

# Quasi-symmetric error field correction in tokamaks

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# Overview

- Background: Non-axisymmetric error field control (EFC) in tokamaks
  - Recent progress on resonant EFC
  - Issue with residual magnetic perturbations after resonant EFC
- Modeling: EFC optimization towards quasi-symmetric (QS) residuals
  - Minimization of variation in field strength and 3D neoclassical transport
  - Optimization via torque response matrix
- Experiment: Testing quasi-symmetric magnetic perturbations (QSMP)
  - In comparison with RMPs and NRMPs in KSTAR and DIII-D
  - Safety of QSMPs during transient phase
- Summary and outlook

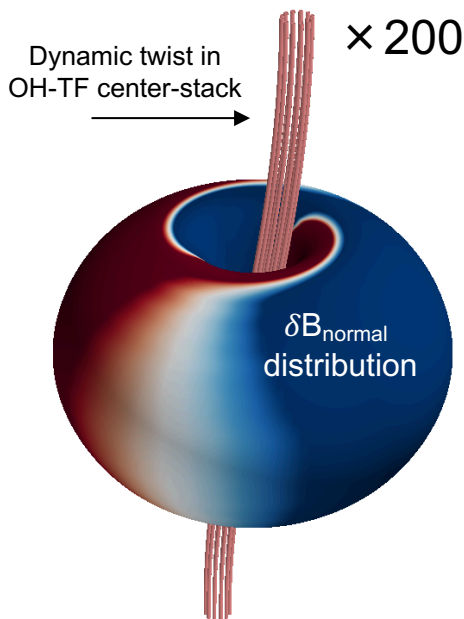
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# A small non-axisymmetric magnetic field can greatly change tokamak performance and thus must be under control

- A tokamak has always intrinsic non-axisymmetric (3D) error fields (EF)
  - Due to imperfect magnets and components
- A 3D field can also be introduced on purpose
  - Mostly for instability control, as highlighted by "RMP ELM control" in tokamaks
- In either case, a 3D field as small as  $\delta B/B=10^{-3}\sim 10^{-4}$  can greatly degrade or even disrupt tokamak plasmas, if not properly controlled or judiciously used
- Any dangerous or unnecessary 3D field components must be compensated

## NSTX Example Error fields driving locked modes

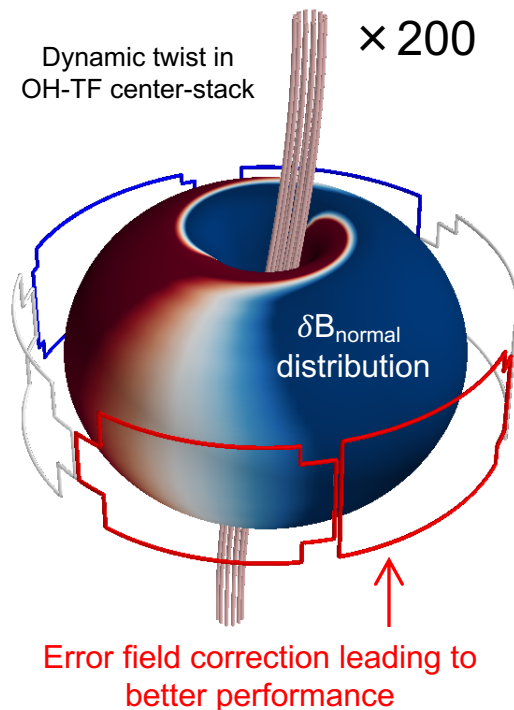


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- Any dangerous or unnecessary 3D field components must be compensated
  - Error Field Correction (EFC)

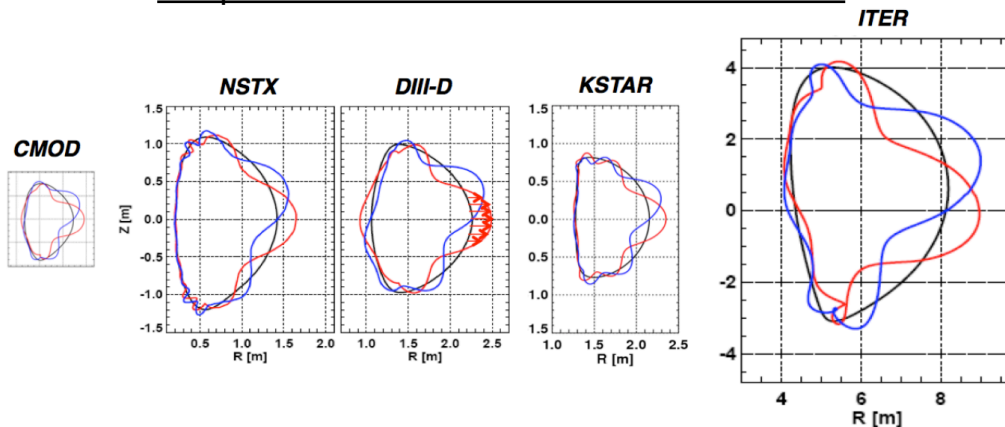
## NSTX Example

Error fields driving locked modes



# Recent progress on plasma response and MHDs is offering a reliable leading-order EFC scheme

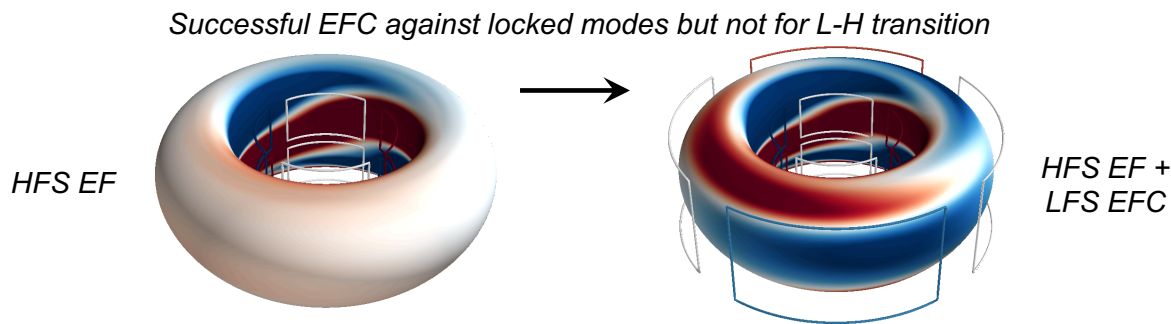
## Shape of dominant resonant field distribution



- Ideal MHD clearly shows which 3D field is most resonant with tokamak plasmas and thus must be compensated if not necessary
  - Leading to a major change in EFC approach, via “resonant overlap” field
    - Extensively validated in tokamak devices including DIII-D [LanctotPOP10, Paz-SoldanPOP14]
- Present ITER EFC strategy: Reduce overlap with dominant resonant field below “EF penetration” threshold
  - Two-fluids MHDs then can offer prediction of EF penetration threshold in practice [TM1 (Hu), EPEC (Fitzpatrick)]
    - See N. Logan’s poster for resonant EFC summary

# Residual EFs may not be disruptive in stable operating conditions but shown to be still problematic transiently

- COMPASS studies with the high-field-side proxy-EF show
  - Locked modes could indeed be avoided by resonant EFC, but large non-resonant residual EFs could still be disruptive during L-H transition [MarkovicEPS2018]



- NSTX-U and DIII-D also showed that NTV rotational damping by residual EFs which can eventually cause instability issues [Paz-SoldanPOP14, ParkAPS18]

Needs a complementary EFC approach

for residual EFs which often have greater non-axisymmetry and create non-linear effects

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# Minimizing all the prominent residual EF effects reminisces optimization towards a quasi-symmetry

- Eliminating all static EF effects in guiding center plasmas is in principle achievable by:

$$\text{Variation in the field strength } |\delta \vec{B}|_{\text{particles}} \rightarrow 0 \quad [\text{NurenbergPLA88, BoozerPPCF95}]$$

- Ideally, there is a linear path to perturb a tokamak while holding this condition:
- However, it is the force balance in plasma that dictates the  $\vec{\xi}$  profiles

$$|\delta \vec{B}| \sim \hat{b} \cdot \left( \underbrace{\vec{\nabla} \times (\vec{\xi} \times \vec{B}_0)}_{\substack{\text{Eulerian} \\ \text{changes in a fixed space}}} + \underbrace{\vec{\xi} \cdot \vec{\nabla} B_0}_{\substack{\text{Lagrangian} \\ \text{changes with field lines}}} \right) \rightarrow 0 \quad \longleftrightarrow \quad \delta \vec{F}[\vec{\xi}] = 0$$

- These two are NOT compatible in general

as well known [Garron&BoozerPFB9]  
from more general 3D geometry

Nonetheless, quasi-symmetric optimization can be performed in average

# Self-consistent perturbed equilibria with neoclassical transport offers a unique QS optimizing scheme, via torque response matrix

- Perturbed equilibria with non-adiabatic pressure (including 3D coils):

$$\delta\vec{F}[\vec{\xi}] = \delta\vec{F}_{ideal}[\vec{\xi}] - \vec{\nabla} \cdot \Pi[\vec{\xi}] = 0$$

- Neoclassical torque is also given by integrating:

$$\tau_{\varphi} = Im \left[ n \int_{plasma} dx^3 (\vec{\xi} \cdot \delta\vec{F}[\vec{\xi}]) \right]$$

- Torque minimization leads minimized 3D neoclassical particle, momentum, heat transport, although its momentum part (called NTV) is mostly pronounced in tokamaks

$$\tau_{\varphi} \propto \Gamma_{NTV} \propto Q_{NA} \sim 0$$

- Full solutions provide torque response matrices to given 3D fields or coils

$$\begin{aligned} \tau_{\varphi}(\psi) &= (\text{Fourier modes})^+ \cdot \mathbf{T}(\psi) \cdot (\text{Fourier modes}) \\ &= (\text{Coil currents})^+ \cdot \mathbf{T}_c(\psi) \cdot (\text{Coil currents}) \end{aligned}$$

- Method above has been implemented in general perturbed equilibrium code (GPEC) which has been used as a primary tool to design QSMP configurations

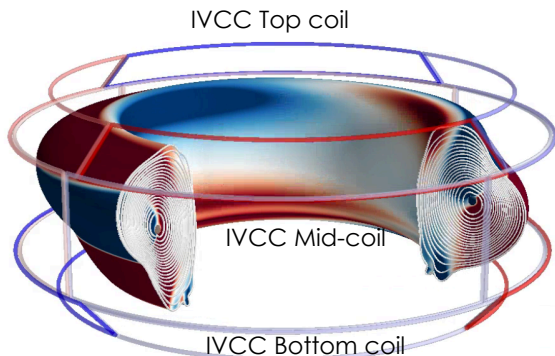
# Torque response matrix contains all the information of neoclassical torque that a tokamak can drive with available coils

- All possible neoclassical torque that a tokamak (e.g. KSTAR) can drive using their 3 rows of coils are given by
  - 3x3 matrix, per each  $n$ , per a target equilibrium and its kinetic profiles

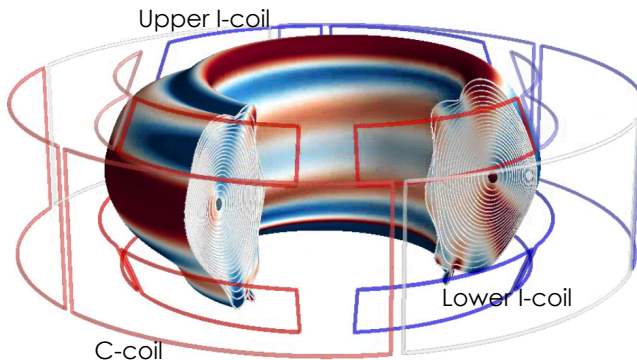
$$\tau(\psi) = (I_T e^{-i\phi_T} \quad I_M e^{-i\phi_M} \quad I_B e^{-i\phi_B}) \cdot \begin{pmatrix} T_{TT}(\psi) & T_{TM}(\psi) & T_{TB}(\psi) \\ T_{MT}(\psi) & T_{MM}(\psi) & T_{MB}(\psi) \\ T_{BT}(\psi) & T_{BM}(\psi) & T_{BB}(\psi) \end{pmatrix} \cdot \begin{pmatrix} I_T e^{i\phi_T} \\ I_M e^{i\phi_M} \\ I_B e^{i\phi_B} \end{pmatrix}$$

- Its eigenvector for the minimum eigenvalue of the torque-coil response matrix: The best possible quasi-symmetric magnetic perturbation (QSMP) in a tokamak

*KSTAR  $n=1$  QSMP*

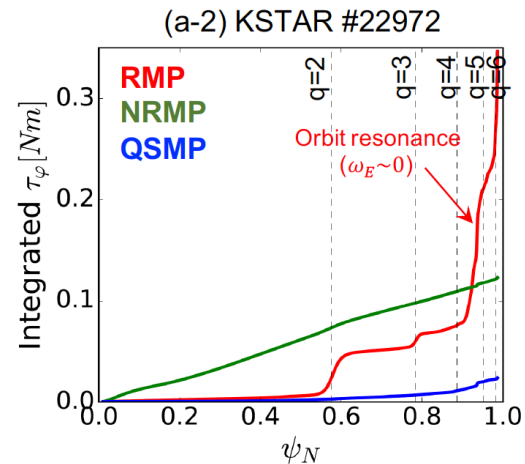
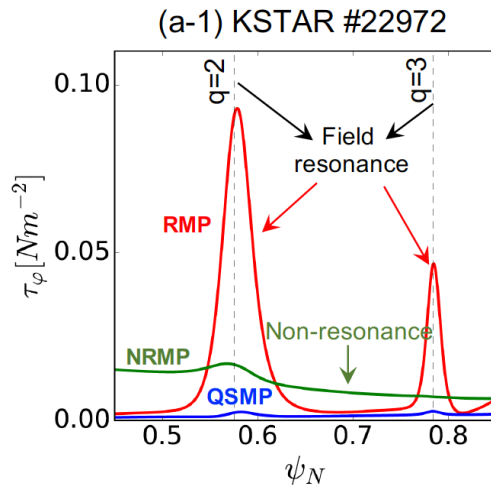


*DIII-D  $n=1$  QSMP*



# QSMP is clearly contrasted to two other categories of small 3D fields in tokamaks

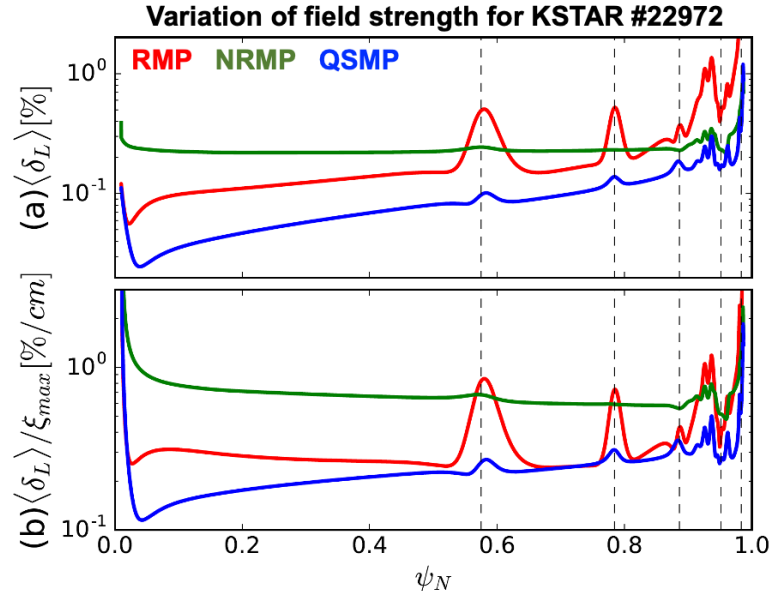
- **RMP** creates strong resonant response (at the rational surfaces)
- **NRMP** can drive substantial non-resonant NTV, but without resonant response
- **QSMP** suppressed both resonant and non-resonant response while maintaining the same power norm of field amplitudes or currents



# QSMP optimized by GPEC indeed minimizes variation in the field strength at best upon constrained by force balance and torque

- GPEC finds the best possible QSMP by minimizing total torque, within force balance
- Resulting in minimization of plasma response and variation in the field strength
- Resulting in optimization of displacement spectrum

$$\delta_L \equiv \frac{|\delta B|}{B_0} = \hat{b} \cdot \vec{\nabla} \vec{\xi} \cdot \hat{b} - \vec{\nabla} \cdot \vec{\xi} \sim 0$$

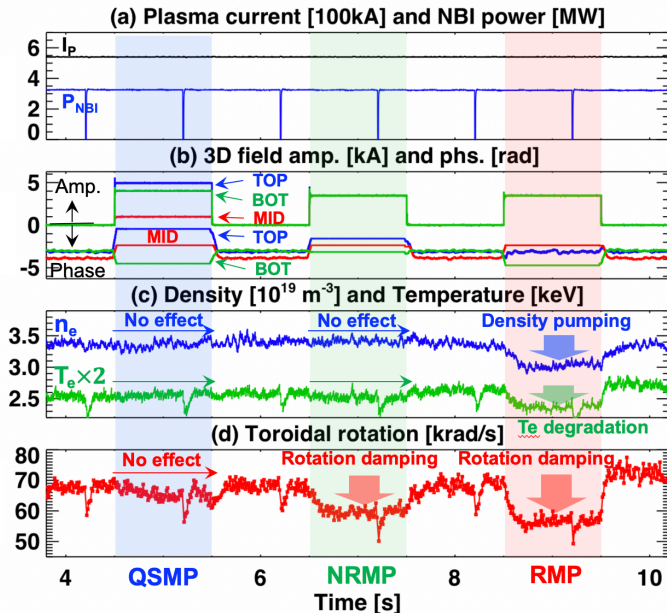


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# QSMP designed and tested in KSTAR indeed did not bring any meaningful effects despite the large amplitudes

- **RMP** caused density pumping, confinement degradation, and rotational damping
  - Could suppress ELMs if further optimized
- **NRMP** induced rotational damping only (without density pump-out)
- **QSMP** did not show any degradation, even with the maximum currents applied (10kAt)

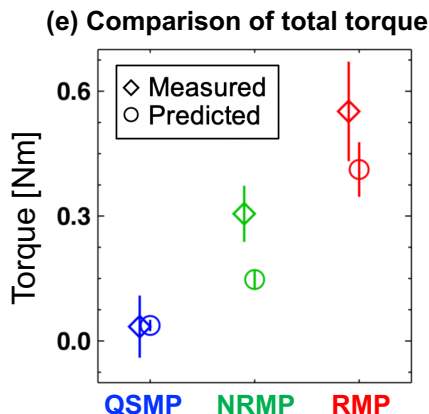
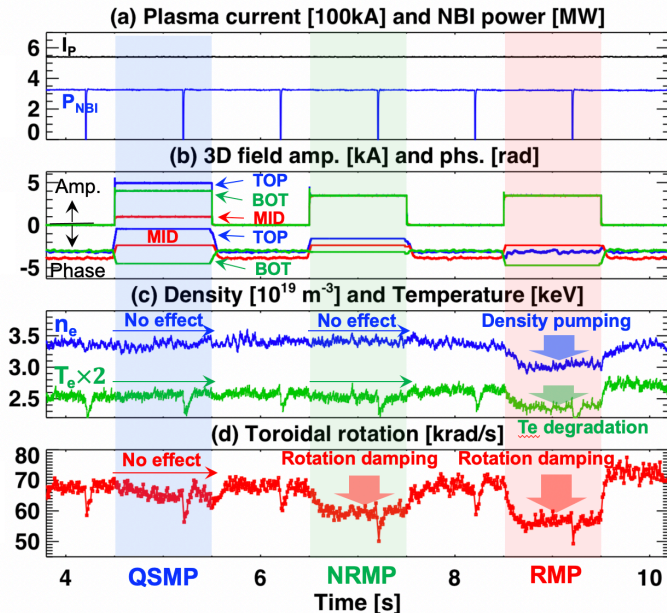


$P_{\text{NBI}} \sim 3\text{MW}$ ,  
 $T_{\text{NBI}} \sim 2.9\text{Nm}$ ,  
 $I_p = 0.5\text{MA}$ ,  $B_T = 1.8\text{T}$ ,  
 $\beta_N \sim 1.8$ ,  $q_{95} \sim 5$ ,  
 $n_e \sim 3.4e19\text{m}^{-3}$ ,  
 $T_i(\text{Core}) \sim 2.2\text{keV}$ ,  
 $T_e(\text{Core}) \sim 2.3\text{keV}$ ,  
 $\omega_\phi \sim 100\text{krad/s}$

[S. M. Yang, 2019 KSTAR Campaign]

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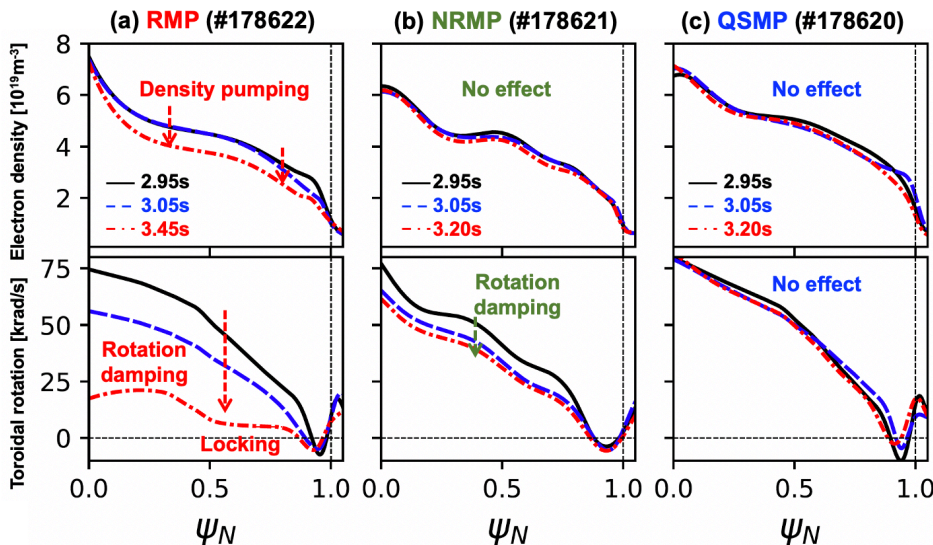


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# QSMP did not induce any visible effects in DIII-D either despite strong 3D response expected otherwise

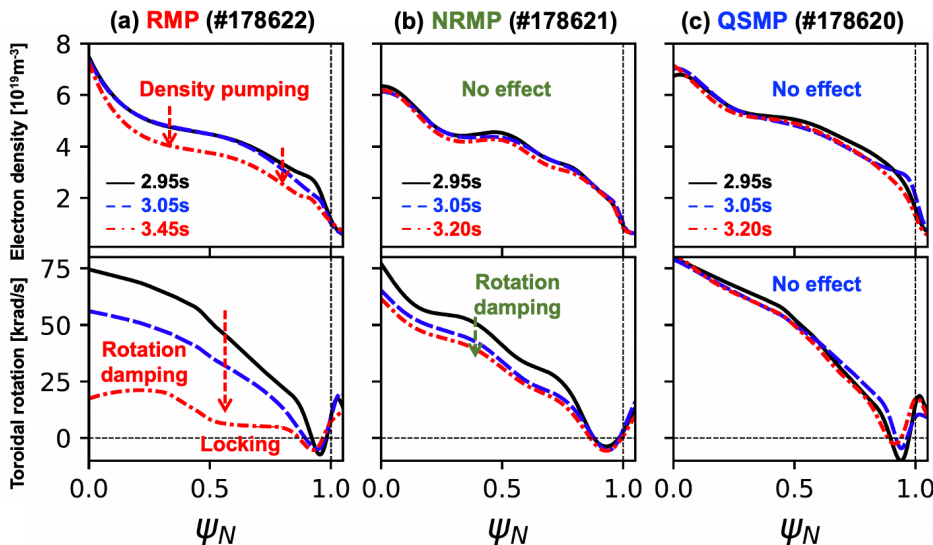
- **RMP** caused density pumping, confinement degradation, and rotational damping
  - Eventually caused a locking due to strong resonant response
- **NRMP** induced rotational damping only
- **QSMP** did not show any degradation, despite maximum currents applied (5kAt)



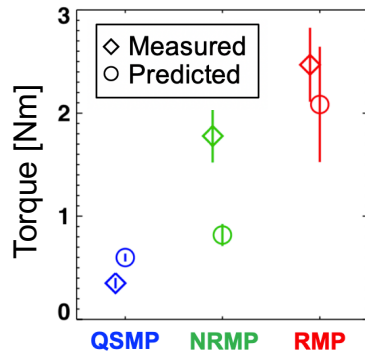
$P_{\text{NBI}} \sim 8.5 \text{ MW}$ ,  
 $T_{\text{NBI}} \sim 6.7 \text{ Nm}$ ,  
 $I_p = 1.2 \text{ MA}$ ,  
 $B_T = 1.8 \text{ T}$ ,  
 $\beta_N \sim 3.1$ ,  
 $q_{95} \sim 4.3$ ,  
 $n_e \sim 5.0 \times 10^{19} \text{ m}^{-3}$ ,  
 $T_i \sim 6.0 \text{ keV}$ ,  
 $T_e \sim 4.3 \text{ keV}$ ,  
 $\omega_\phi \sim 80 \text{ krad/s}$

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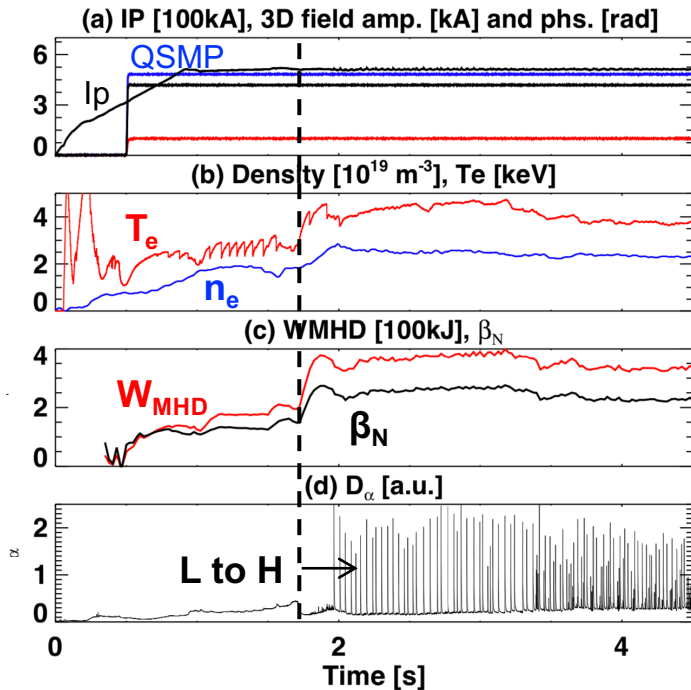


(d) Comparison of total torque



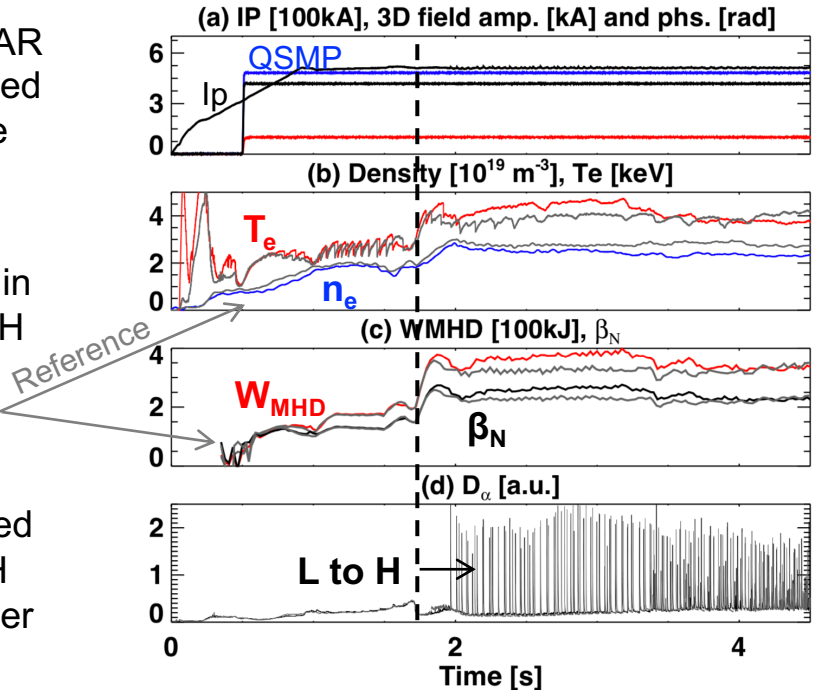
# QSMP remains also safe through early ramp-up and L-H transitions

- QSMP for a new 2020 KSTAR target is designed and applied during the ramp-up, with the maximum amplitude



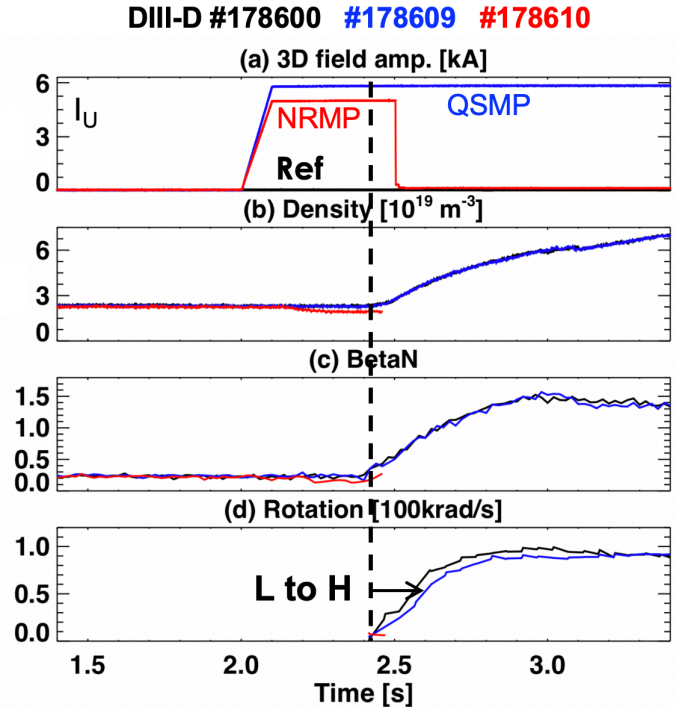
# QSMP remains also safe through early ramp-up and L-H transitions

- QSMP for a new 2020 KSTAR target is designed and applied during the ramp-up, with the maximum amplitude
- Did not leave any influence in the ramp-up and through L-H transition, compared to the reference without 3D fields
- QSMP plasma in fact showed better confinement after L-H transition which will be further investigated



# L-H transition with marginal power remained intact by QSMP, although disrupted by NRMP

- QSMP applied also to a marginal H-mode in DIII-D
- @ L-H:
  - $P_{\text{NBI}} \sim 1\text{MW}$ ,  $T_{\text{NBI}} \sim 0.83\text{Nm}$ ,
  - $I_{\text{p}} = 1.2\text{MA}$ ,  $B_{\text{T}} = 1.8\text{T}$ ,
  - $\beta_{\text{N}} = 0.24 \sim 1.5$ ,  $q_{95} \sim 4.0$ ,
  - $n_{\text{e}} \sim 2.2 \times 10^{19}\text{m}^{-3}$ ,  $T_{\text{e}} \sim 1.7\text{keV}$ ,  $\omega_{\phi} \sim 17\text{krad/s}$
- No impact by QSMP, although NRMP disrupted plasma through L-H
  - As observed in COMPASS
  - In DIII-D, locked modes were observed before L-H transitions
    - Indicating NRMP is not entirely optimized
    - Still, showing value of QS optimization



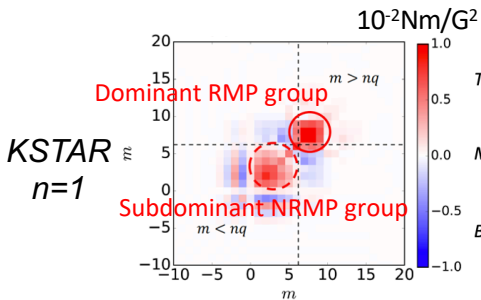
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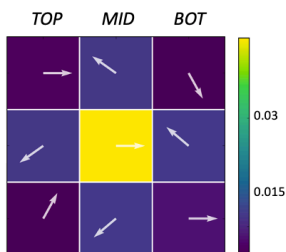
# Torque response matrix offers fundamental approaches to design coils and create large quasi-symmetric tokamak deformation

- Torque mode matrix reveals the second dominant group which should be targeted in subsidiary residual EF correction
- If coils are already designed, torque-coil matrix can be used to deform EF to a quasi-symmetric residual using the correction coils

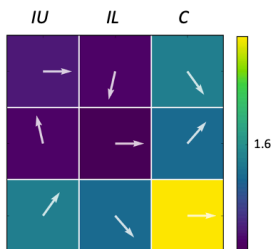
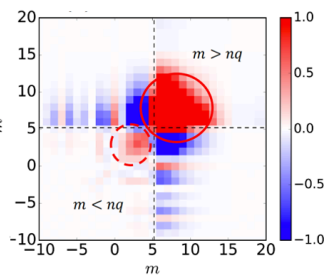
Torque mode matrix



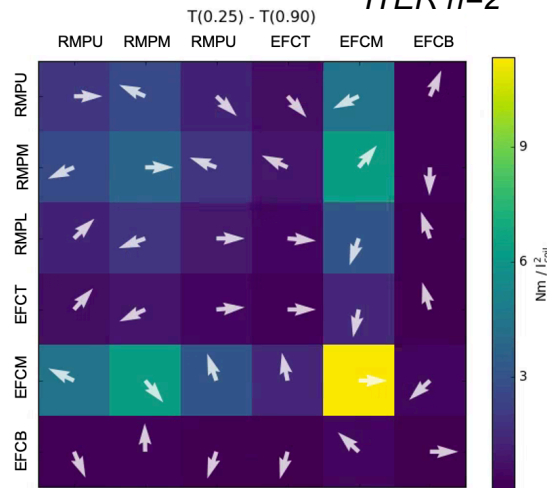
Torque-coil matrix



DIII-D n=1



ITER n=2



\*RMP (10kA), EFC (100kA)

\* Arrow indicates phase

# Summary

- Residual non-axisymmetry after EFC against dominant resonant mode can still cause a significant impact depending on cases (e.g. NSTX-U or COMPASS)
- As a complementary approach, residual non-resonant EF can be further optimized towards quasi-symmetry
- Such a quasi-symmetric magnetic perturbation (QSMP) has been designed using GPEC torque matrix and tested in KSTAR and DIII-D using its available coils
- No negative effects were found with QSMPs in the studied cases in contrast to RMP or NRMP, despite the large overall amplitudes of perturbations
- The results indicate QSMP renders a group of safe non-axisymmetric fields, showing the feasibility of QS even in a perturbed tokamak