

Current Profile Control Issues

Advanced Tokamak Workshop
General Atomics, 3/9-11/99

C. Kessel

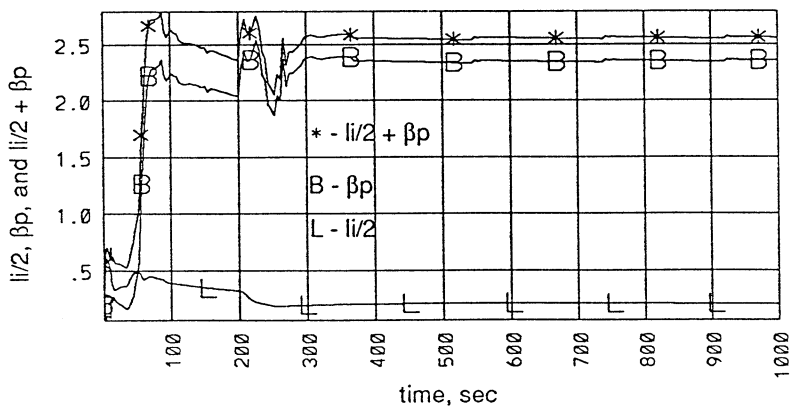
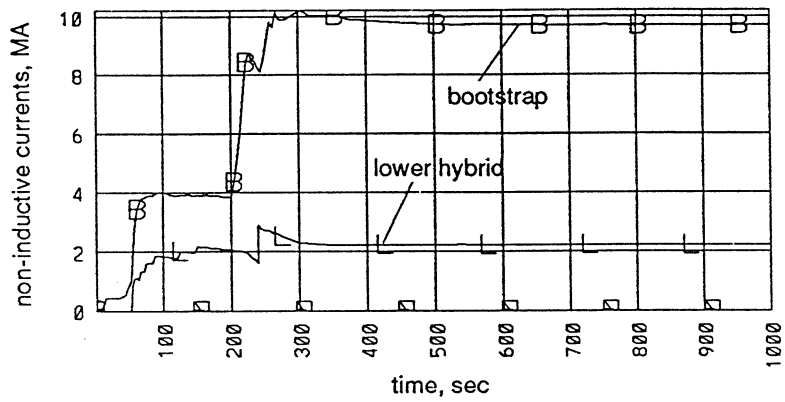
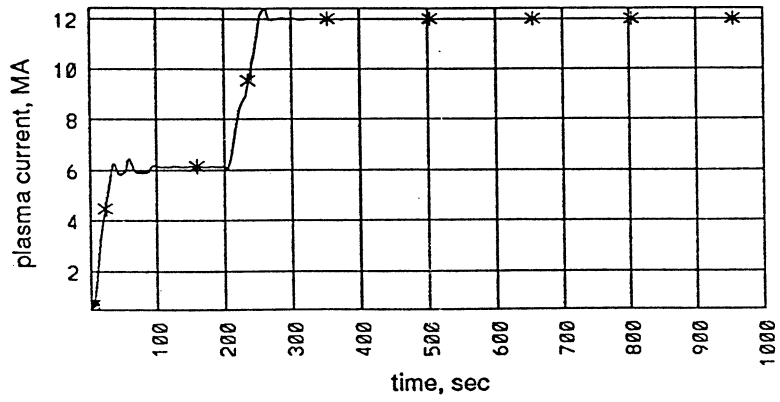
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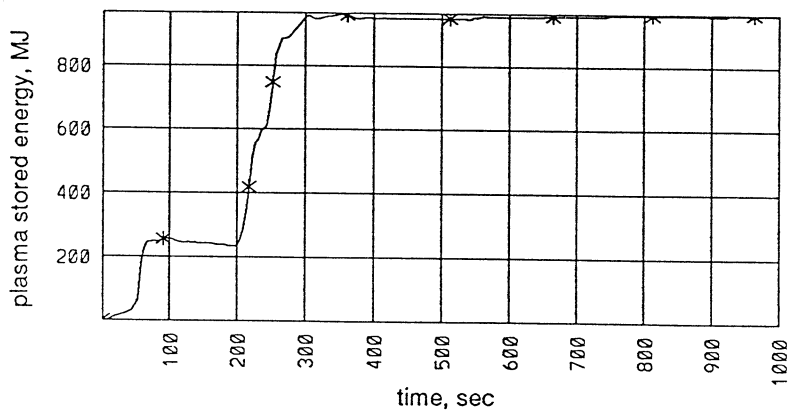
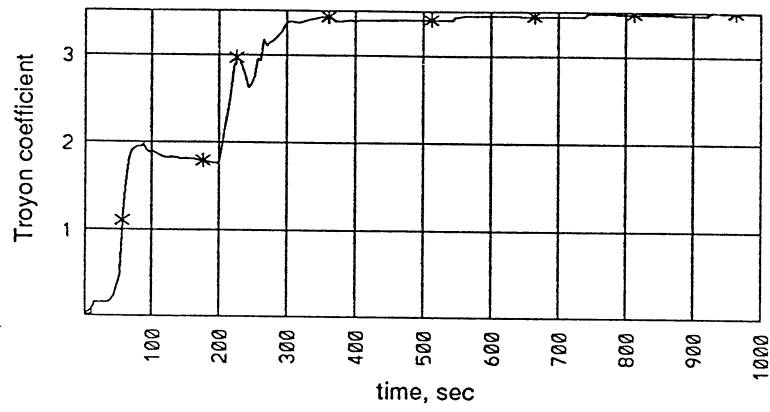
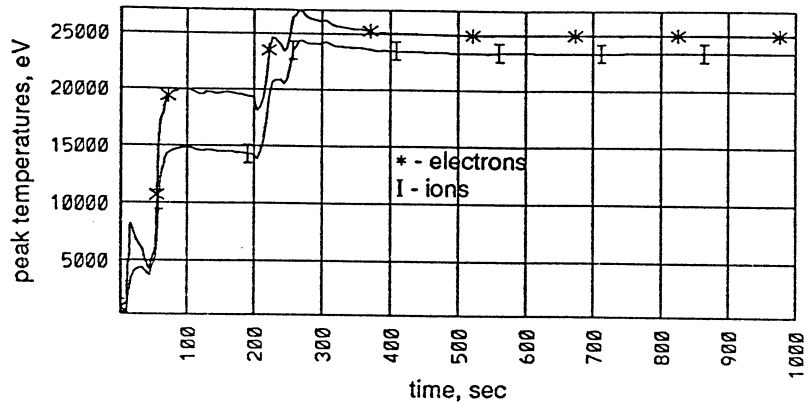
Introduction

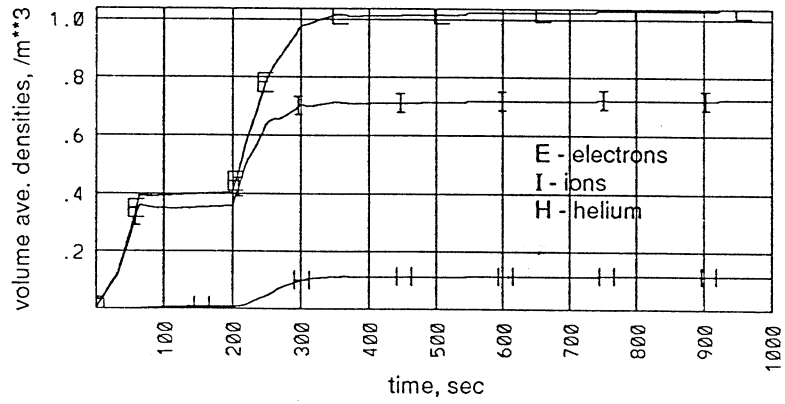
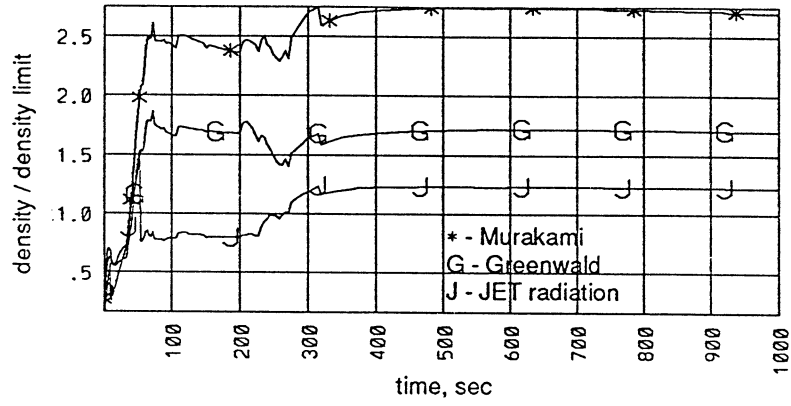
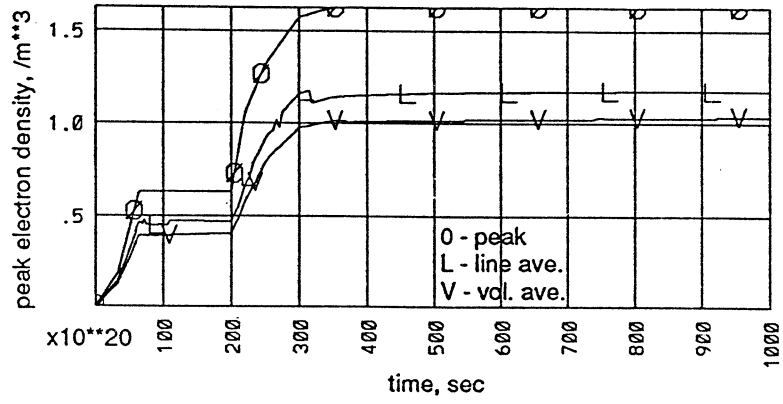
- the current profile control problem in tokamaks breaks into two distinct areas
 - controlling in the presence of significant inductive current
 - controlling fully non-inductive current
- these two areas further break down into
 - pre-programmed current profile control (open loop)
 - feedback current profile control (closed loop)
- present results of TSC simulations of the ITER reverse shear discharge
- present results of attempt to model CMOD evolution to a reverse shear plasma

Simulation of ITER Reversed Shear Plasma

- this represents current profile control through pre-programming
- the discharge can be broken up into five phases
 - inductive plasma current rampup
 - transition to low β – low current non-inductive plasma
 - transition to high β – high current non-inductive plasma
 - flattop
 - rampdown (not modelled)

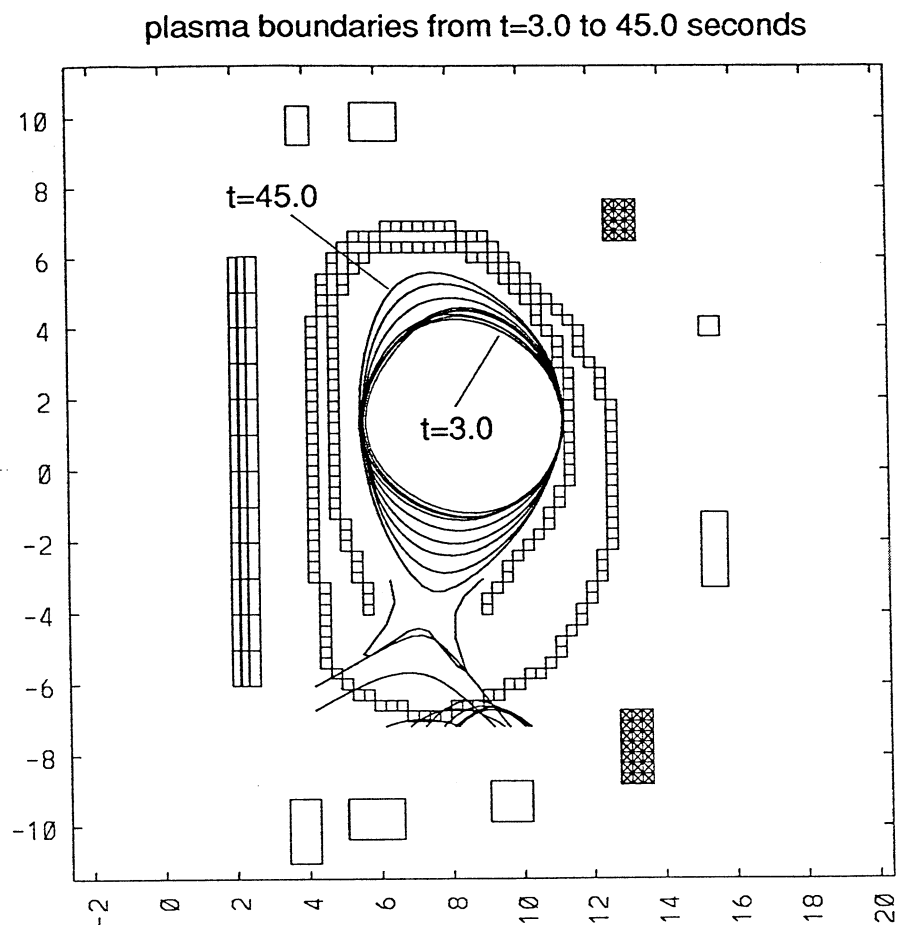


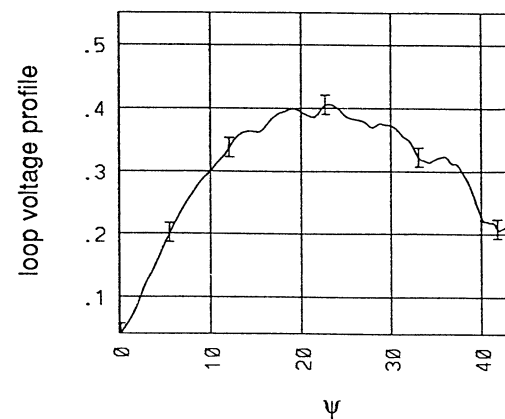
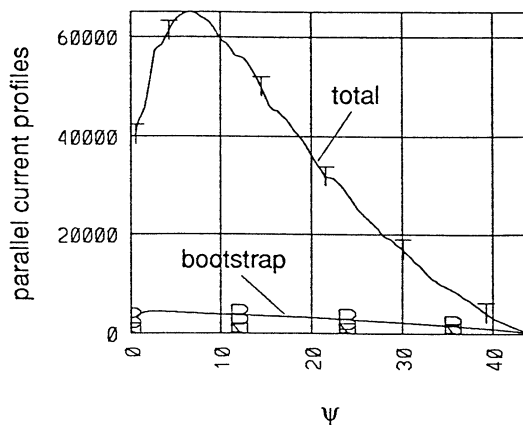
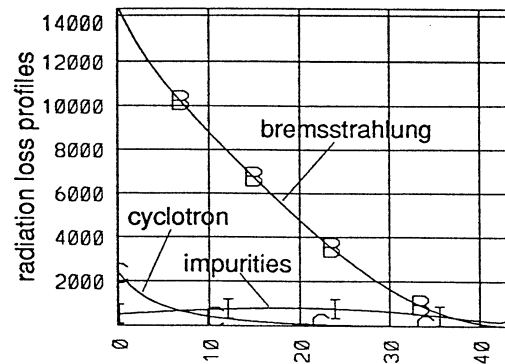
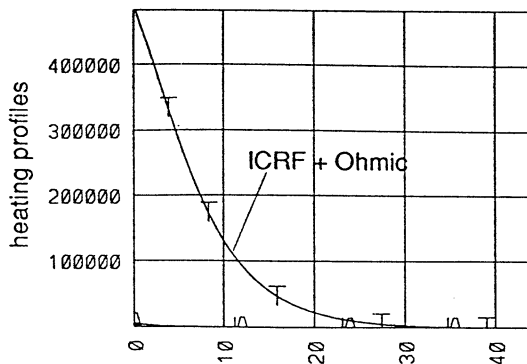
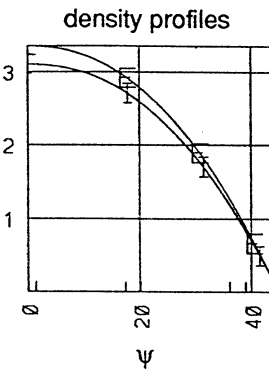
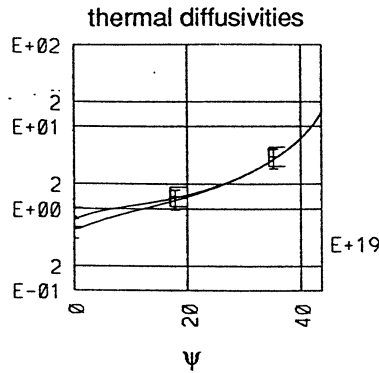
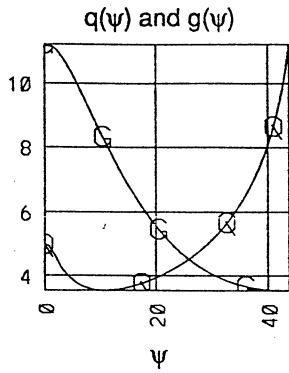
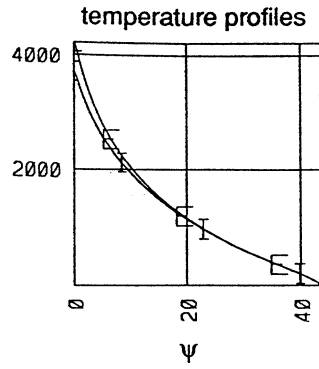
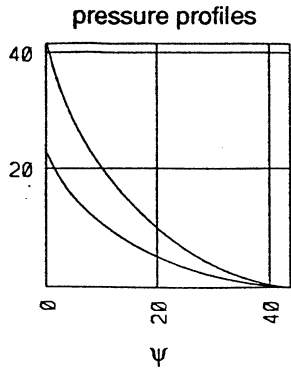


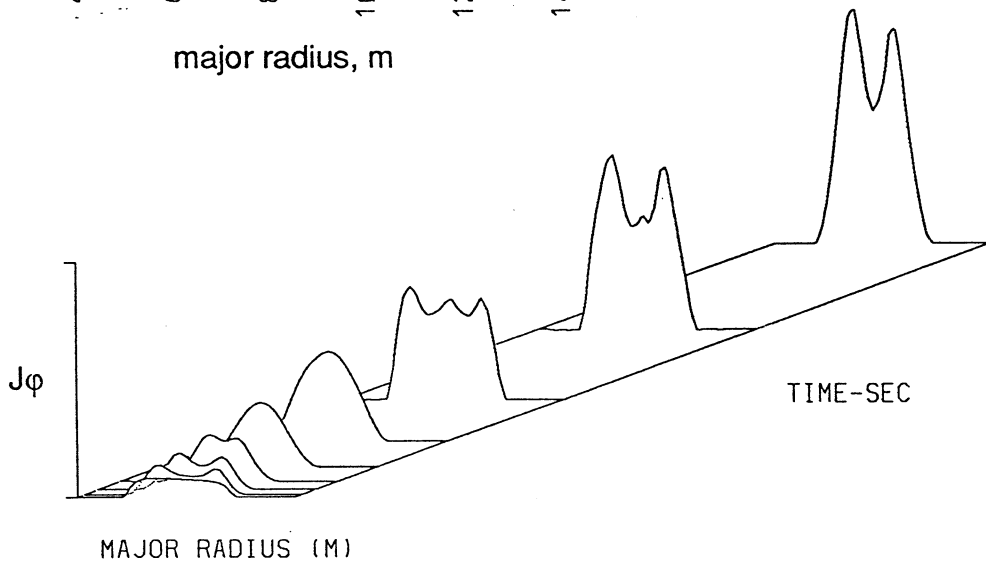
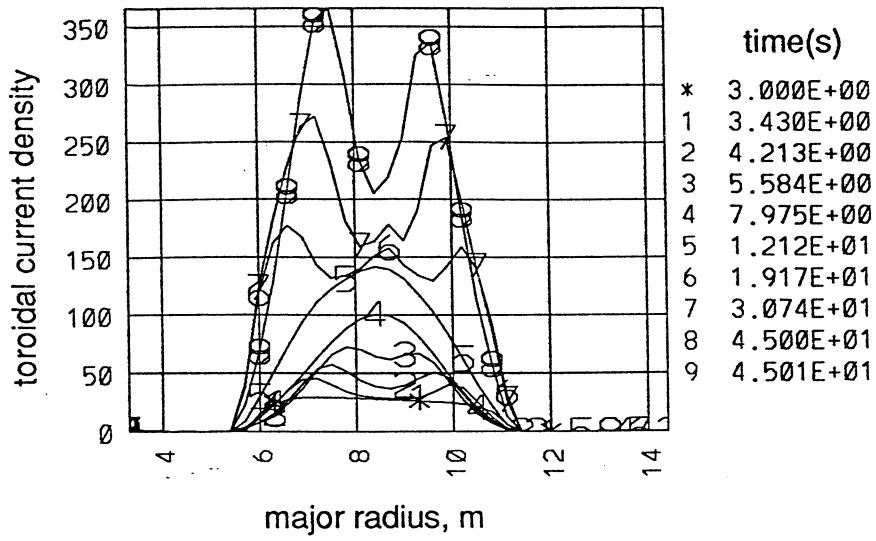


Simulation of ITER Reversed Shear Plasma Inductive Plasma Current Rampup, $t=0-45$ s

- the plasma starts as a large circular plasma and is shaped during this phase
- the plasma current is ramped up to 6 MA, half its final value
 - the lower plasma current is to avoid large inductive current which will diffuse to the core and make hollow profile formation difficult
- between 5-15 MW of power is injected to slow current diffusion
- the electron density is ramped up to $0.5 \times 10^{20} / \text{m}^3$

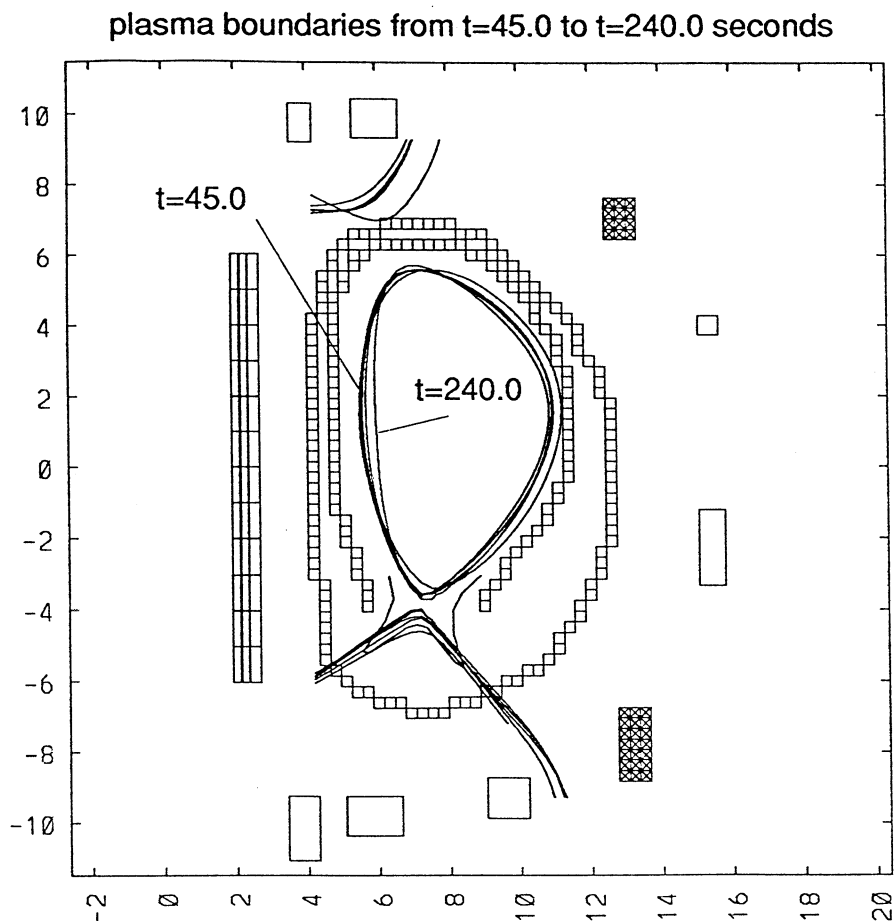


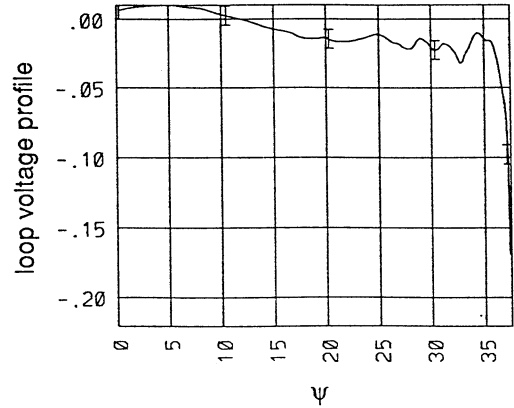
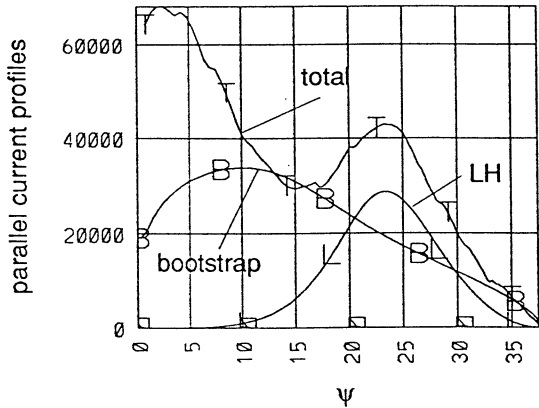
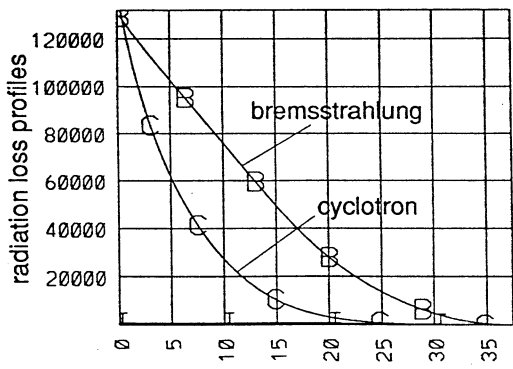
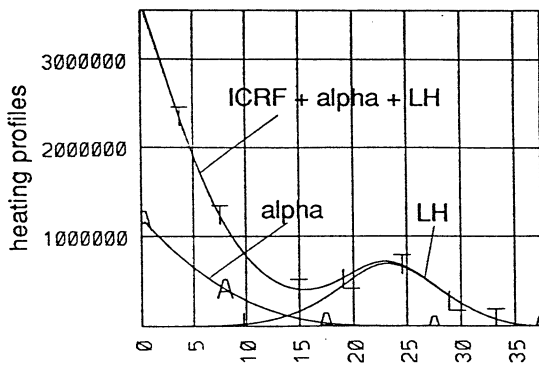
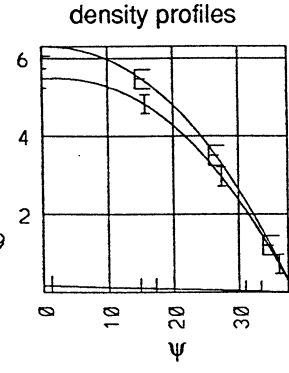
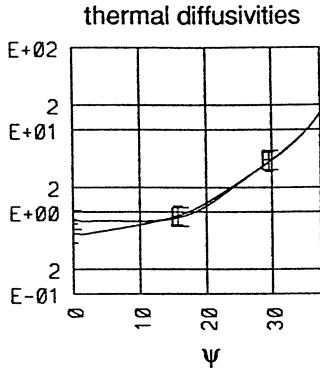
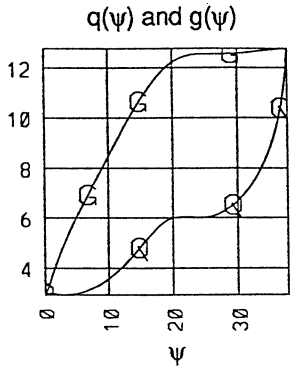
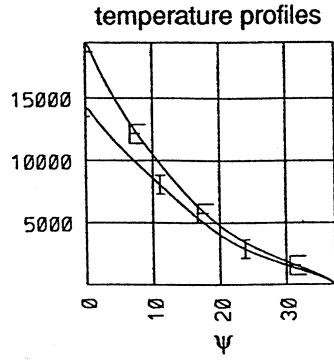
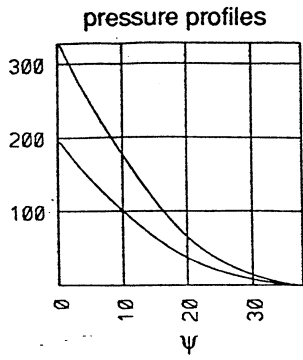


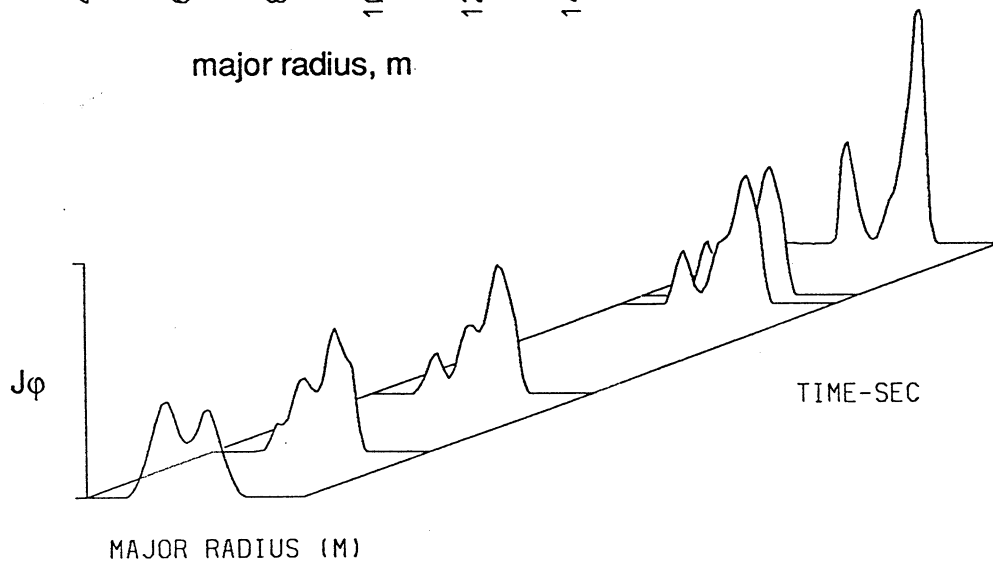
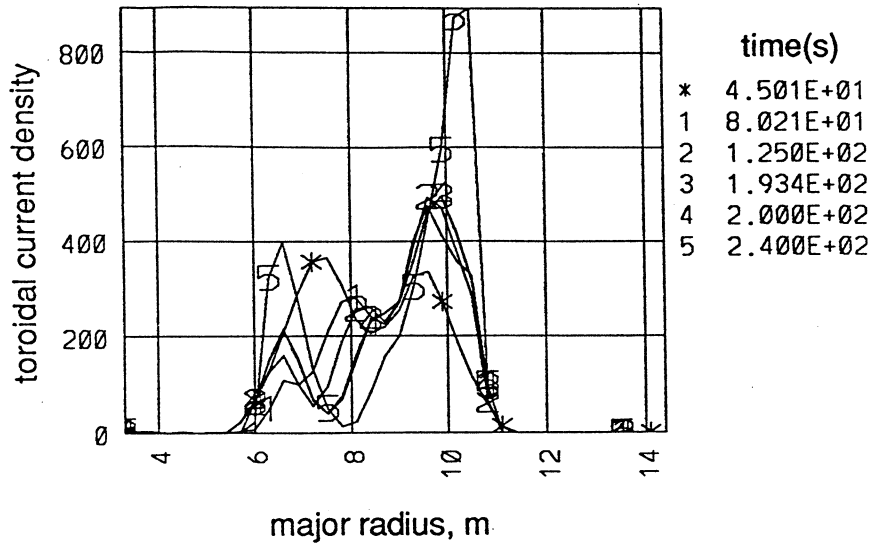


Simulation of ITER Reversed Shear Plasma Transition to Low- β – Low I_p , $t=35-240$ s

- the plasma current is held at 6 MA
- 50 MW of ICRF (FW) and 50 MW of LHCD are applied
- the peak electron density is fixed at $0.6 \times 10^{20} / \text{m}^3$
- β_p increases to 2.25 raising the bootstrap current to 4 MA
- the LHCD provides 2 MA
- the peak electron and ion temperatures become 20 and 15 keV
- l_i drops to 0.5 as the inductive current dissipates and the off-axis currents create a hollow profile
- the plasma minor radius drops and the plasma shaping is becoming more pronounced



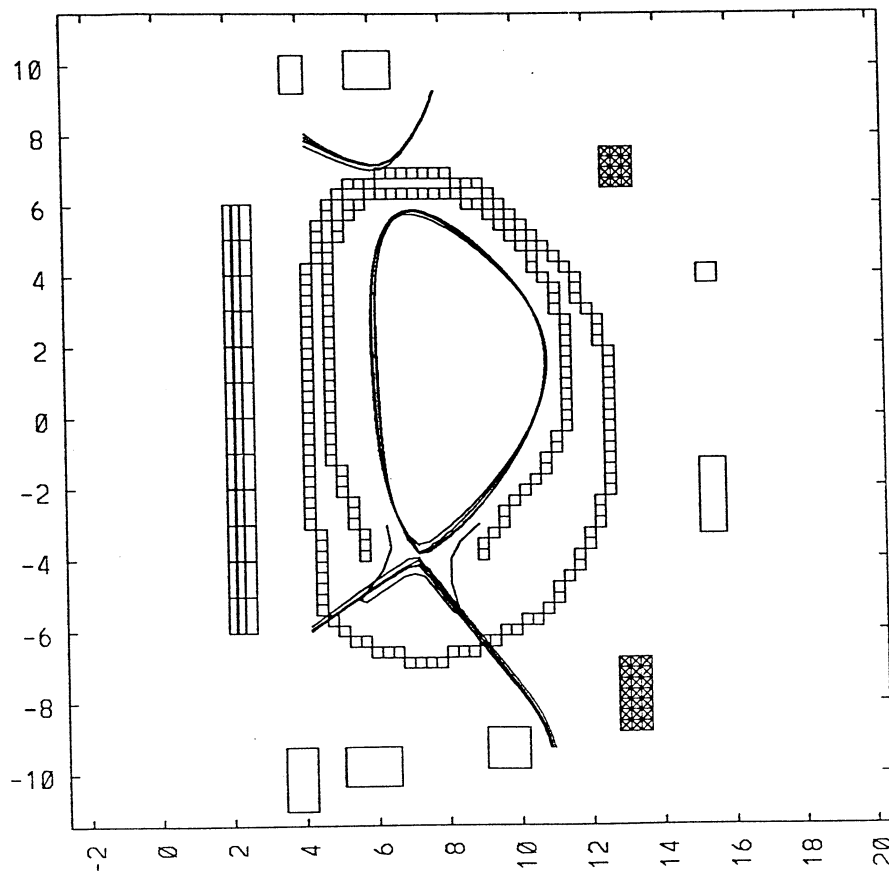


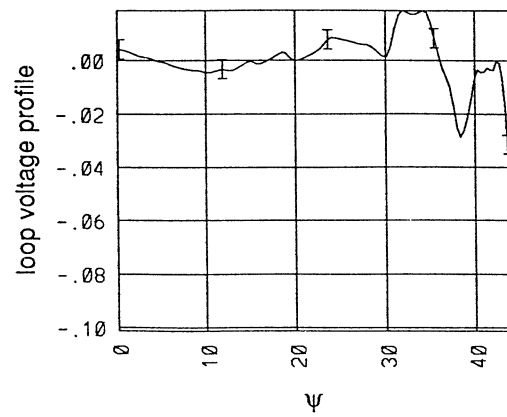
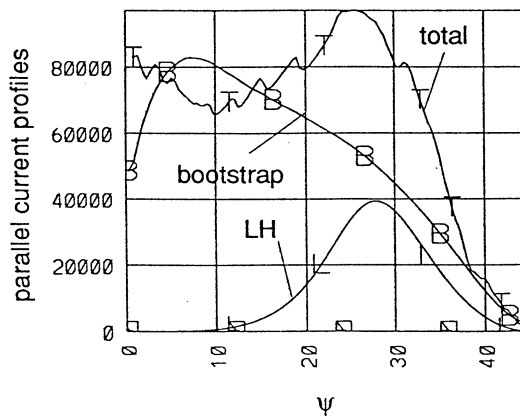
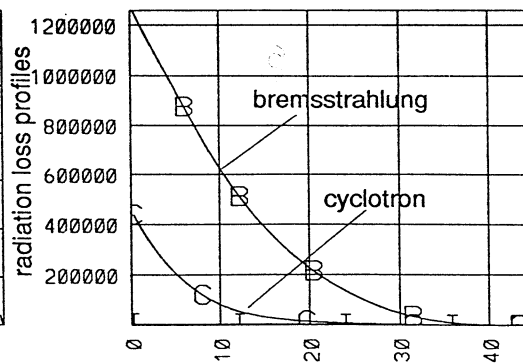
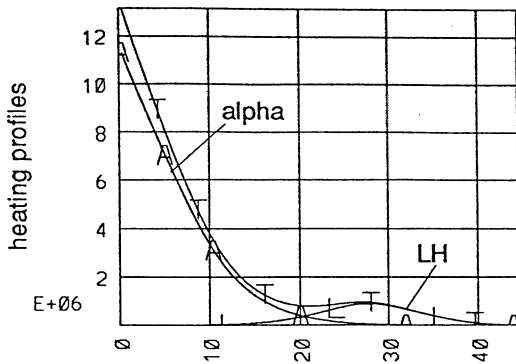
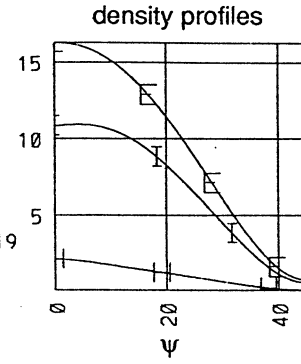
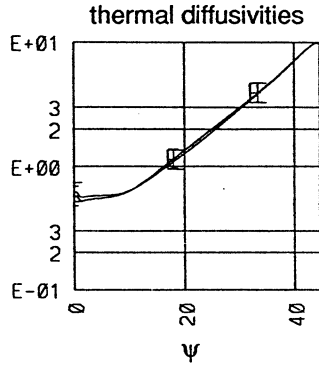
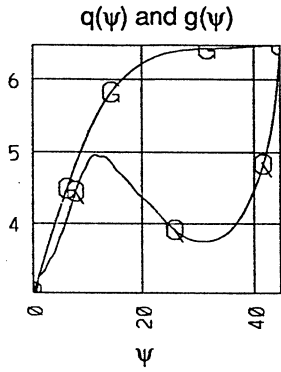
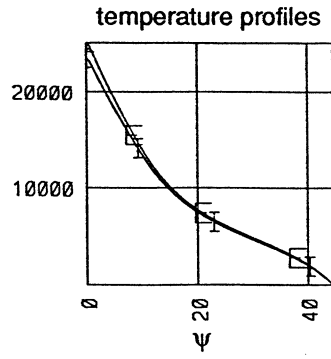
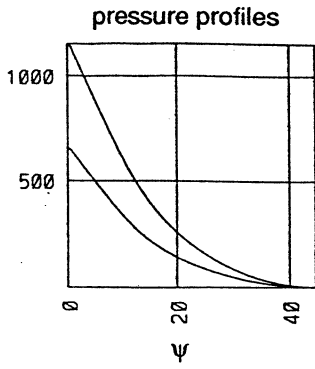


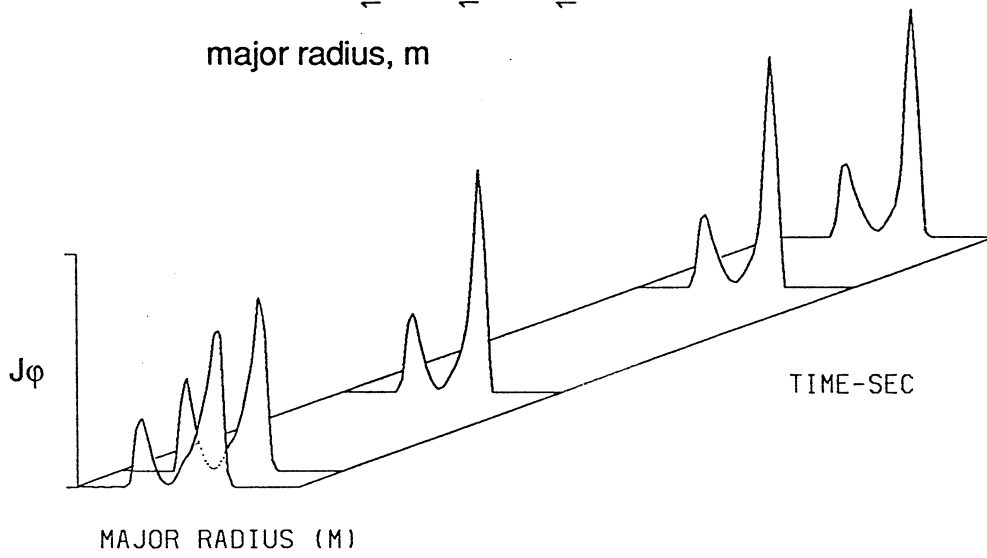
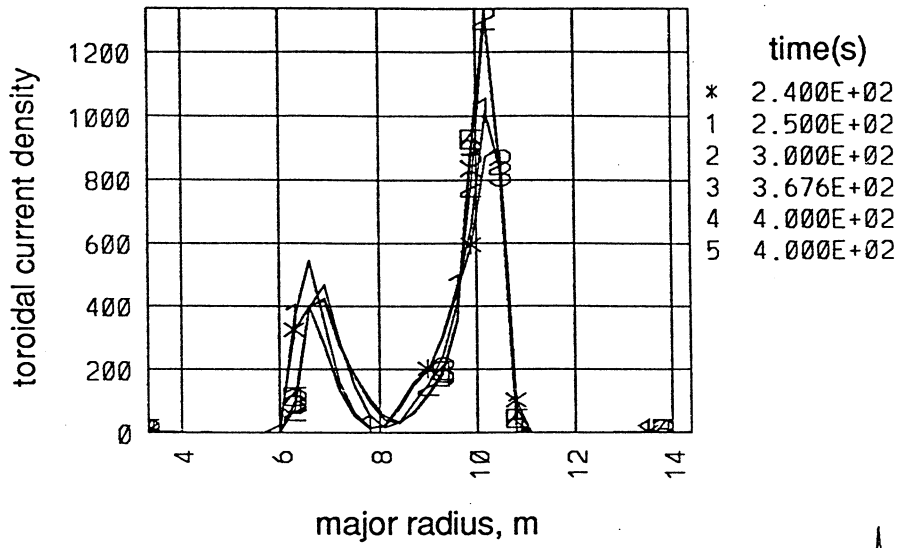
Simulation of ITER Reversed Shear Plasma Transition to High- β – High I_p , $t=240-400$ s

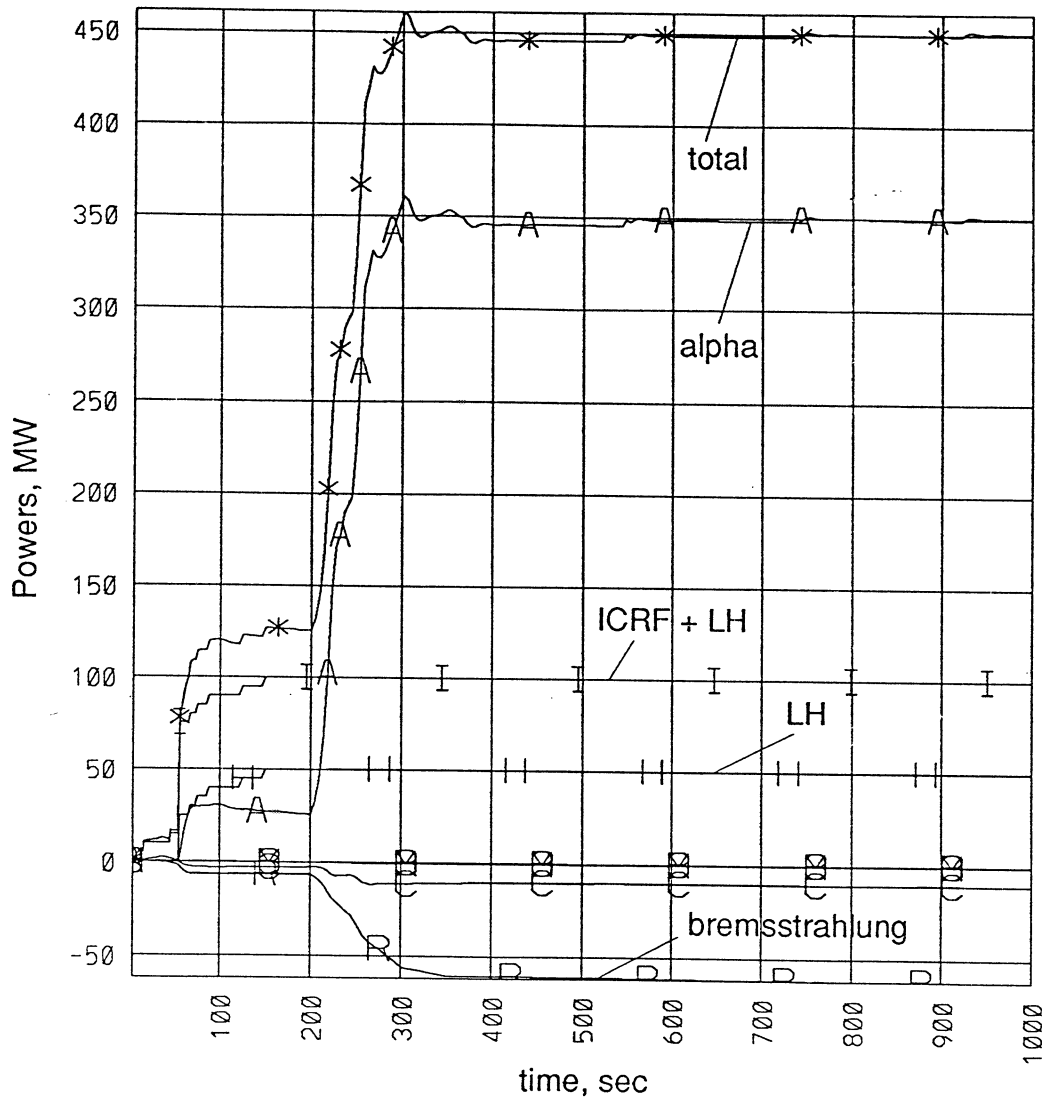
- in this phase the density is increased and the formation of an internal transport barrier is assumed, leading to a more peaked density profile
- strong fusion heating follows
- the electron density increases to $1.6 \times 10^{20} / \text{m}^3$
- β_N increases to 3.5
- the bootstrap current increases to 10 MA, while the LH current remains at 2 MA
- the plasma becomes even more strongly shaped with $\kappa=2.0$ and $\delta=0.5$
- the peak ion and electron temperatures reach 25 keV

plasma boundaries from $t=240.0$ to 400.0 seconds







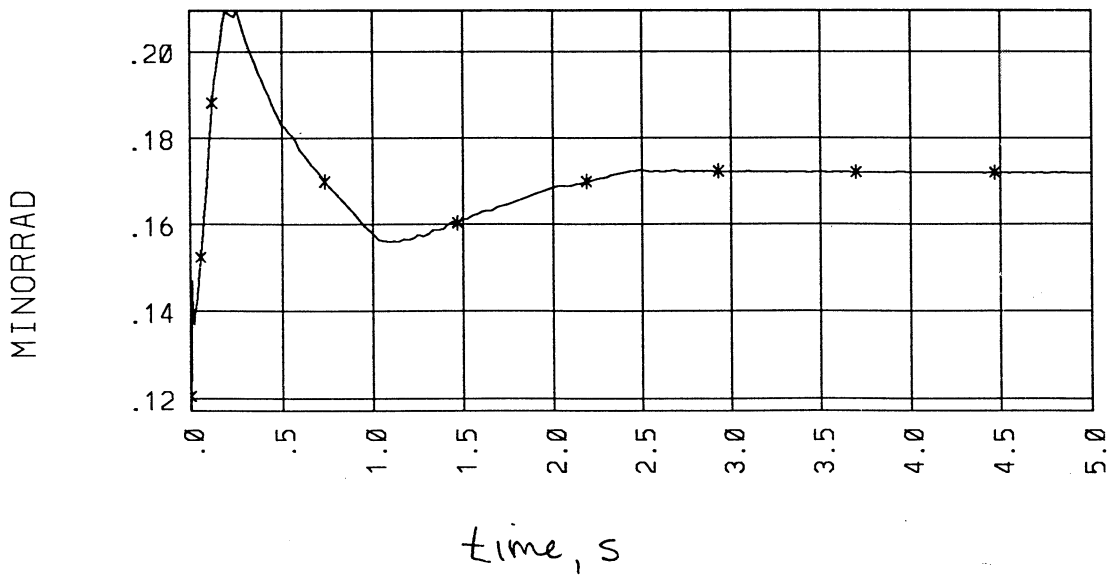
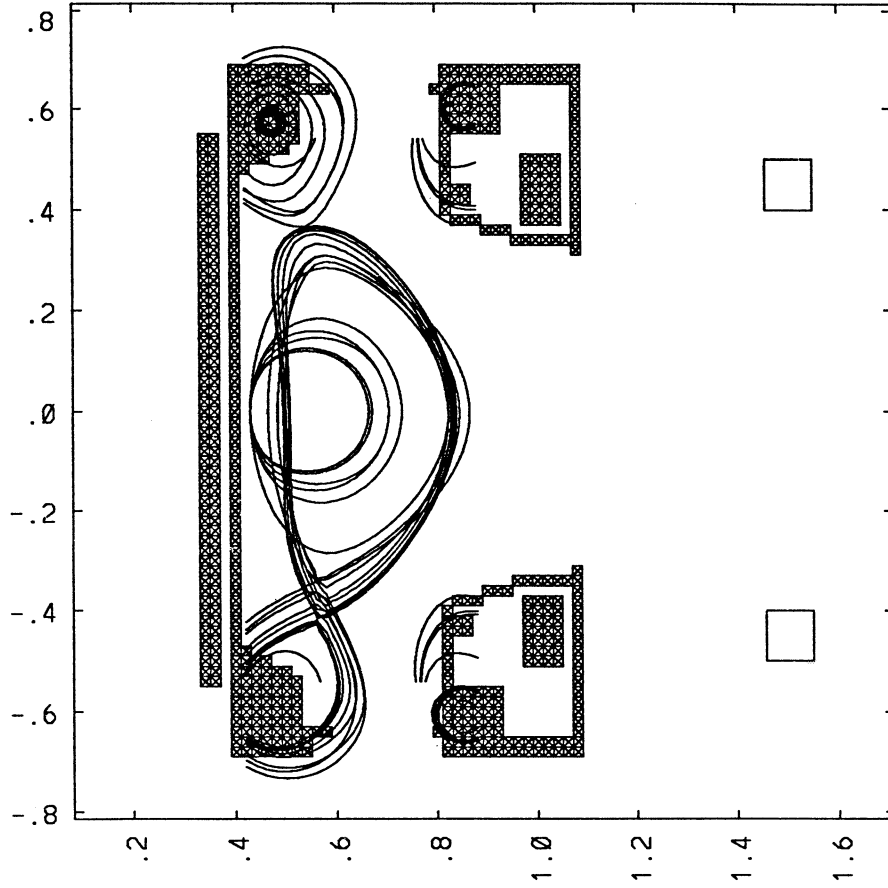


Simulation of CMOD Reversed Shear Evolution

- using 3 MW of LHCD and 2.5 MW of ICRH, at $B_T=4.3-4.5$ T
- with minimization of inductive current and sufficient CD/heating power we have obtained a nearly 100% non-inductive state in CMOD
- amount of CD/heating required depends primarily on the amount of bootstrap current obtained, which is uncertain
- we have used prescribed LH current deposition profiles based on CD calculations from Bonoli
- future work includes
 - improve PF coil current waveforms
 - use LSC for CD calculations in TSC
 - get better alignment with CMOD parameters

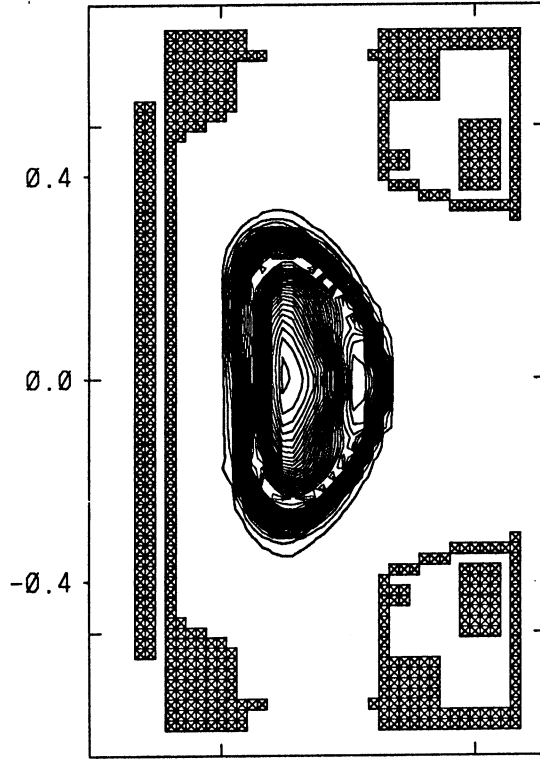
CMOD Simulation with TSC

plasma boundaries $t=0 \rightarrow 5s$

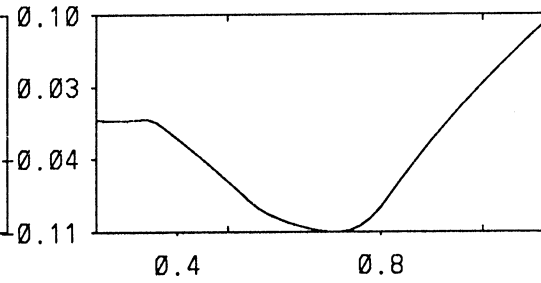
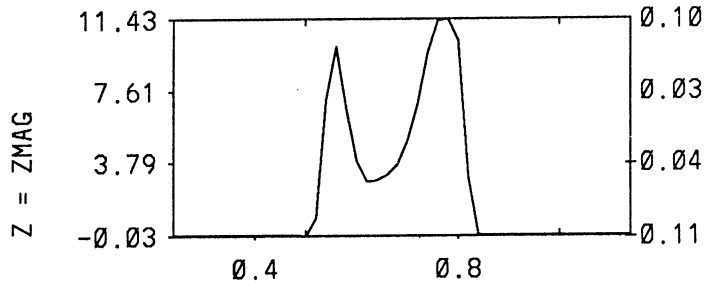
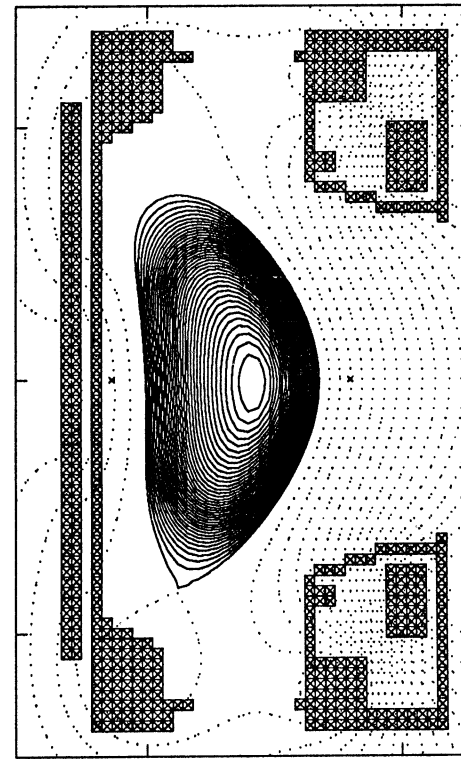


CMOD Simulation with TSC

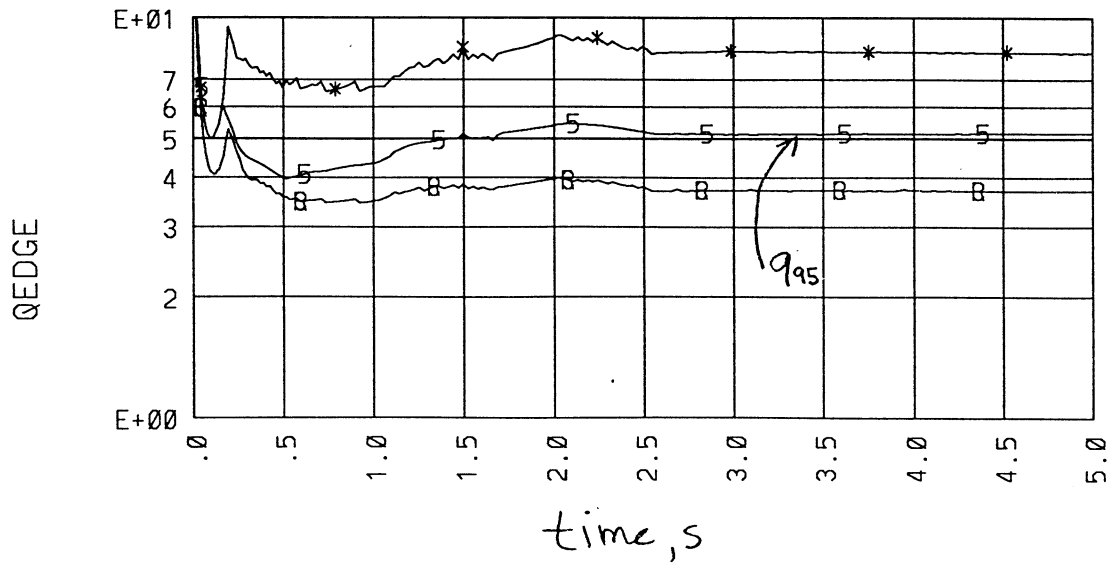
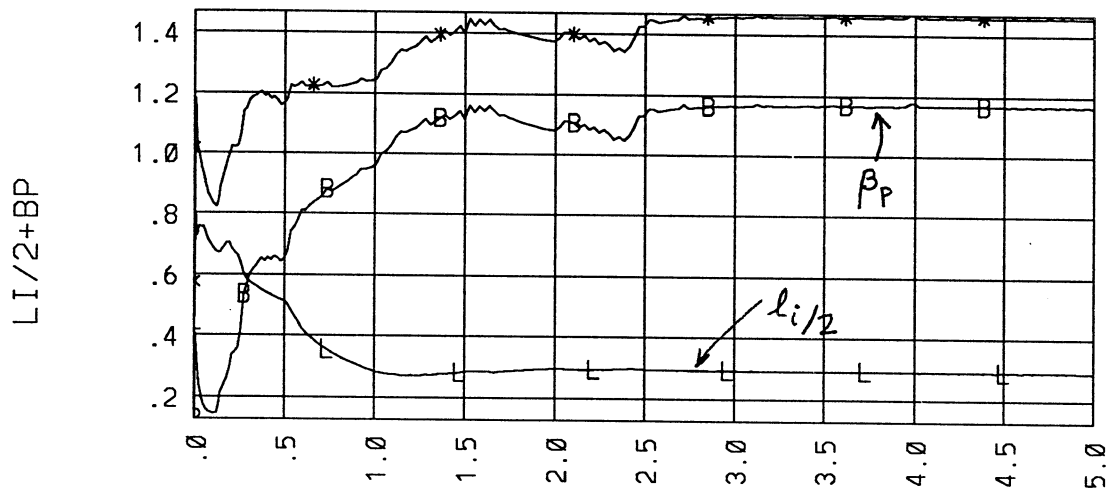
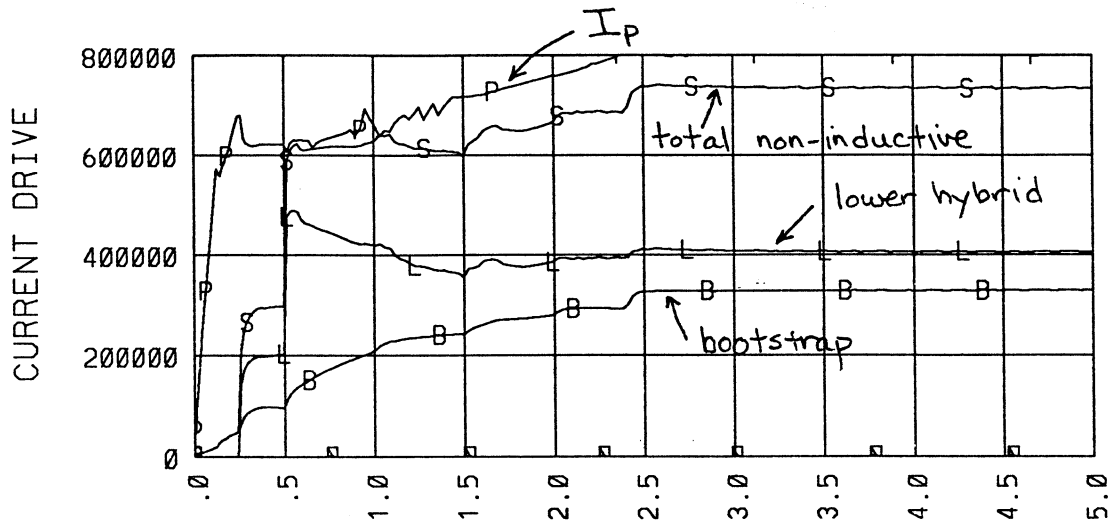
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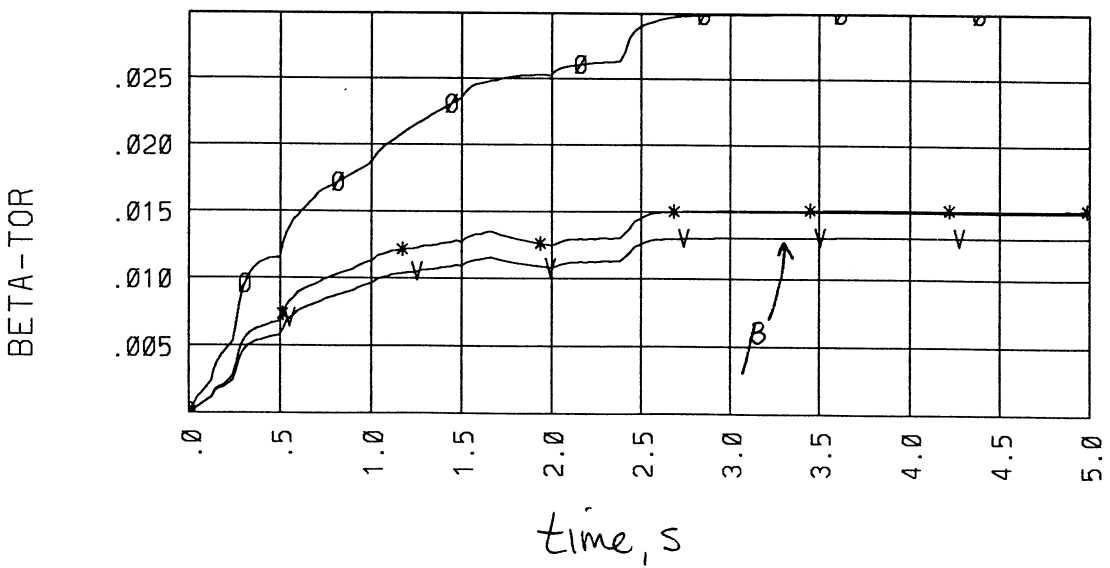
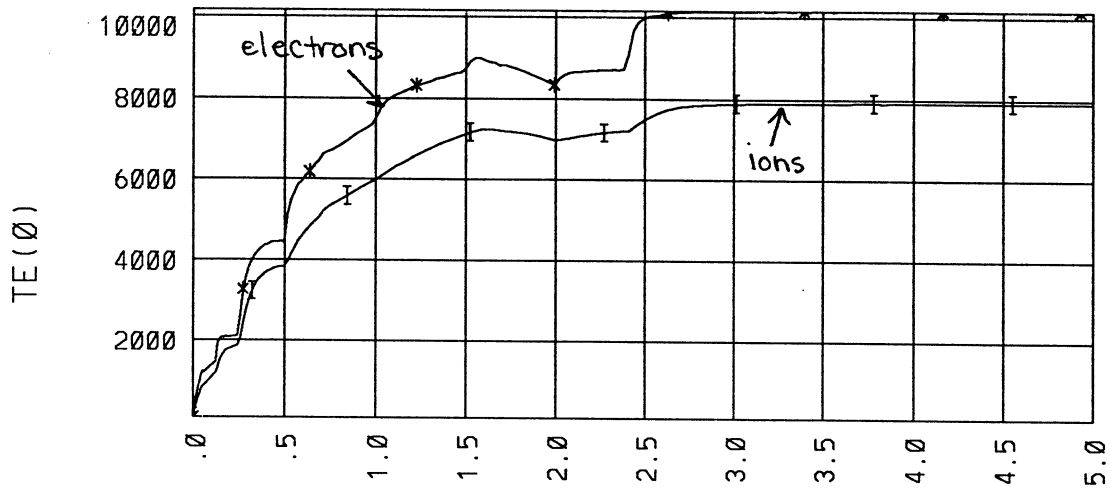
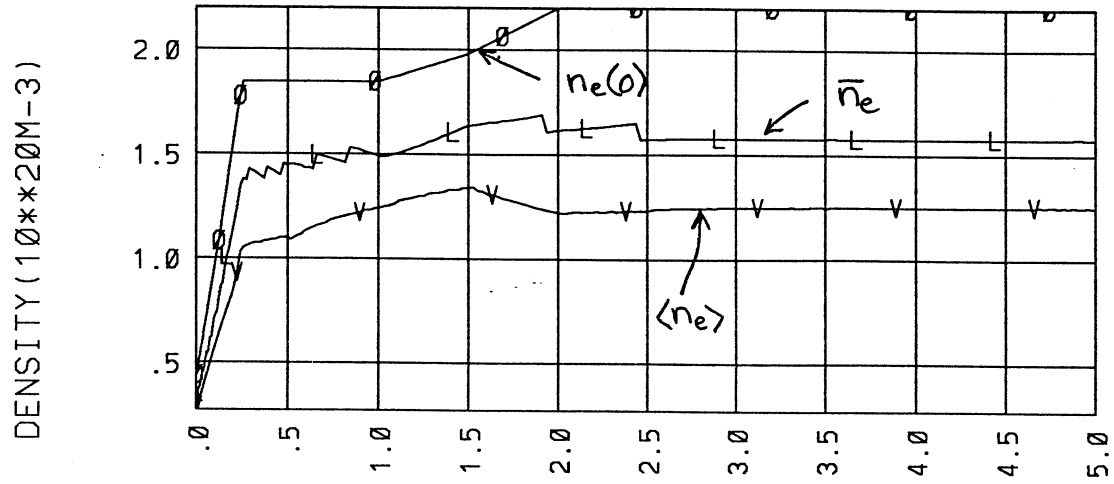
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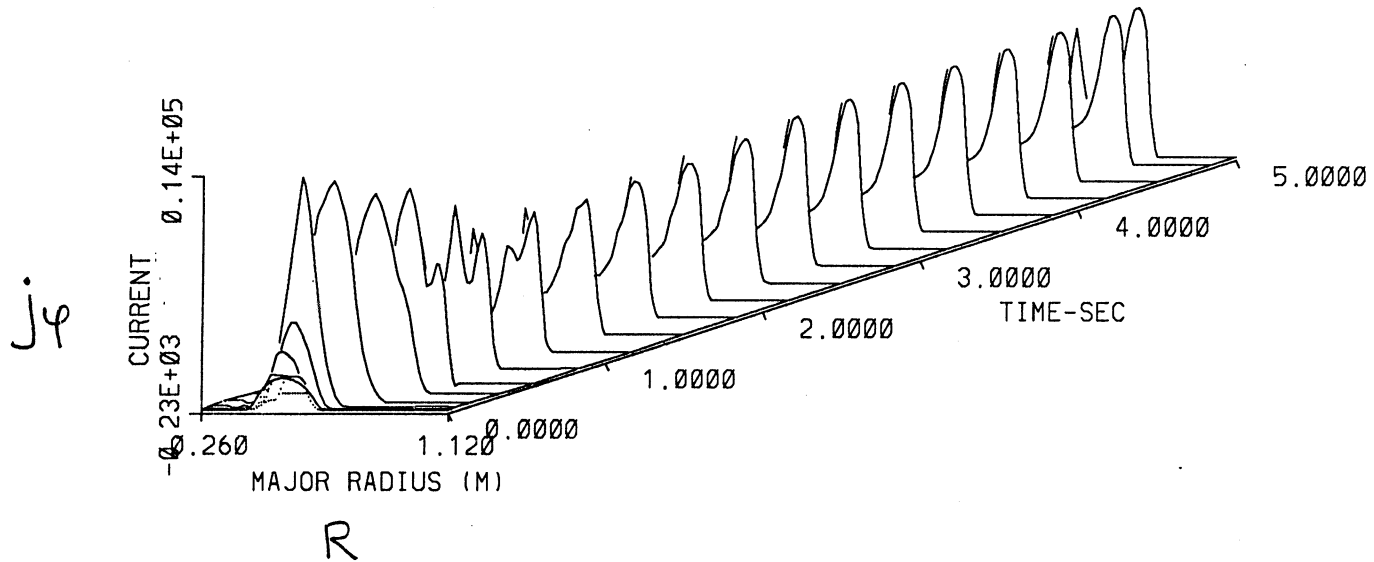
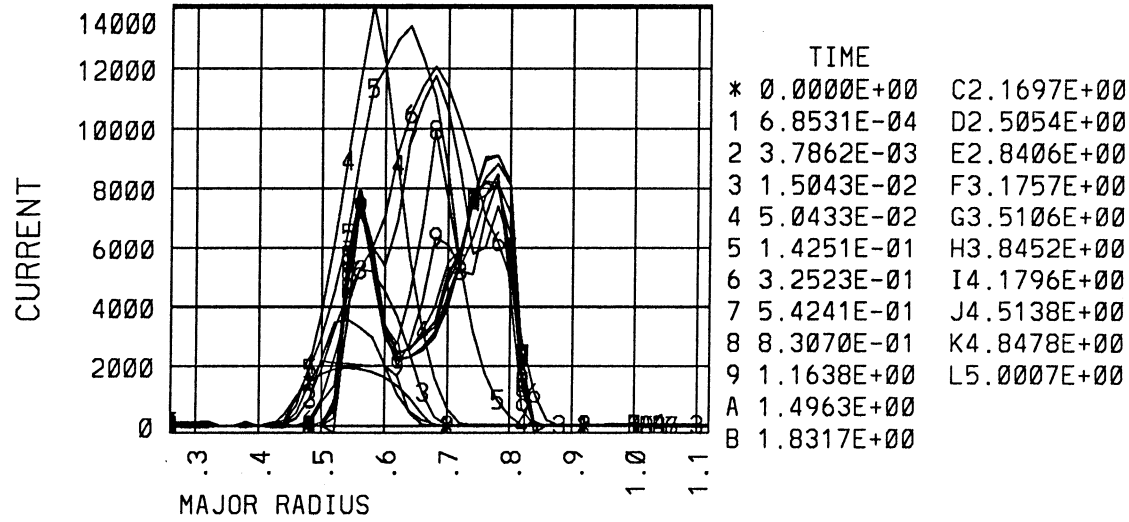
CMOD Simulation with TSC



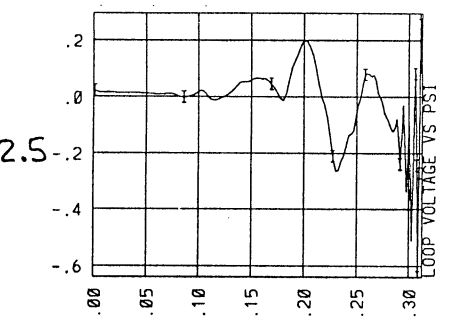
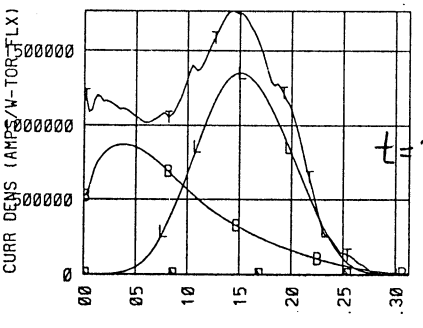
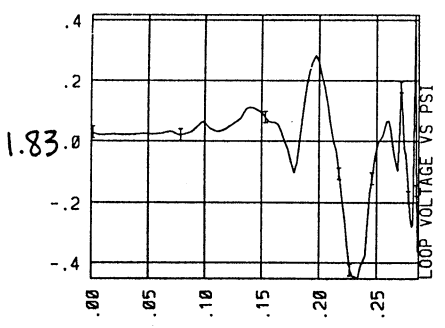
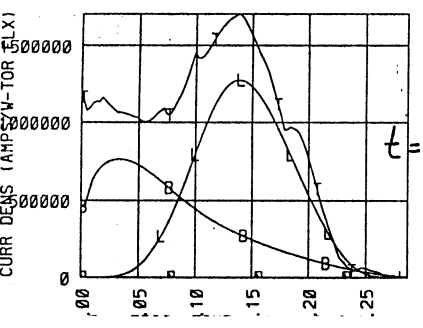
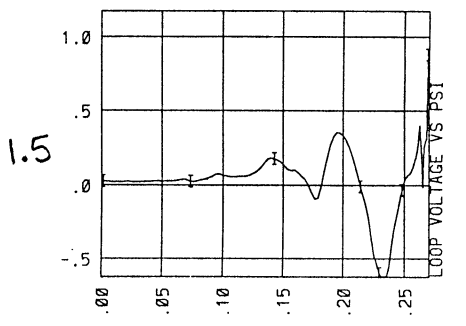
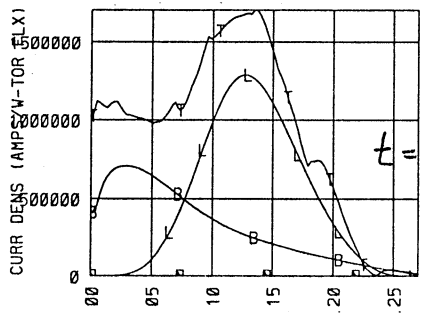
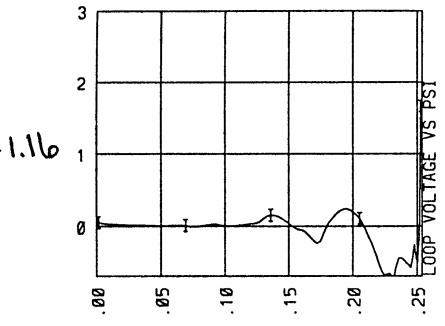
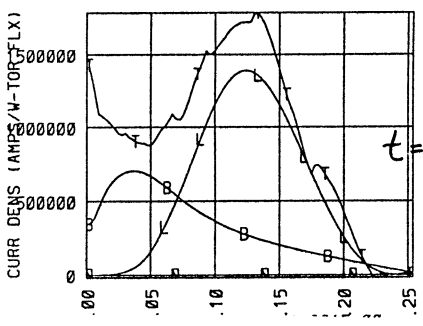
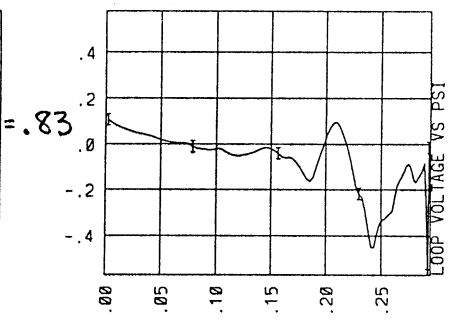
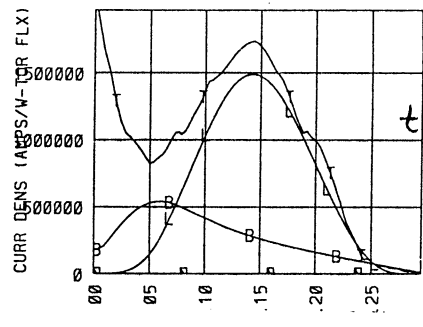
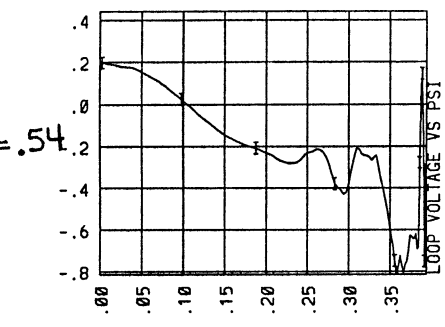
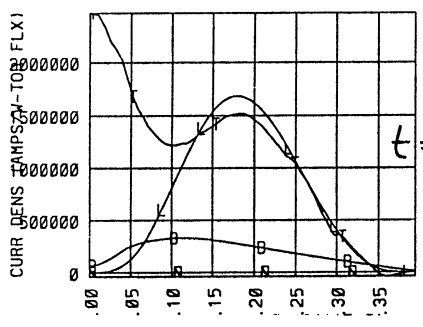
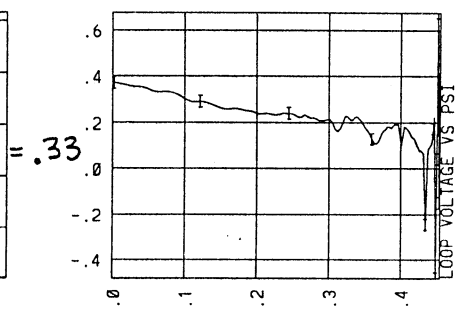
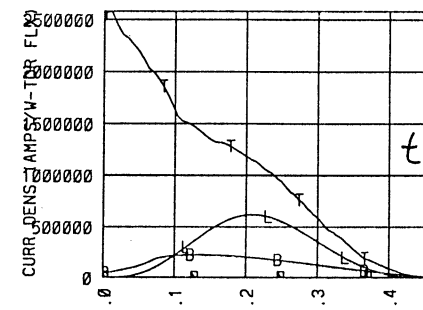
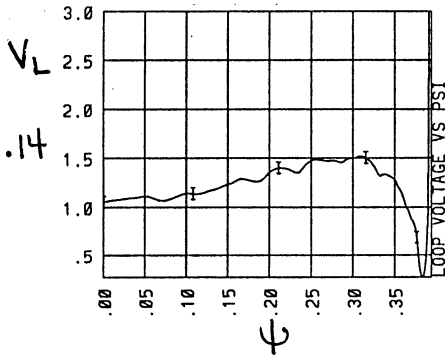
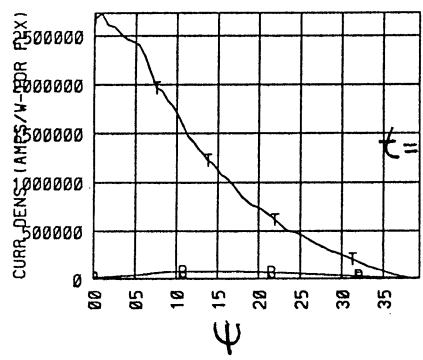
CMOD Simulation with TSC

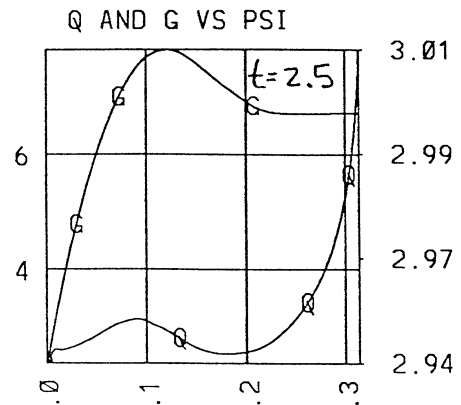
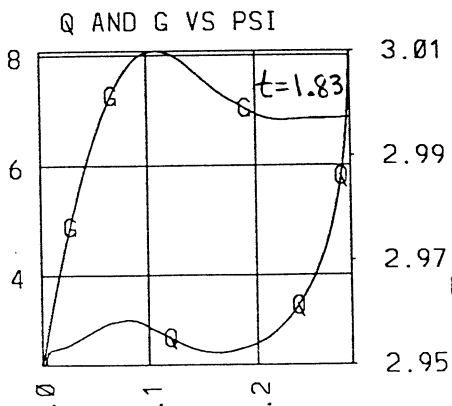
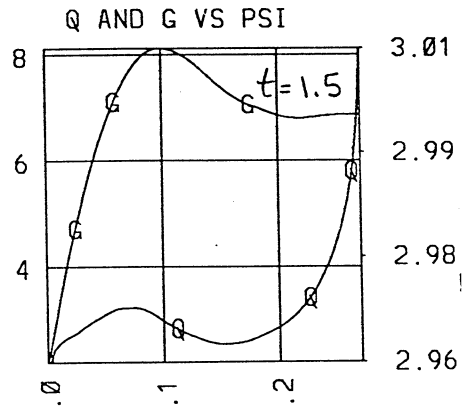
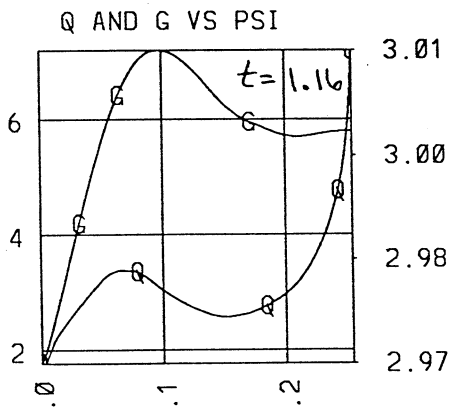
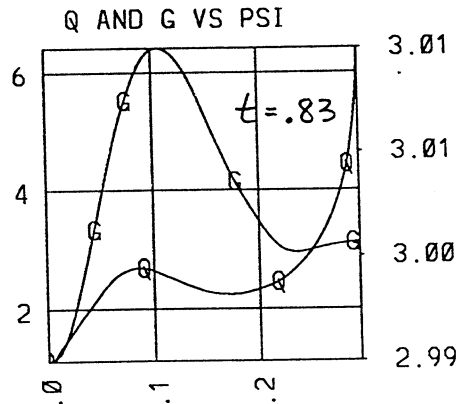
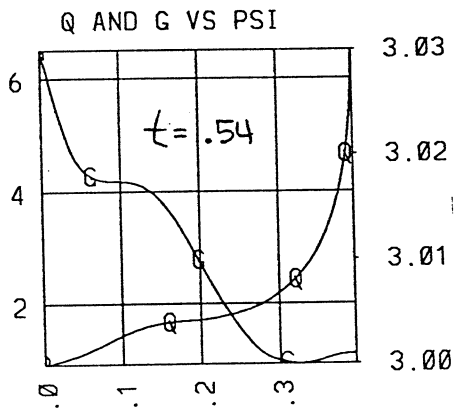
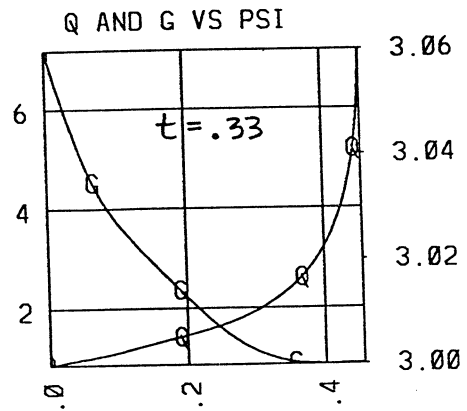
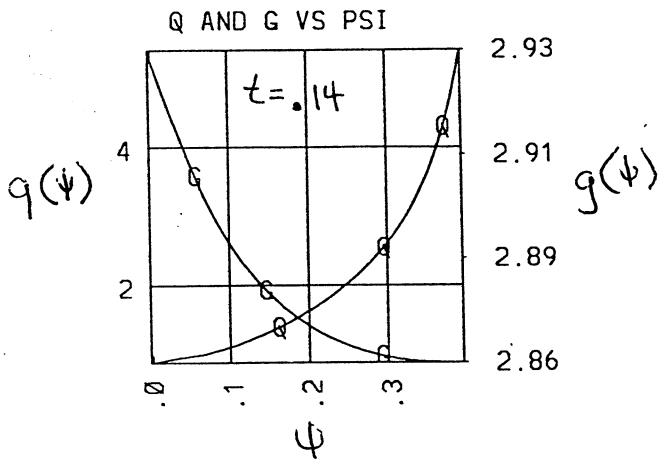


CMOD Simulation with TSC



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CMOD Simulation with TSC

