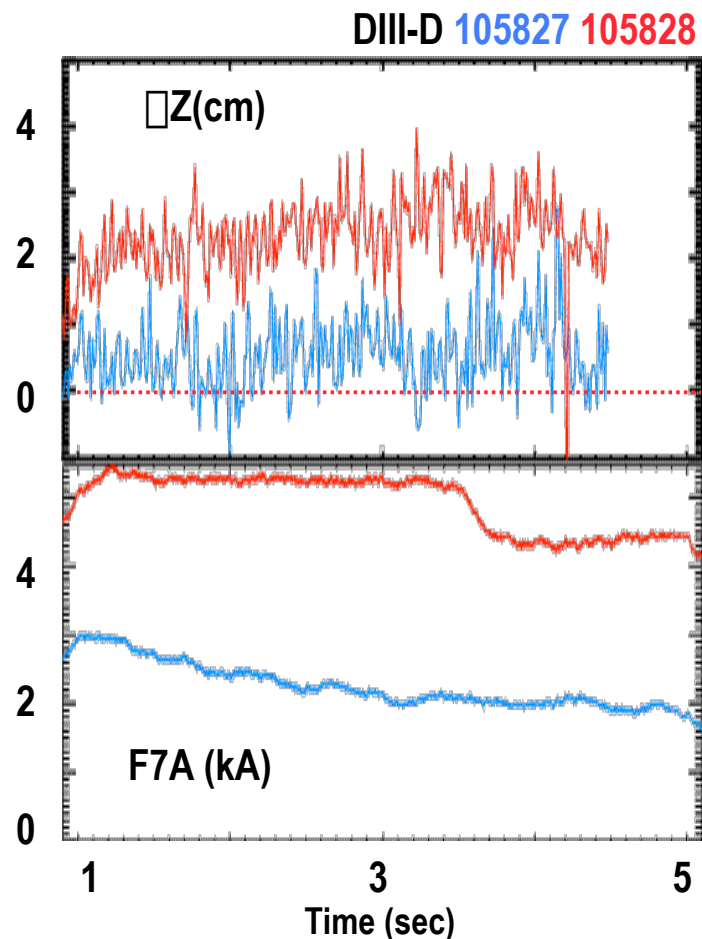
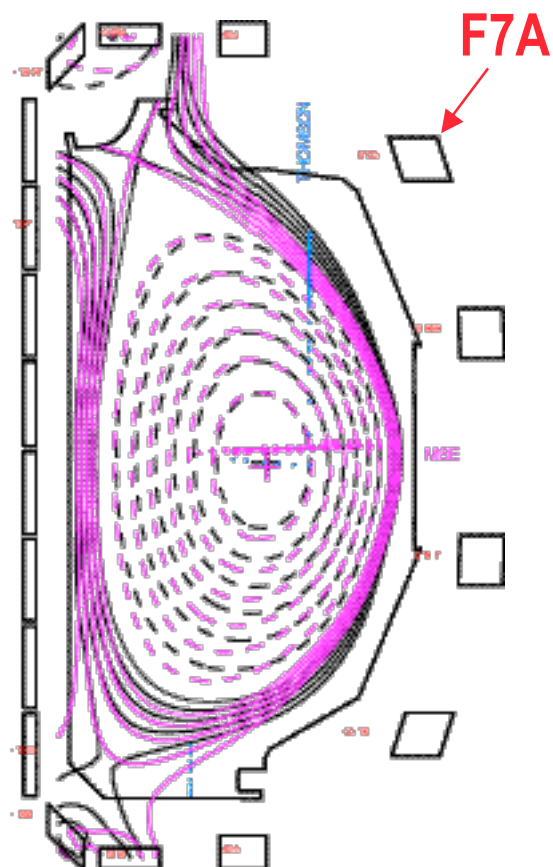
# VERTICAL POSITION FITTING SWITCH SOMETIMES HELPS FINDING A BETTER NUMERICAL SOLUTION

- From 2001 QH mode experiment using 1.3 MA shape control panel
- Z dependence only enters Grad-Shafranov equation implicitly
- $\psi$  is allowed to have a rigid body shift during iteration  $\psi^n = \psi^{n-1} + \Delta Z \psi_z^{n-1}$
- When effective both  $\psi^2$  and convergence are usually improved



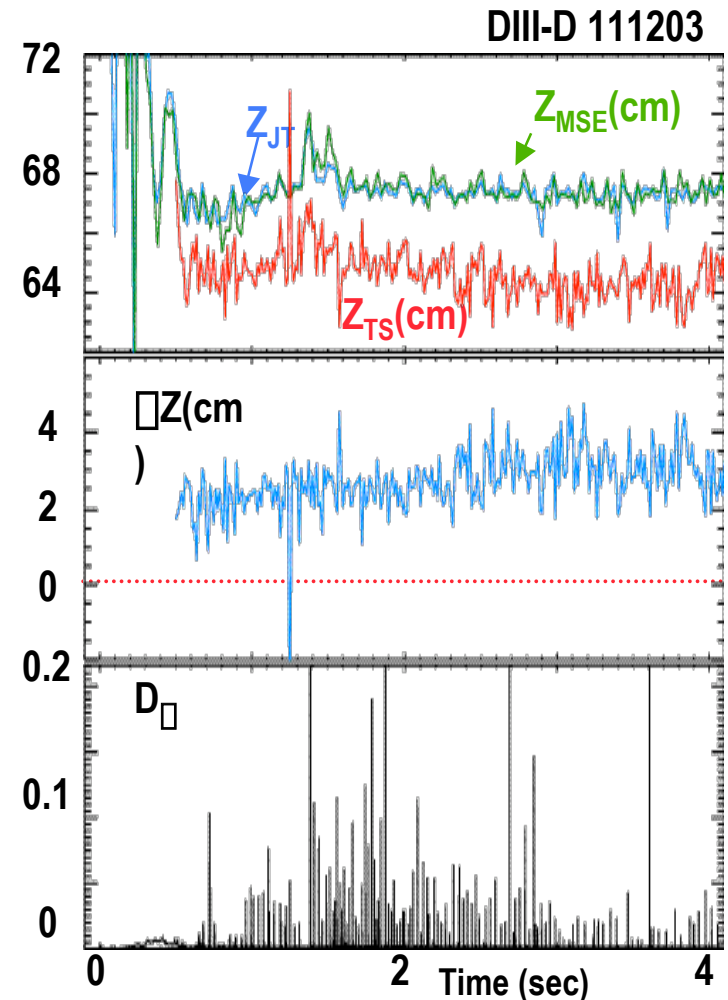
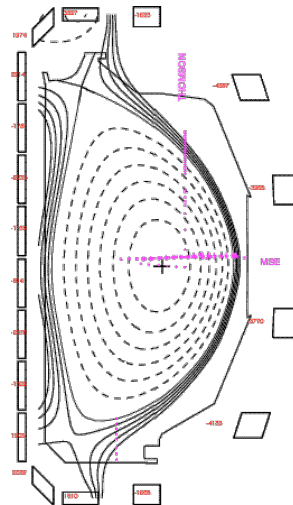
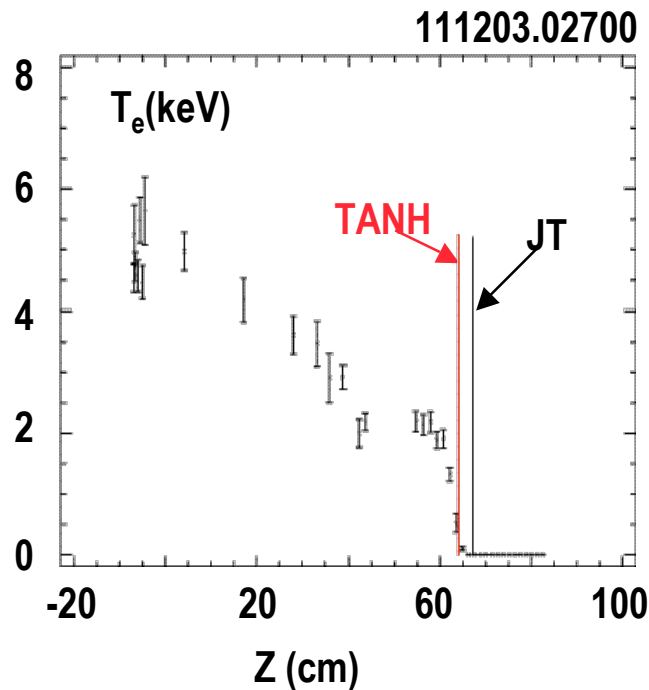
# CURRENT IN F7A APPEARS TO PLAY A ROLE IN THE EFIT-THOMSON SEPARATRIX LOCATION DIFFERENCE

- From 2001 QH mode experiment using 1.3 MA and 1.6 MA panels



# SEPARATRIX LOCATIONS FROM EFIT AND THOMSON CAN DIFFER BY 1-4 cm IN UPPER SINGLE-NULL DISCHARGES

- From 2002 AT ECCD experiment
- Metal plate on n1-coil has been removed but makes little difference



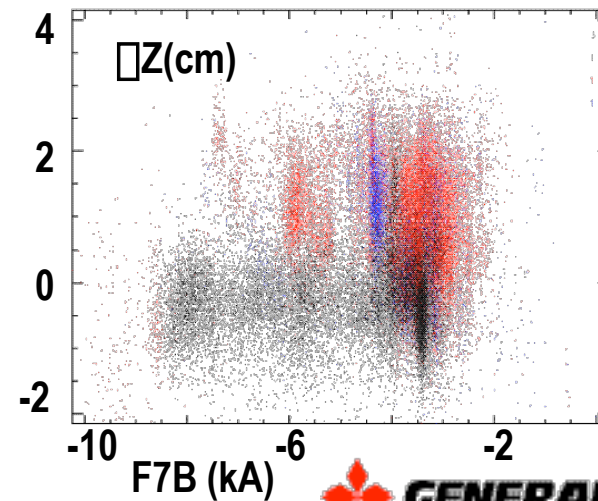
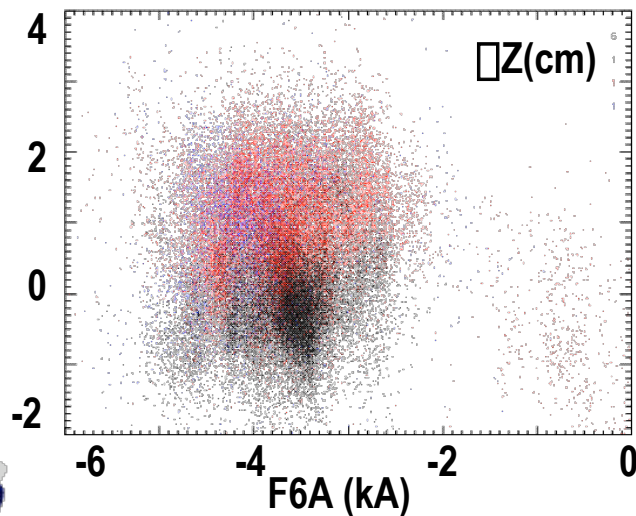
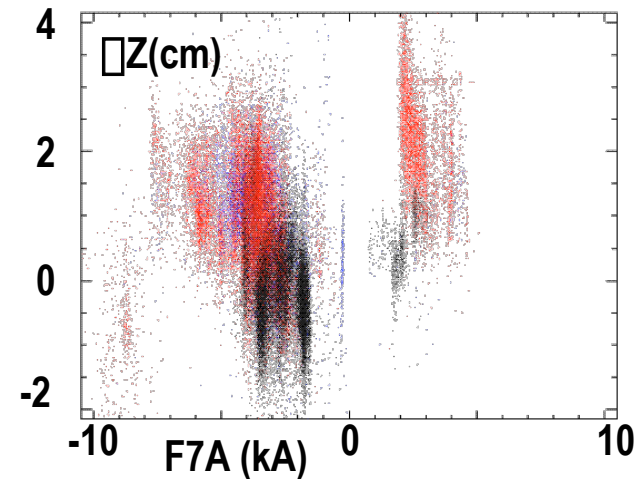
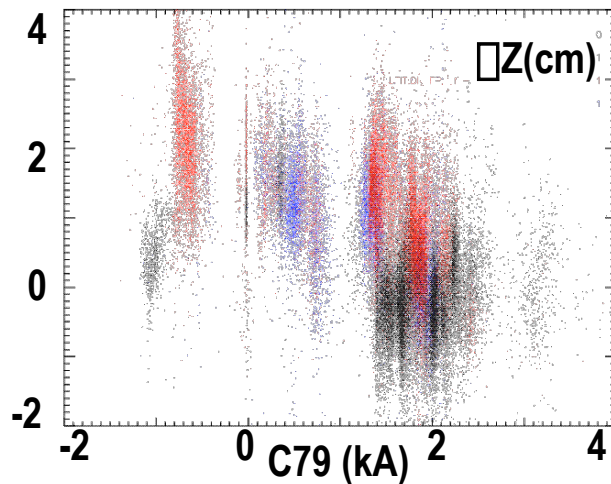
# DATABASE SURVEY

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- Shot range ~ 97600 - 105500, 1999 - early 2001 using an IDL code
- H-mode only with  $1 \text{ sec} < \text{time} < 4 \text{ sec}$  (based on electron edge pedestal height and width) and JT-like control room EFITs (EFIT01), sorted by
  - Lower single null (LSN),  $\Delta R_{\text{SEP}} < -1 \text{ cm}$
  - Double null (DND),  $-1 \text{ cm} < \Delta R_{\text{SEP}} < 1 \text{ cm}$
  - Upper single null (USN),  $\Delta R_{\text{SEP}} > 1 \text{ cm}$
- EFIT-Thomson separatrix difference generally within acceptable bound for LSN, but can differ by 1-4 cm in DND and USN cases
  - No correlation with electron edge pedestal pressure,  $\Delta p$  and  $\Delta_N$ , lower triangularity
  - Weak correlation with upper triangularity and  $I_i$
  - Some correlation with F7A and C-Coil currents, and  $q_{95}$

# DATABASE SURVEY INDICATES SOME CORRELATION WITH CURRENTS IN F7A AND C-COIL

- Main sorting variables: **DND**, **USN**, and LSN. H-mode only, 1999 - early 2001



# DIVERTOR HEAT FLUX ANALYSIS

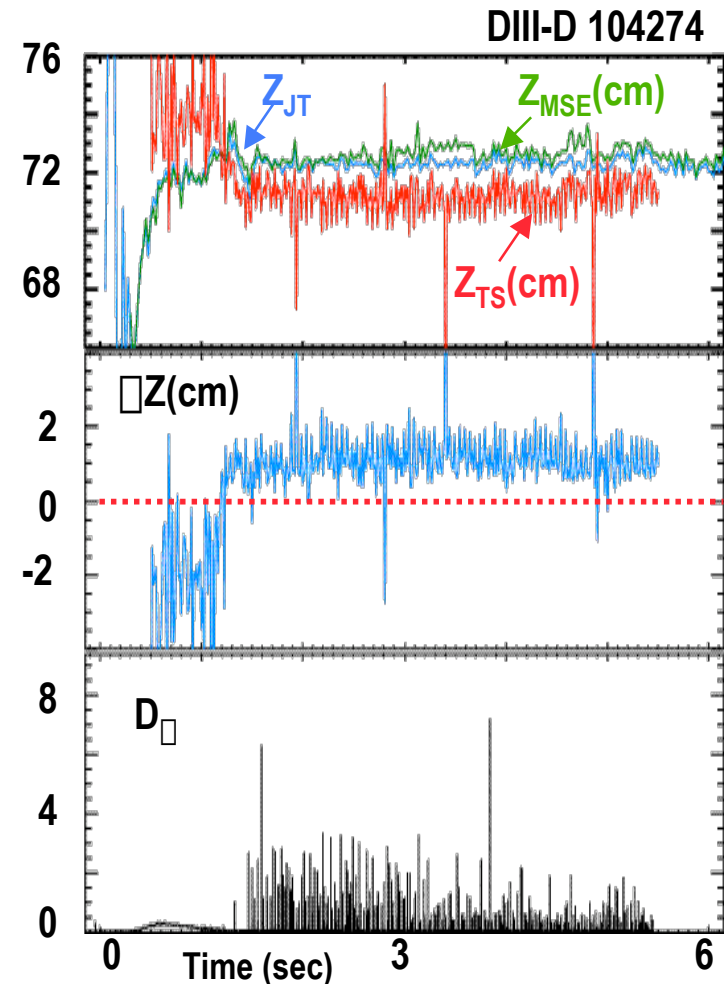
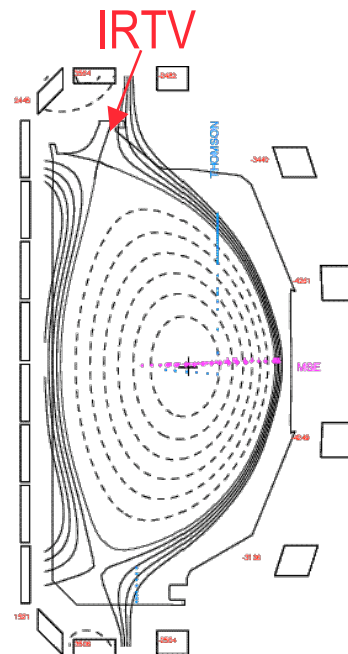
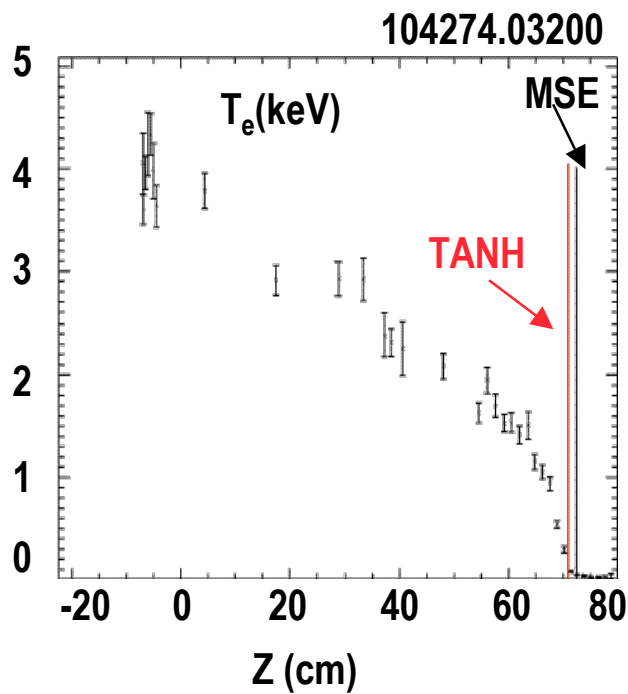
- To determine if the upstream  $T_e$  along the separatrix implied by the IR camera heat flux measurement ( $\square \sim -225^\circ$ ) is consistent with that from Thomson scattering based on EFIT mapping
- Approach
  - Use upper single-null discharge 104274 from a 2001 AT experiment
  - Assumed SOL in sheath-limited regime

$$n_{eu} \approx \frac{18.2 \times q_{\square}}{\square [T_{eu}]^{1.5} \times \left\{1 + \frac{T_{iu}}{T_{eu}}\right\}^{0.5} \times \left[\frac{B_z}{B}\right]_t}$$

- $\square$  = sheath heat transmission coefficient
- Assume peak  $q_{\square}$  is at separatrix location

# SEPARATRIX LOCATIONS FROM EFIT AND THOMSON DIFFER BY ~ 1-2 cm IN THIS USN DISCHARGE

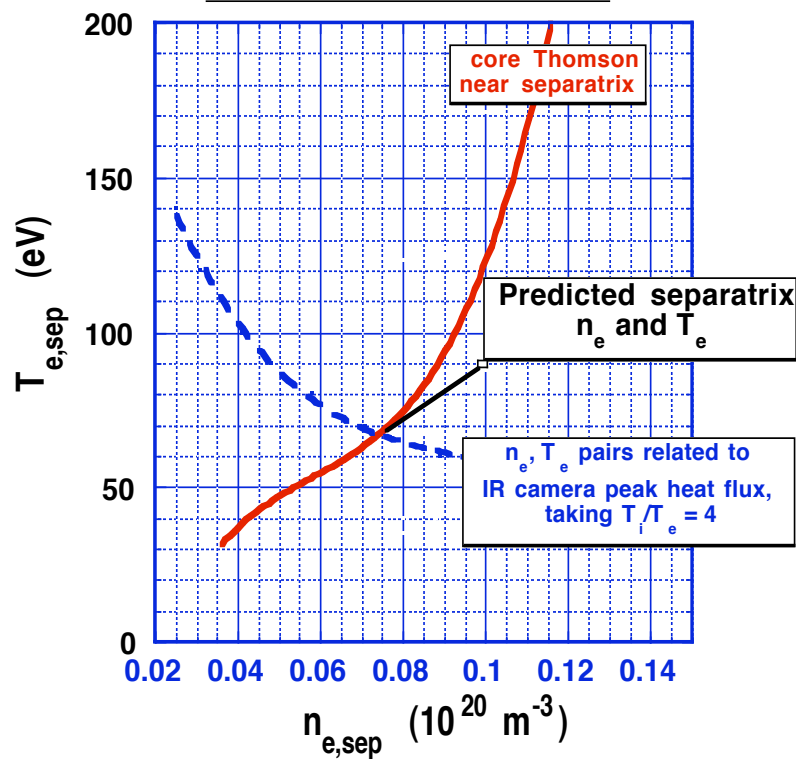
- From 2001 AT experiment,  $\bar{n}_N \sim 2.7$ ,  $q_{95} \sim 4.3$



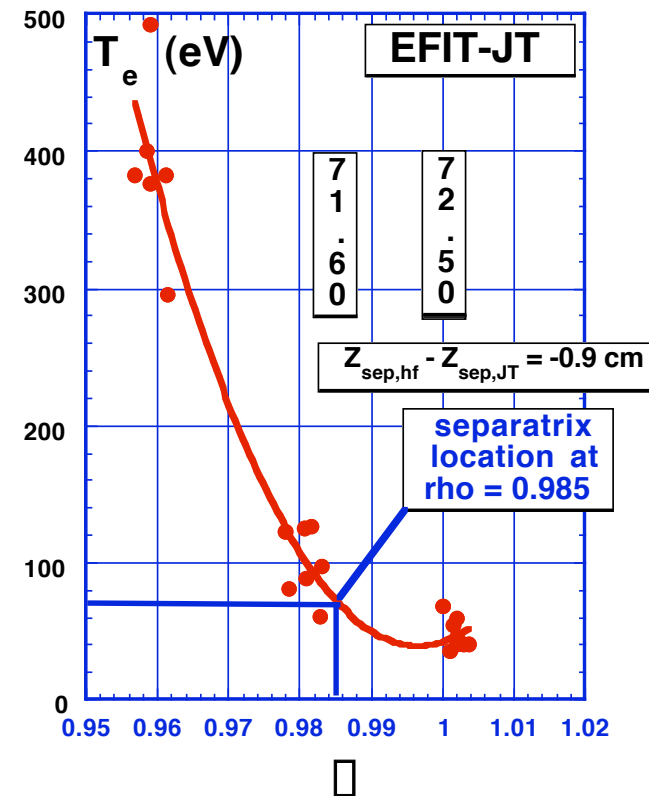
# HEAT FLUX ANALYSIS RESULTS ARE CONSISTENT WITH A 1-2 cm DIFFERENCE SEEN BETWEEN EFIT AND THOMSON

- Using JT-like EFIT for mapping

UPSTREAM ELECTRON DENSITY AND TEMPERATURE ALONG THE SEPARATRIX MUST SIMULTANEOUSLY SATISFY BOTH THOMSON SCATTERING AND HEAT FLUX CONDITIONS



**SHEATH-LIMITED ESTIMATE OF SEPARATRIX LOCATION IS SLIGHTLY AT VARIANCE WITH EFIT-JT**



# THE EFFECTS OF ERROR FIELD ON MAGNETIC SURFACES ARE ESTIMATED USING A PERTURBATIVE APPROACH

- F-coil irregularities has recently been re-measured
  - F7A has largest ~ 1.2 cm radial shift as in previous measurements [1]
- Perturbative 3-D calculations
  - 3-D effects taken to be small, plasma assumed to remain in a 2-D axisymmetric equilibrium state
  - 3-D corrections to the magnetic surfaces are then computed by superimposing the asymmetric external magnetic field onto the background toroidally symmetric magnetic field due to the plasma using a Green's function approach
  - PLOTLINE code

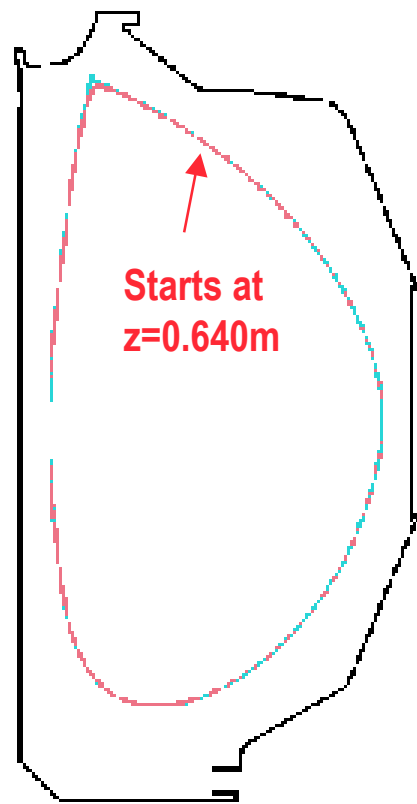
Radial Deviations of F-coil from B-coil (Luxon, Schaffer)

F-coil	Toroidal Angle	$\Delta r$ (cm)
F1A	24.237	0.210
F2A	-16.500	0.250
F3A	3.336	0.322
F4A	9.155	0.247
F5A	-37.307	0.271
F6A	-153.112	0.554
F7A	-101.527	1.206
F8A	-81.411	0.361
F9A	-95.623	0.310
F1B	-58.591	0.125
F2B	-43.359	0.295
F3B	-73.825	0.277
F4B	-72.300	0.185
F5B	-92.760	0.261
F6B	-108.792	0.692
F7B	-137.713	0.739
F8B	-163.708	0.604
F9B	-113.650	0.534

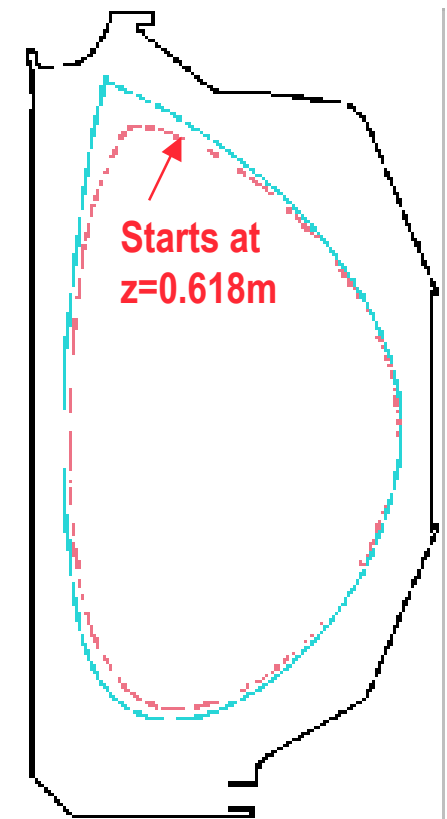
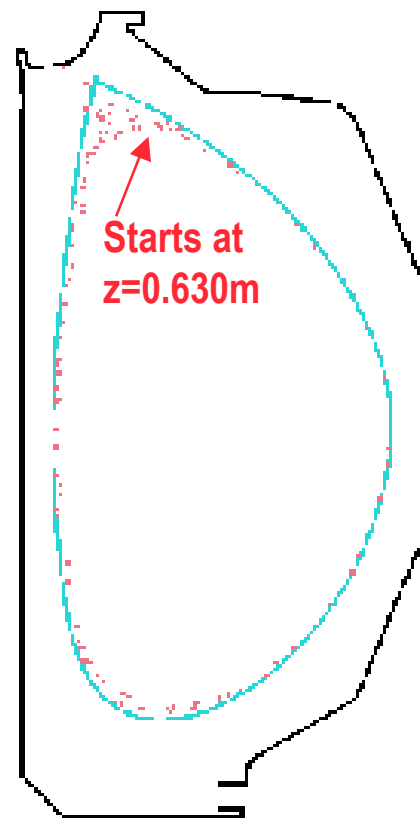
# A 1-2 cm RADIAL SHIFT IN F-COILS CAN LEAD TO FORMATION OF A LARGE EDGE STOCHASTIC REGION

- 3-D perturbative calculations based on QH-mode USN discharge 105828 at 3000 ms

Axisymmetric

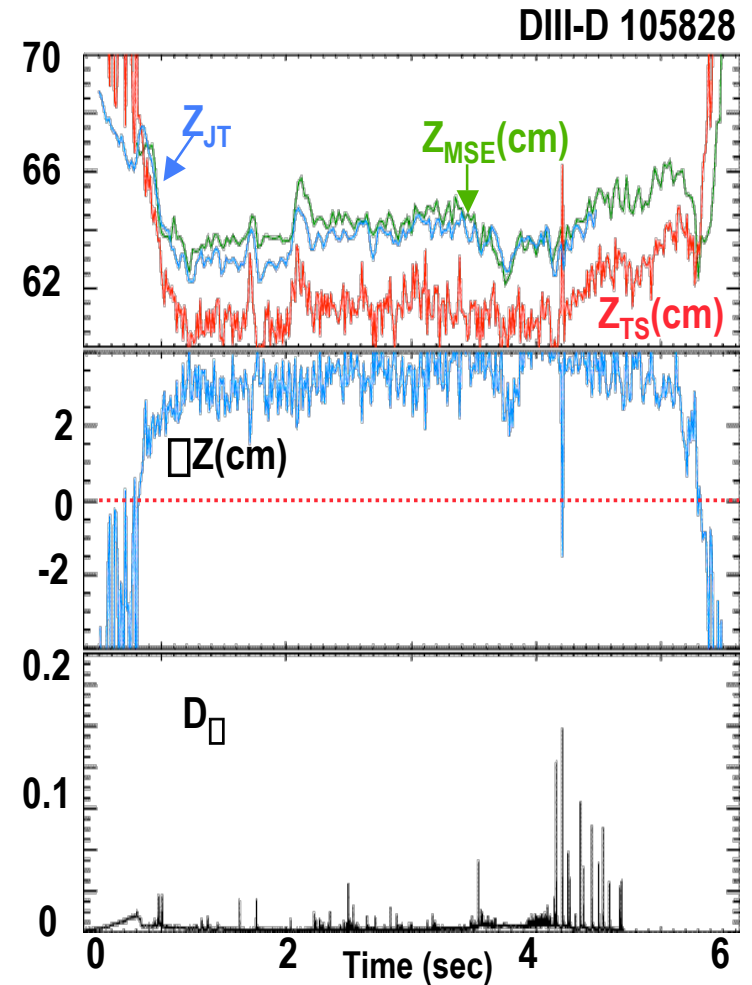
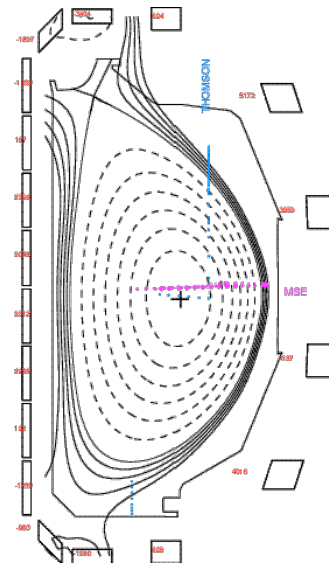
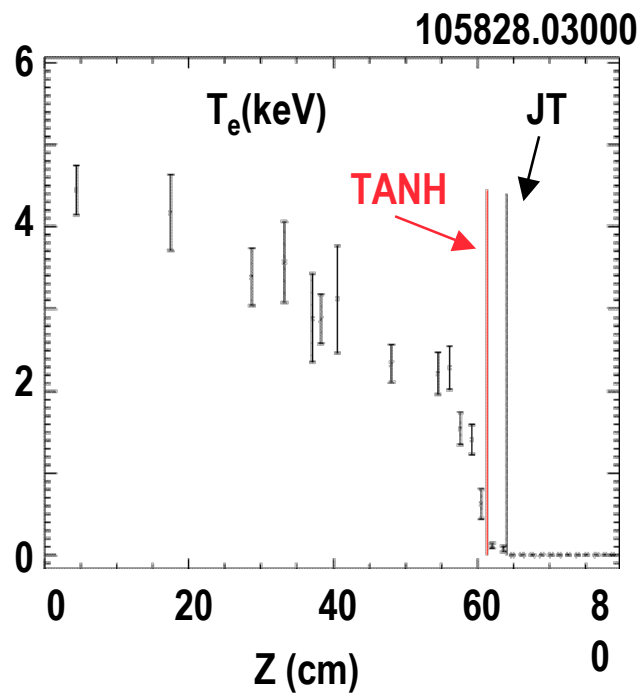


Shifted F-coils  
+ C-coils



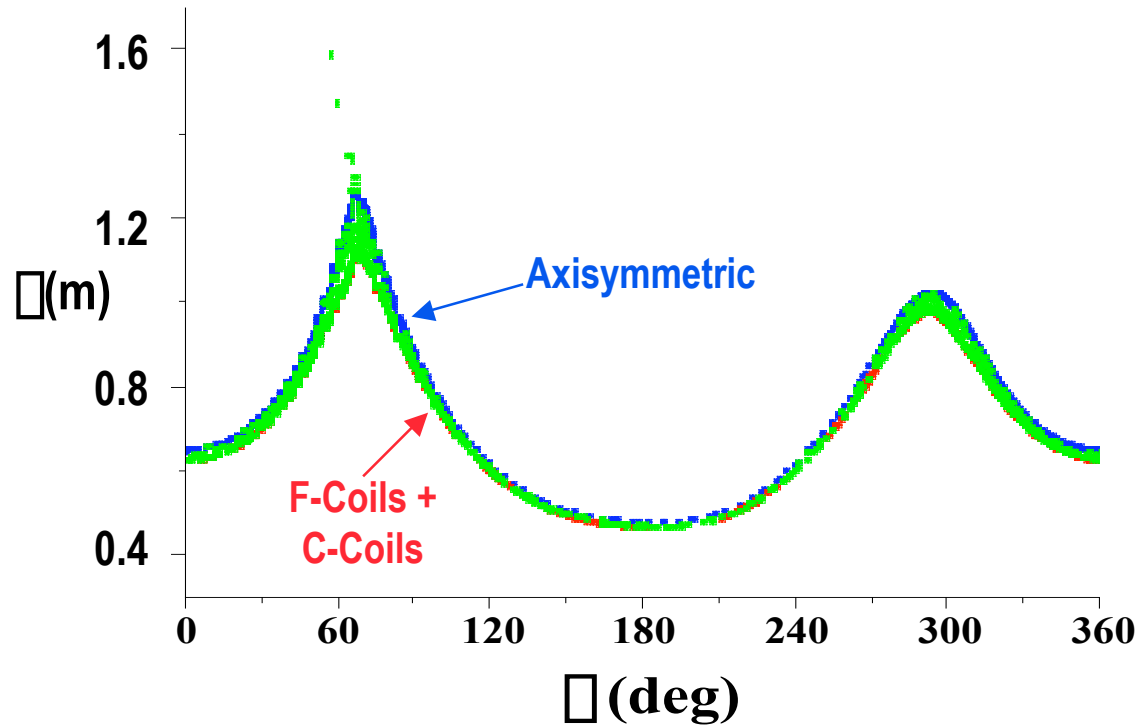
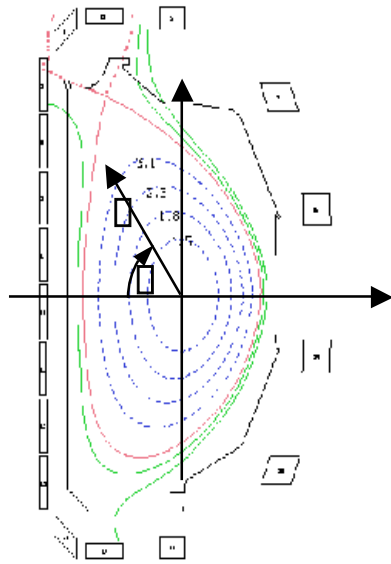
# SEPARATRIX LOCATIONS FROM EFIT AND THOMSON DIFFER BY 2-4 cm IN THIS UPPER SINGLE-NULL DISCHARGE

- From 2001 QH mode experiment using 1.6 MA shape control panel



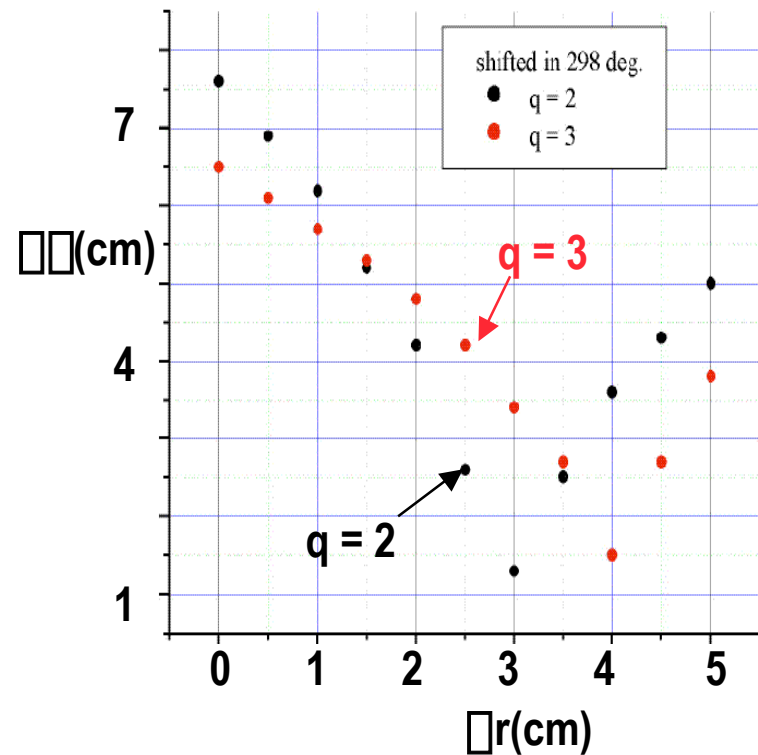
# A SMALL TOROIDAL ASYMMETRY OF F-COIL LOCATION CAN PRODUCE A LARGE EDGE STOCHASTIC REGION

- DIII-D USN discharge 105828.03000

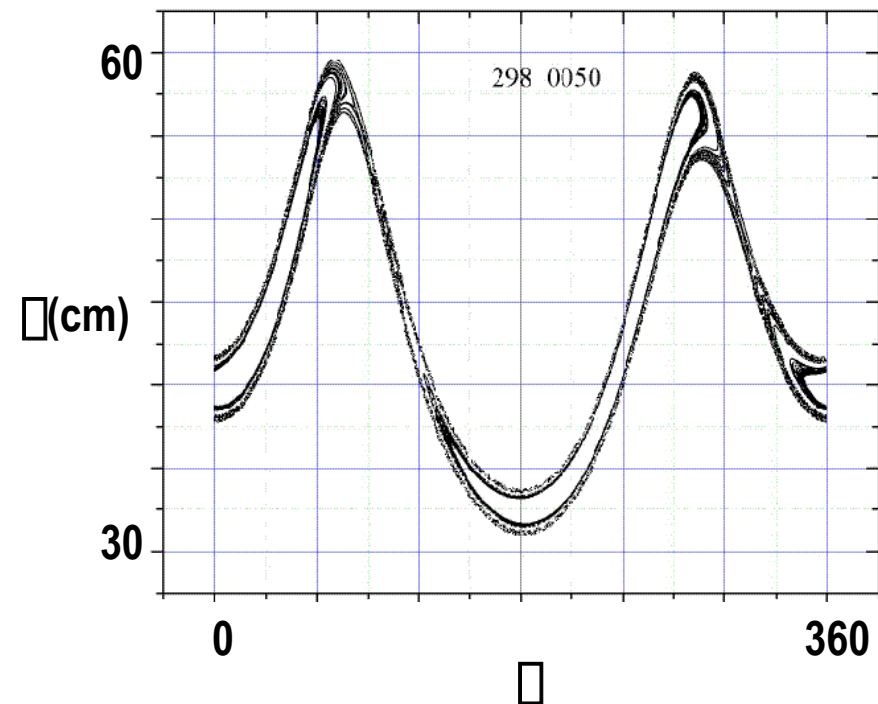


# C-COIL CAN PRODUCE LARGE MAGNETIC ISLANDS AT OUTER RATIONAL $q$ SURFACES WITHOUT PLASMA RESPONSE

- F-coil shifted uniformly along the radial direction at  $\theta = 298$  degree to reduce the C-coil effects at  $q = 2$  and 3 surfaces
- Simulations based on DIII-D USN discharge 105828.03000

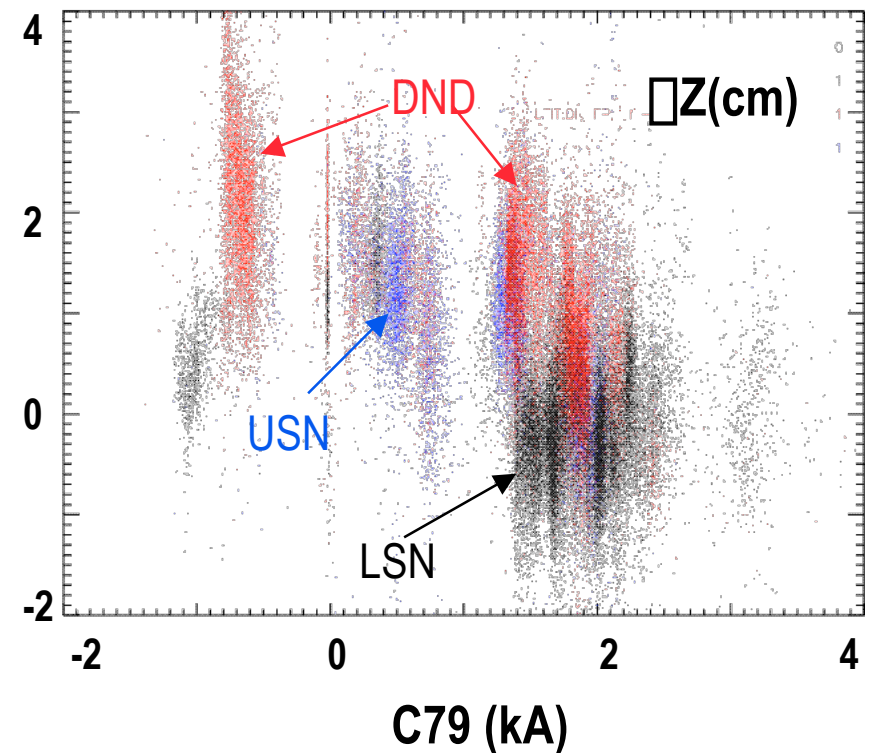
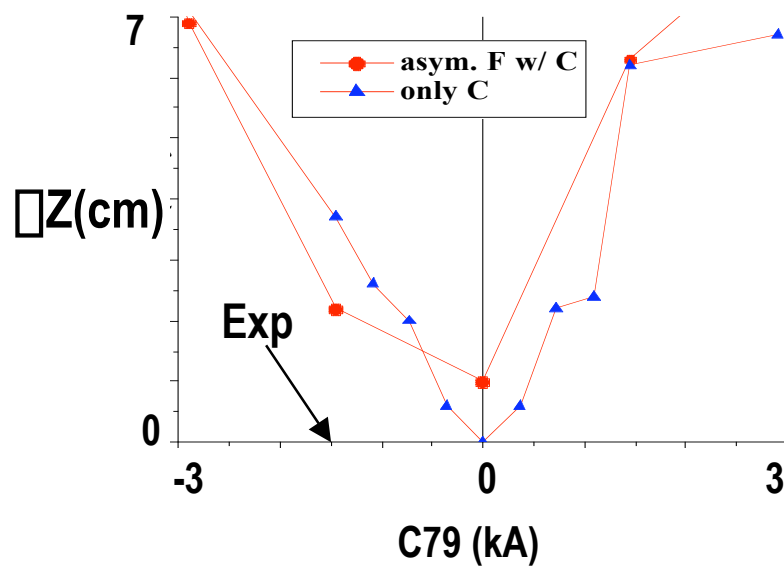


$r = 5$  cm at  $\theta = 298$  degree



# C-COIL RESPONSES ARE NOT CONSISTENT WITH OBSERVATIONS WITHOUT PLASMA RESPONSE

- Main sorting variables: **DND**, **USN**, and LSN. H-mode only, 1999 - early 2001
- Simulations based on USN discharge 105828.03000 with asymmetric F-coils + C-coils and with C-coils only



# STATUS OF 3-D EQUILIBRIUM / RECONSTRUCTION TOOLS TO MODEL PLASMA RESPONSE

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- 3-D equilibrium
  - **VMEC**, restricted to nested flux surfaces
  - **PIES**, can model magnetic island and stochasticity but computationally intensive
  - Plasma response from linear stability code such as **GATO** and **MARS**
    - **V3RFUN + V3POST** [1]
- 3-D reconstruction tools in development
  - **V3FIT**, stellarator reconstruction project based on the efficient EFIT response function approach + VMEC (ORNL, GA, Auburn University) **RP1 27**
  - Complementary integral approach
    - Filamental plasma response **MFIT3D** (GA, PPPL)
    - Full plasma response **EFIT3D**

[1] Hirshman, Lazarus, Hanson, Knowlton, Lao to appear in Physics Plasma

# SUMMARY / PLAN

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- A 1 - 4 cm difference still exists between the separatrix vertical position determined from EFIT magnetic analysis and those inferred from Thomson  $T_e$  measurements
  - Removal of metal plate on n1-coil makes little difference
- Perturbative calculations without plasma response indicate a small toroidal asymmetry in the external coil location can introduce significant 3-D magnetic effects such as formation of edge stochastic layers and magnetic islands into the magnetic topology
- Response of error magnetic field correction coils (C-Coils) suggest that plasma response may play a crucial role
  - C-coil currents tend to increase the error magnetic field due to the external shaping coils
- Plan
  - New magnetic uncertainty matrix
  - 3-D equilibrium analyses / development
    - VMEC / PIES analyses
    - GATO / MARS + V3RFUN + V3POST
    - MFIT3D, EFIT3D