Predicting Optimal 3d Coil Configurations

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Adjustable Error Field Correction Coils Are a Stable of Today's Tokamaks





We keep adding more and more of them





All These Coils Do More Than Just Correct Error Fields





Wade Et al., Nuclear Fusion 55 (2015). N.C. Logan/ Workshop on MHD Stability Control / 11-2018

² Kim, K., et al., Phys. Plasmas 19,(2012).

Many Combinations of Coils Are Available





Even the Plasma Response is Multi-modal





This Talk: How to Optimize a 5D Coil Phase Space for Desired Physics

NTV in the Edge

NTV Throughout







Review: Resonant Coupling



The ideal MHD resonant coupling "dominant mode" is the leading error field correction metric for ITER

 Linearity and speed of ideal MHD perturbed equilibria provides resonant coupling matrix



- SVD provides an ortho-normal basis describing the entire space of Φ_x that drives resonant current
 - When 1st singular value >> 2nd, we say V^T₁ is a "dominant mode"
- The "overlap"¹ or "coupling" is a measure of how efficiently a given external field drives this dominant mode



Recent multi-modal work has shown it is possible to separate the edge and core resonances

- LM disruptions & ELM suppression both governed by resonant field penetration
- Use same metric, just separate core & edge

 To date, focus on brute-force mapping





Latest Addition: NTV Torque



Generalized Perturbed Equilibrium Code (GPEC) Provides a Matrix Formulation for the NTV Torque Similar to MHD Stability

 Including an anisotropic pressure tensor in DCON-like perturbed force balance produces self consistent 3D equilibria and torques

$$\delta W = -\frac{1}{2} \int dx^{3} \xi \cdot (\delta J \times B + J \times \delta B + \nabla \delta P + \nabla \cdot \Pi)$$

F = 0 = $\delta J \times B + J \times \delta B + \nabla \delta P + \nabla \cdot \Pi$

Eigenvectors describe space of force free 3D states Contains NTV physics, collisionality regimes, etc.



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• Torque, like energy, is reduced to boundary term

$$\delta W = \mathbf{\Phi}^{\dagger} \cdot \operatorname{Re}(\mathbf{W}) \cdot \mathbf{\Phi}$$
$$T = \mathbf{\Phi}^{\dagger} \cdot \operatorname{Im}(2n\mathbf{W}) \cdot \mathbf{\Phi}$$

• We use plasma response physics to connect this to any external field

$$\mathbf{T} = \mathbf{\Phi}_{\mathbf{x}} \cdot \mathbf{T} \cdot \mathbf{\Phi}_{\mathbf{x}}$$



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Matrix Formulation Insights & Multi-modal Optimization



- Matrix decomposition quickly provides "optimal" spectrum
 - Ideal stability: $\delta W = \Phi^{\dagger} \cdot W \cdot \Phi$
 - 1st eigenmode = "least stable mode"
 - Ideal resonant coupling: $\Phi_r = \mathbf{C} \cdot \Phi_x$ 1st SVD is EFC "dominant mode"
 - Kinetic torque: $T_{NTV} = \mathbf{\Phi}^{\dagger} \cdot \mathbf{T} \cdot \mathbf{\Phi}$
 - 1st eigenmode is maximum NTV

Eigendecompose: $T = U \cdot \Lambda \cdot U^{-1}$

- u_1 spectrum produces maximum λ_1 Nm/G²
- Multiple $\lambda_i \sim \lambda_1$ enables profile manipulation



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Torque Eigenmodes Provide Insight into Decoupled Transport



- Coil combinations orthogonal to the "dominant" (resonant) mode cause little pumpout but still cause momentum transport
- Torque matrix reveals low m≲nq of coils couples well to 2nd and 3rd torque eigenmodes
- Would need to null out all 15>m>4 to avoid breaking

¹ Paz-Soldan, C., et al., Nuclear Fusion 55, 083012 (2015). N.C. Logan/ APS DPP / 11-2018

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- For DIII-D, we get a 3x3 complex Coil NTV matrix
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The Optimal Field Spectrum Changes if Localized Core or Edge NTV is Desired for Rotation Profile Control

- There is a profile of these matrices, $T_{coil}(\psi)$, describing the torque within ψ
- The most localized NTV is simply u_1 of $T^{-1}_{coil}(\psi=1) [T_{coil}(\psi_1) T_{coil}(\psi_2)]$





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Both Core and Edge Resonant Fields Create Broad Rotation Damping, with Similar Final Results Across the Core Profile



DIII-D NATIONAL FUSION FACILITY

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Edge Resonant Fields Changed the Local Edge Rotation Gradient, Broad Core Fields Do Not





Edge Resonant Fields Changed the Local ExB shear even more - combines with ambipolar transport





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Takeaways and Talking Points



We have validated metrics for EFC, ELM control and rotation profile control

- Matrix formalism enable us to calculate optimal spectra for each
 - Useful in single dominant mode approximations
 - Provides targets for future coil designs

• Can be related directly to existing coil sets

- Provides direct optimization of coil currents*
- Brute force maps can be used to investigate phase space features







Next steps



• Projection of thresholds to ITER

- LM threshold database exists... can we do the same for ELMs?
- Unaccounted for 3D field implications
 - Particle Pumpout related to resonance in outer pedestal?
- Multiple, continuous criteria
 - How much edge RMP would you trade for every Nm of braking?



Backup



Let's Be Clear About "Single-mode" vs "Multi-modal"

- Single-mode: A single, coherent radial and poloidal structure dominates 3D perturbations (tuning fork)
 - Implication: EFC only needs to correct 1 poloidal spectrum for stability and transport
- "Mode" does not refer to single m/n
 - Everything here is single n (decoupled toroidally)
- Multi-mode: The radial and poloidal structure of 3D perturbations depends on the driving field
 - Implication: EFC must better match EF, separation of resonant and nonresonant effects, seperation of radial impact





The Multi-modal Plasma Response in DIII-D





These Optimized Spectra for n=2 NTV in the DIII-D ITER Similar Shape Scenario Have Clear Distinctions





Optimized Spectra for n=2 NTV in the DIII-D ITER Similar Shape Scenario Look Promising



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The 10's Saw Great Advancements In The Details Of Ntv Theory That Hint At "Resonant" Nature

- Many new regimes described by both reduced and numerical models¹
- "Super banana" plateau \rightarrow NTV largest when $\omega_{_{E}}{\sim}0$
- This means large spikes at E_r=0 near the top of H-mode pedestals
 - Requires accurate kinetic equilibrium reconstruction!



¹ Shaing, Ida, & Sabbagh, Nucl. Fusion 55, (2015).
² Callen, Nucl. Fusion 51, (2011).
³ Wang, et al., Phys. Plasmas 21, (2014) an/ Workshop on MHD Stability Control / 11-2018



Edge Kink

Time Scale of Edge Evolution Challenges Diagnostic Capabilities for Direct Measurement of Localized Torque

- Momentum confinement time $\tau_{\phi} \sim 80$ ms
- Assuming T = v υ_φRmn, predicted local damping rate spikes to ~kHz
- Damping → measurement rate ≪ coil ramp rate





Torque Matrix Provides Immediate Insight into How Best to Induce Nonambipolar Transport in the Core or Edge



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Exp. Revealed Obvious Spectral Dependencies: Pumpout vs Rotation



