

DIII-D and International Research Towards Extrapolating Shattered Pellet Injection Performance to ITER

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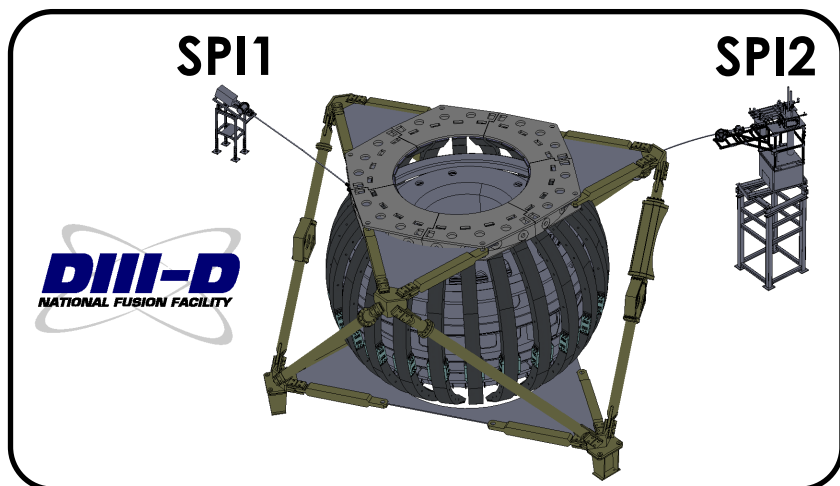
May 14th, 2021



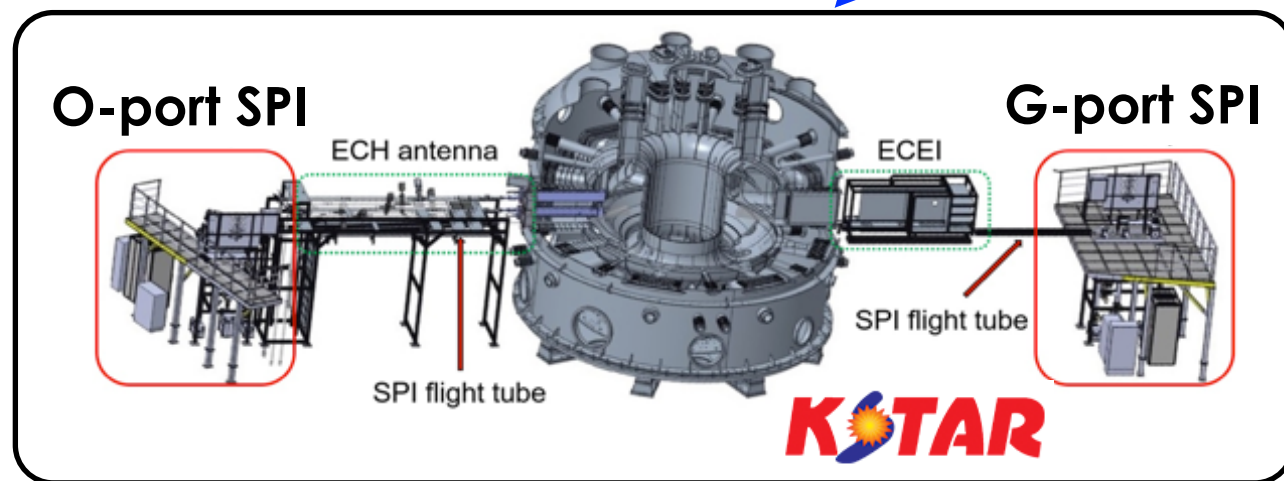
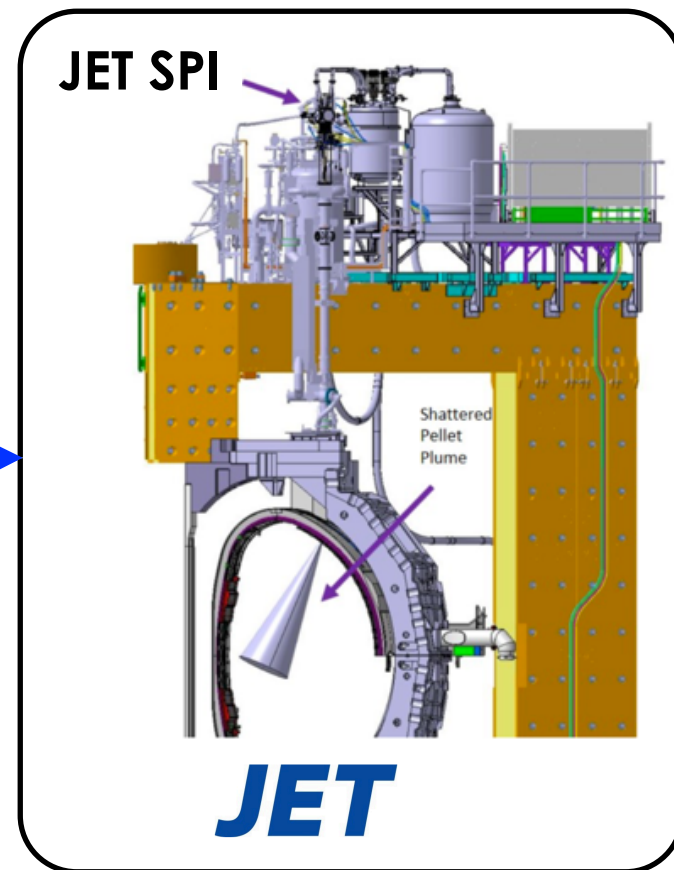
Global SPI performance predictable from validated modeling tested at DIII-D, JET, and KSTAR

- **Neon SPI dynamics primarily driven by global energy balance instead of MHD**
 - Predictive model tested on DIII-D data, also applied successfully to JET and KSTAR experiments
 - Empirical scalings of assimilation consistent with this
 - Multi-pellet shutdowns also described by this picture
- **Neon SPI generates asymmetric TQ radiation, due to localized SPI particle source**
 - Peaking factor estimates from DIII-D are close to ITER surface melt limits
- **Deuterium SPI dynamics driven by MHD growth**
 - Data from DIII-D, JET, and KSTAR support this picture

SPI modeling developed at DIII-D has now been tested on KSTAR and JET data



Predictive
KPRAD
Modeling

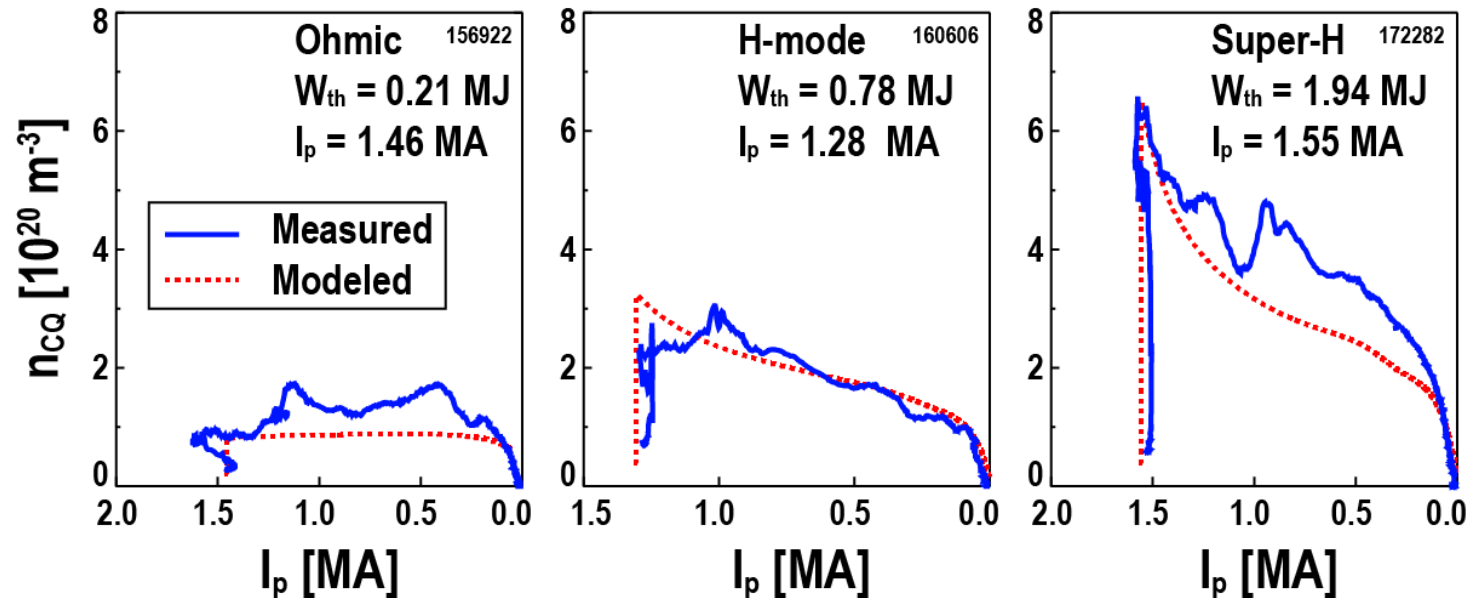


See also:

- EX-5/1220 (S. Jachmich)
- EX-5/831 (J. Kim)

Global energy balance drives disruption dynamics during neon SPI

- **Predictable from 0D KPRAD^{1,2} simulations with SPI ablation model³, which tracks:**
 - Species-dependent, shielding-limited ablation of SPI plume
 - Main-ion and impurity ionization, recombination, and radiation
 - Ohmic heating
 - Inductive coupling to wall currents



- **Simulations do NOT include MHD or particle transport effects**
- **Particle assimilation determined instead by global energy balance**

Empirical scalings for CQ density are also consistent with global energy balance being the dominant physics

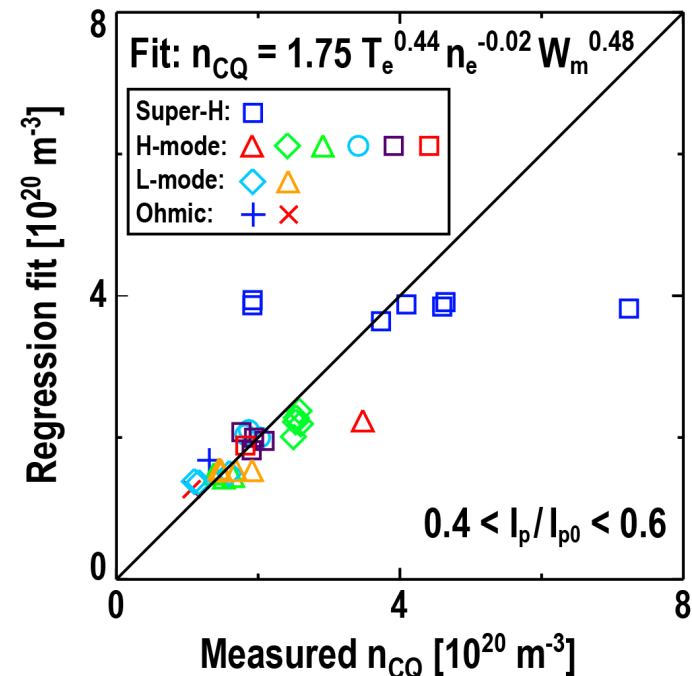
- For given pellet size/composition, CQ densities are predictable from only globally averaged parameters

$$\underbrace{\bar{n}_{CQ}}_{\text{CQ density}} = C \cdot \underbrace{T_e^{\alpha_T} \cdot \bar{n}_e^{\alpha_n} \cdot W_m^{\alpha_m}}_{\text{Pre-SPI plasma parameters}}$$

- Regression fit from large DIII-D database

- $0.8 \text{ MA} \leq I_p \leq 1.6 \text{ MA}$
- $0.1 \text{ MJ} \leq W_{th} \leq 2.0 \text{ MJ}$

DIII-D SPI database, with 7 mm Ne pellet



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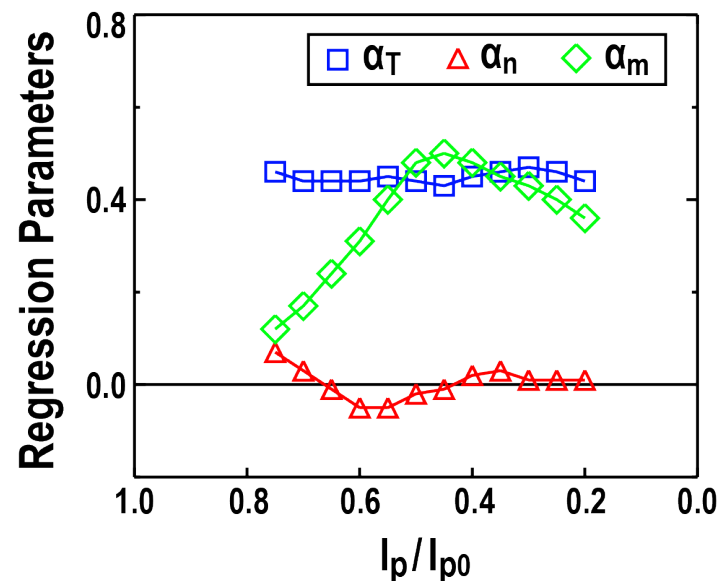
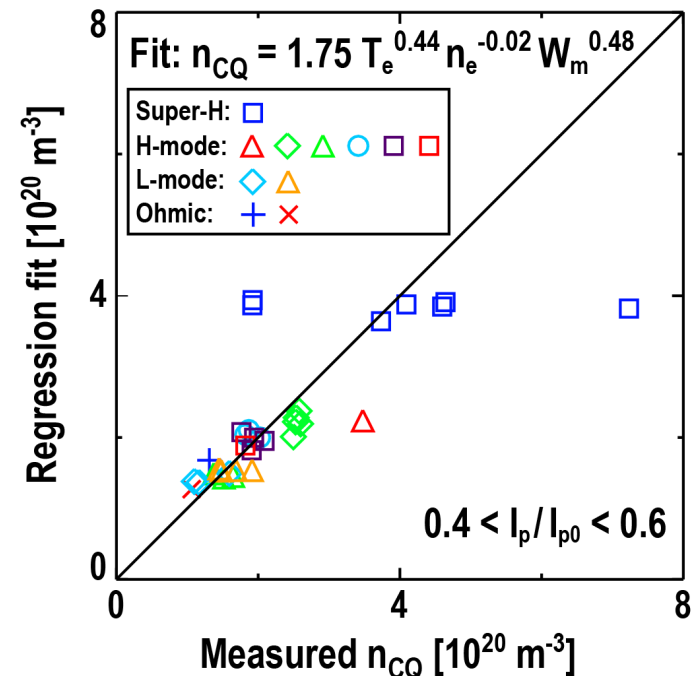
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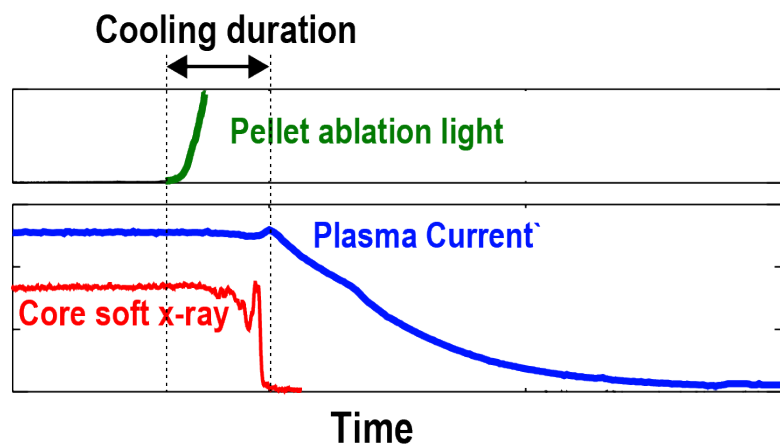
- $0.8 \text{ MA} \leq I_p \leq 1.6 \text{ MA}$
- $0.1 \text{ MJ} \leq W_{th} \leq 2.0 \text{ MJ}$

- Early assimilation depends primarily on electron temperature
- Ohmic dissipation of W_{mag} sustains ionization later in CQ

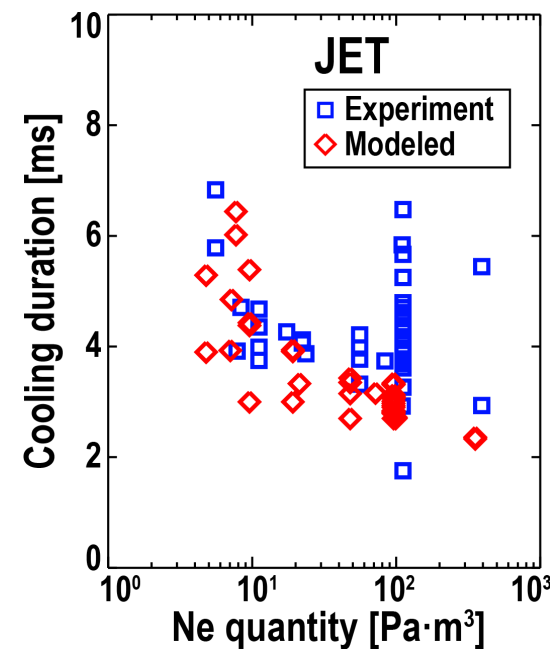
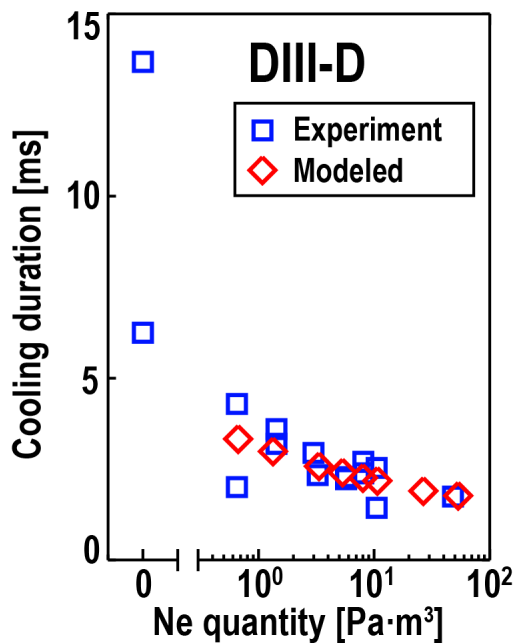
DIII-D SPI database, with 7 mm Ne pellet



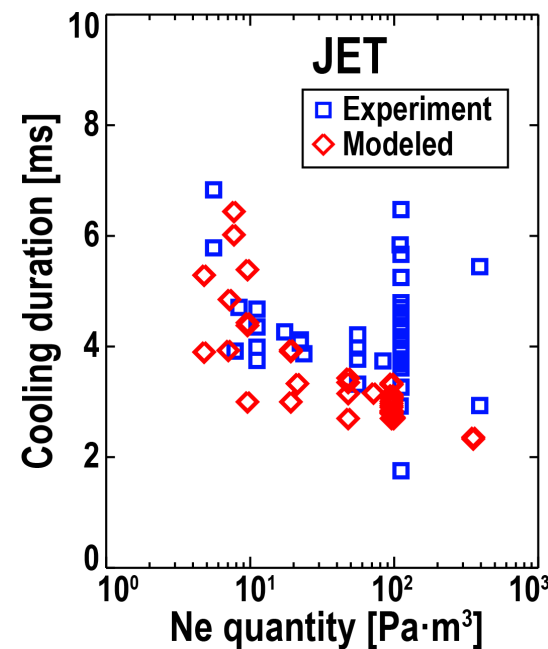
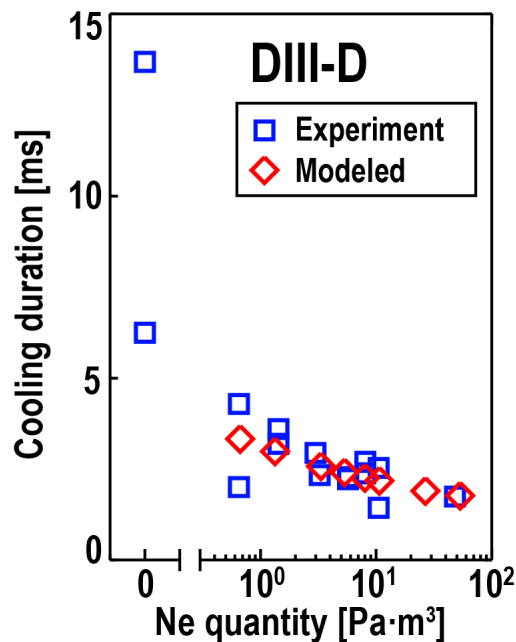
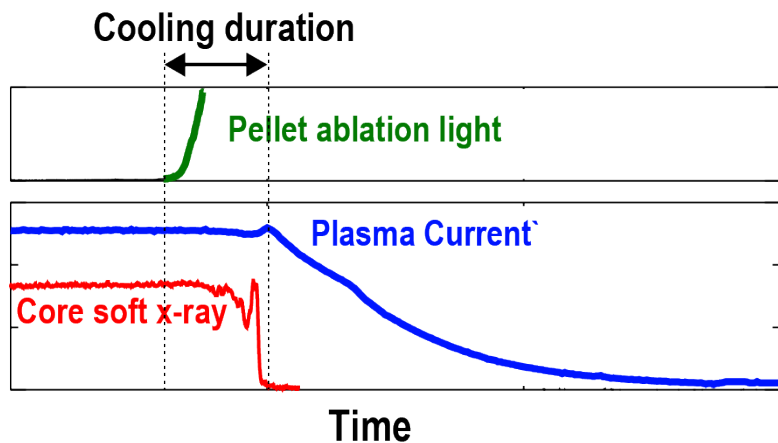
Cooling duration trends are matched by KPRAD, governed primarily by injected neon quantity



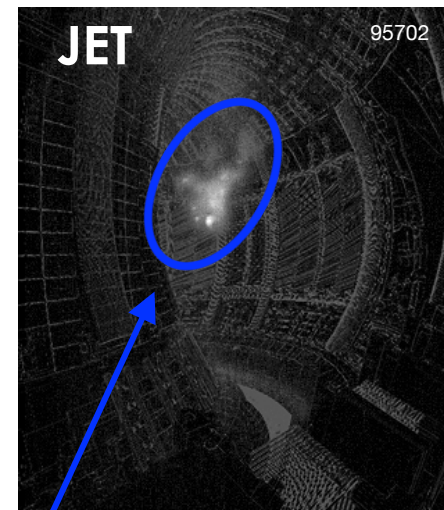
- Cooling duration gives upper bound on time for injection to contribute to TQ mitigation



Cooling duration trends are matched by KPRAD, governed primarily by injected neon quantity



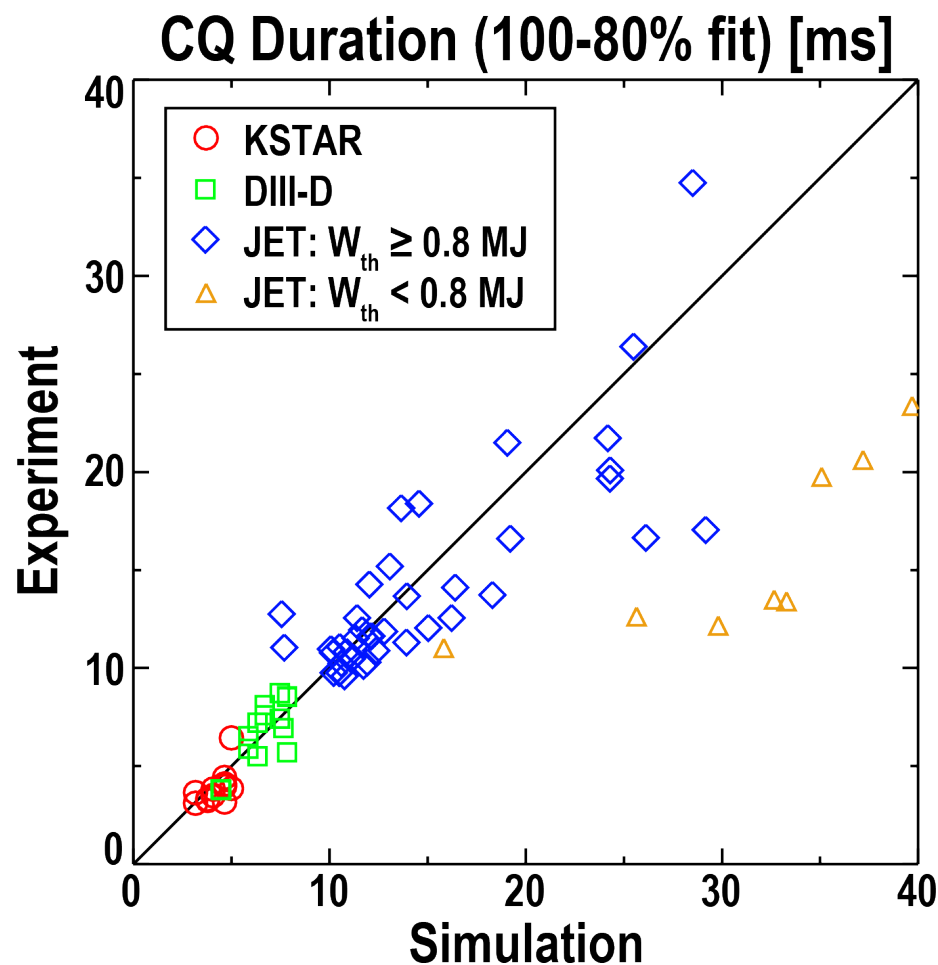
- Cooling duration gives upper bound on time for injection to contribute to TQ mitigation
- Particles unablated by this time (end of TQ) travel ballistically through CQ plasma with minimal assimilation
 - Consistent with fast camera images



Unassimilated fragments

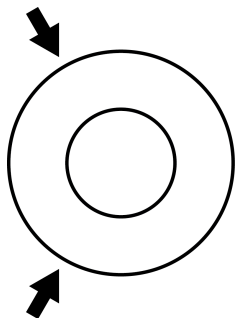
CQ rates are quantitatively predictable, determined largely by neon assimilation and resulting post-TQ plasma resistivity

- **Model accurate except at lowest energy ($W_{th} < 0.8$ MJ)**
 - Difficult to accurately model ablation rates at low temperature
- **Model accounts for both plasma parameters and injection species mixture**
 - Predictive capability required for ITER DMS to determine appropriate injection quantities/species
- **Simulations are accurate across device size, giving confidence in projectability**

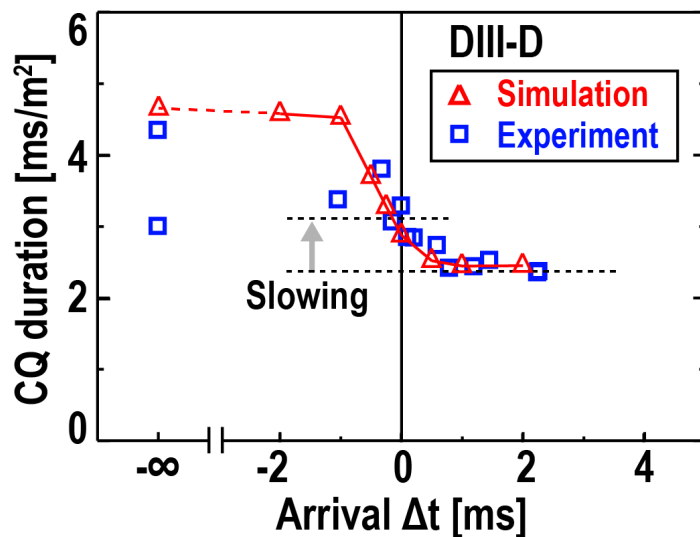


Dual-SPI performance matched by KPRAD, consistent with global energy balance being primary physics over 3D effects

53.3 Pa-m³ Ne

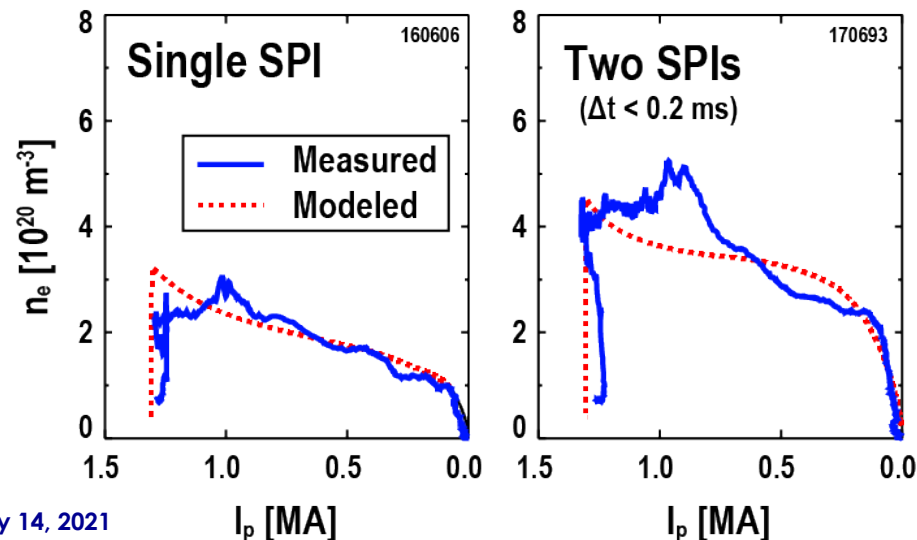


1.3 Pa-m³ Ne
36.0 Pa-m³ D₂



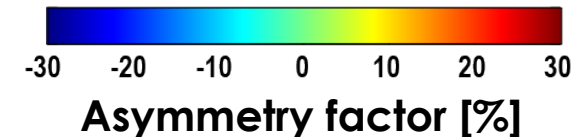
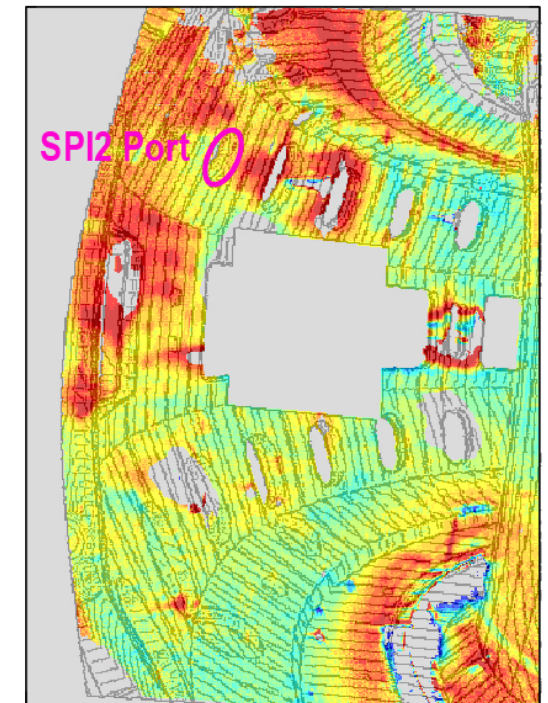
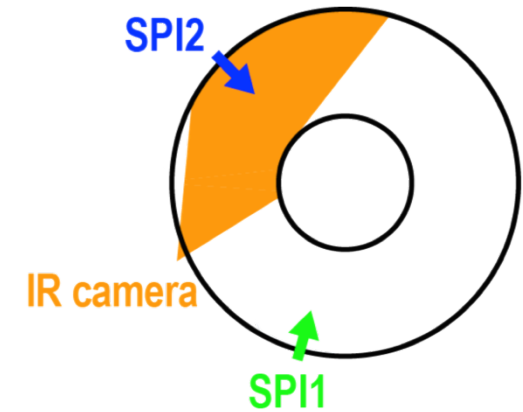
- Synchronous pellets ($\Delta t=0$) result in less neon assimilation and a slower CQ
- 0D simulations match experiment by accounting for additional deuterium
 - Does not account for separate injection locations
- Additional density rise from second SPI matched by KPRAD

- Other physics, such as MHD or localized particle sources, are primarily important for higher-order effects (such as asymmetries)



TQ radiation asymmetry due to localized injection source is observed experimentally and in extended-MHD simulations

- Asymmetries observed from comparing first-wall IR thermography for different SPI systems
 - Due to rapid localized injection, rather than MHD heat flux
- Radiation peak is broadly centered around SPI port
 - Consistent with DIII-D NIMROD simulations¹
- Estimated TQ toroidal peaking factor = $1.9 + 0.5/-0.3$
 - Consistent with DIII-D NIMROD simulations¹
- Close to ITER surface melt limit² (peaking factor ~ 2)
 - Not yet known if multiple SPIs reduce peaking

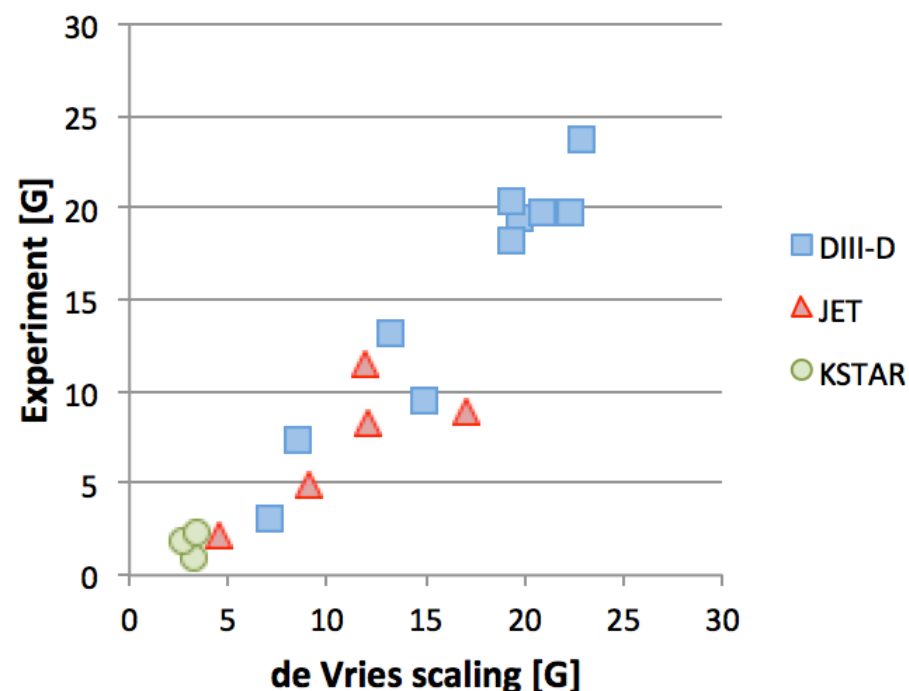
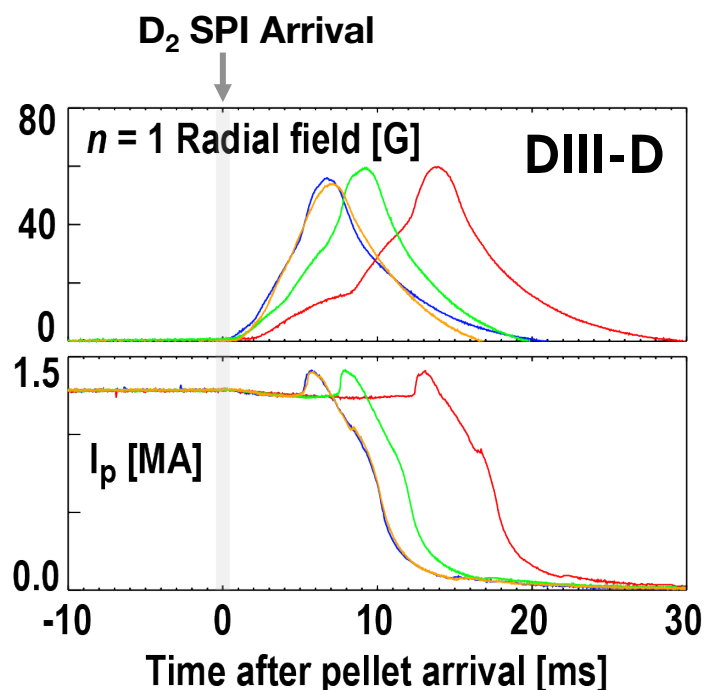


¹ C. Kim, et al., This conference

² M. Sugihara, et al., Nucl. Fusion **47** (2007) 337

MHD growth determines disruption timescales following deuterium SPI

- Deuterium SPI important for RE mitigation, by increasing collisional dissipation and decreasing hot-tail seed formation through dilution cooling
- Without neon impurity radiation, MHD growth becomes important process
- In all three devices, TQ occurs when $n=1$ MHD amplitude reaches critical value¹



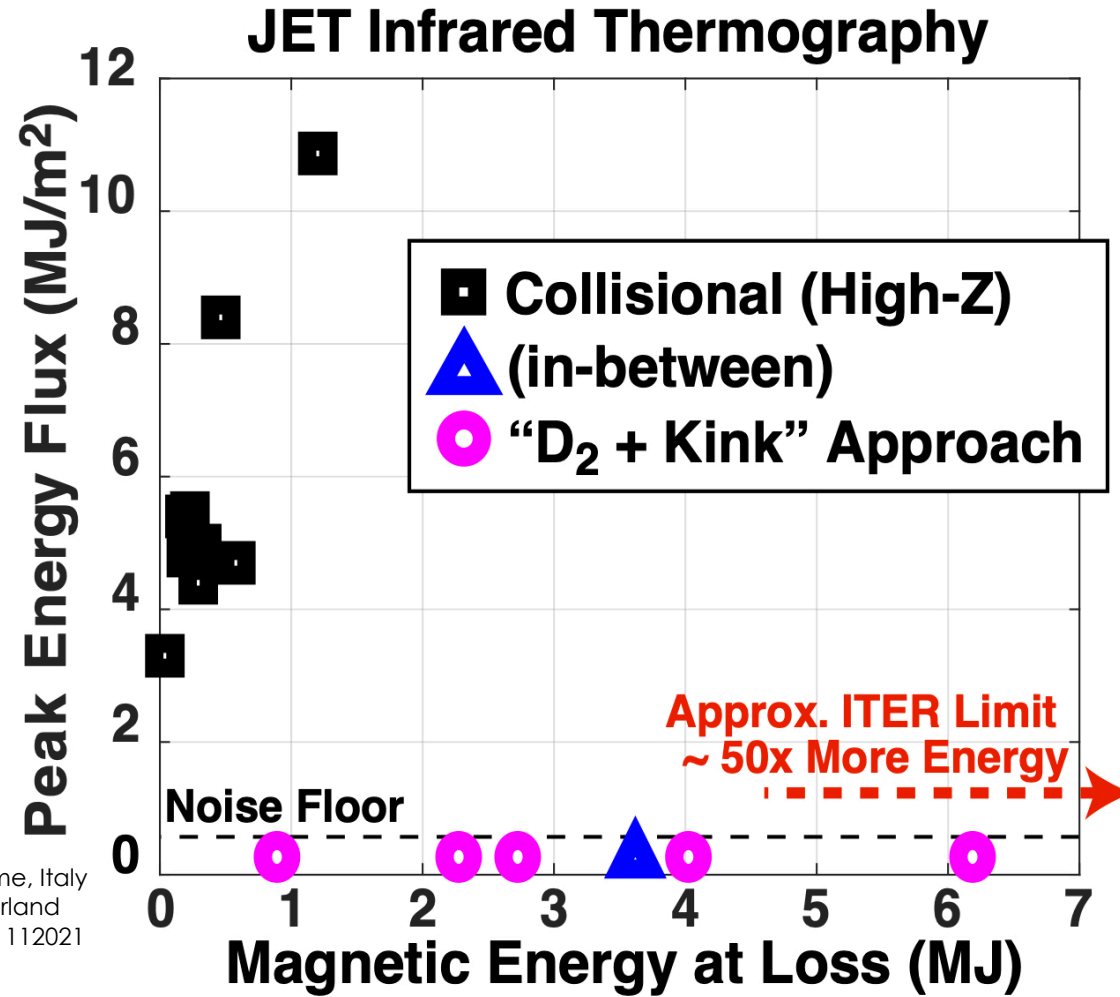
A Novel Path to Runaway Electron Mitigation via Deuterium Injection and Current-Driven Kink Instability

Produced by:
C. Paz-Soldan¹

On behalf of:

C. Reux², Y.Q. Liu³, N. Eidietis³, S. Silburn⁴, K. Aleynikova⁵, P. Aleynikov⁵, S. Sridhar², E. Joffrin², A. Lvovskiy³, L. Bardoczi³, X. Du³, S. Gerasimov⁴, F. Rimini⁴, G. Szepesi⁴, V. Bandaru⁵, M. Hoelzl⁵, G. Papp⁵, G. Pautasso⁵, L. Baylor⁶, D. Del-Castillo Negrete⁶, D. Spong⁶, E. Hollmann⁷, Z. Popovic⁷, I. Bykov⁷, C. Liu⁸, C. Zhao⁸, S. Jardin⁸, S. Jachmich⁹, M. Lehnen⁹, O. Ficker¹⁰, E. Macusova¹⁰, D. Carnevale¹¹, C. Sommariva¹², A. Manzanares¹³, the DIII-D Team and JET Contributors*

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⁸Princeton Plasma Physics Laboratory Princeton, New Jersey 08543-0451, USA
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¹³CIEMAT, Madrid, Spain, *See the author list of E. Joffrin et al. 2019 Nucl. Fusion 59 112021



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EUROfusion



COLUMBIA UNIVERSITY
IN THE CITY OF NEW YORK

Contrasting Approaches Highlights Key Differences

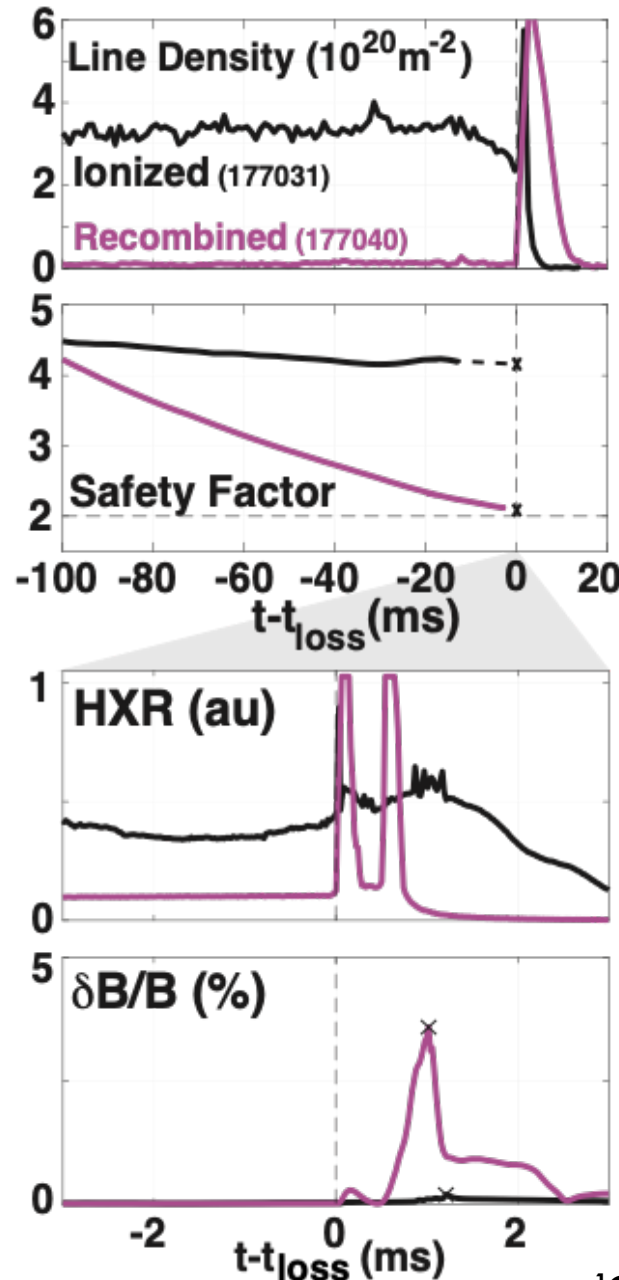
Conventional Approach:

- Inject High-Z (Ar/Ne)
- Collisionally dissipate REs

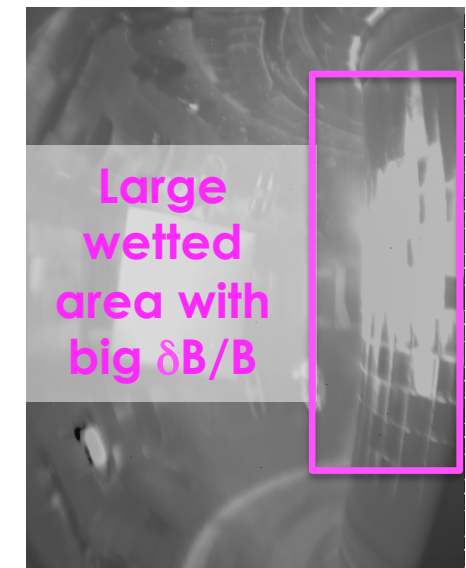
New Approach:^{1,2}

- Inject $D_2 \rightarrow$ collisionless
 - via high-Z expulsion and bulk recombination³
- Access bigger & faster MHD kink instabilities
- ~100% REs instantaneously dumped to the first wall

Only New Approach Avoids First Wall Heating



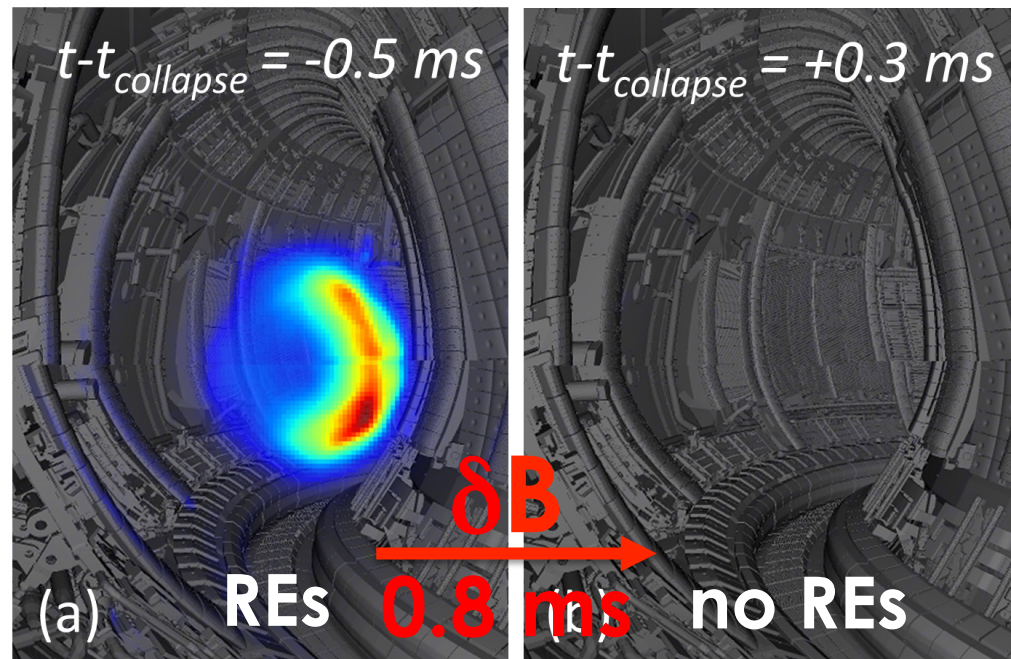
Conventional: $q \sim 4$



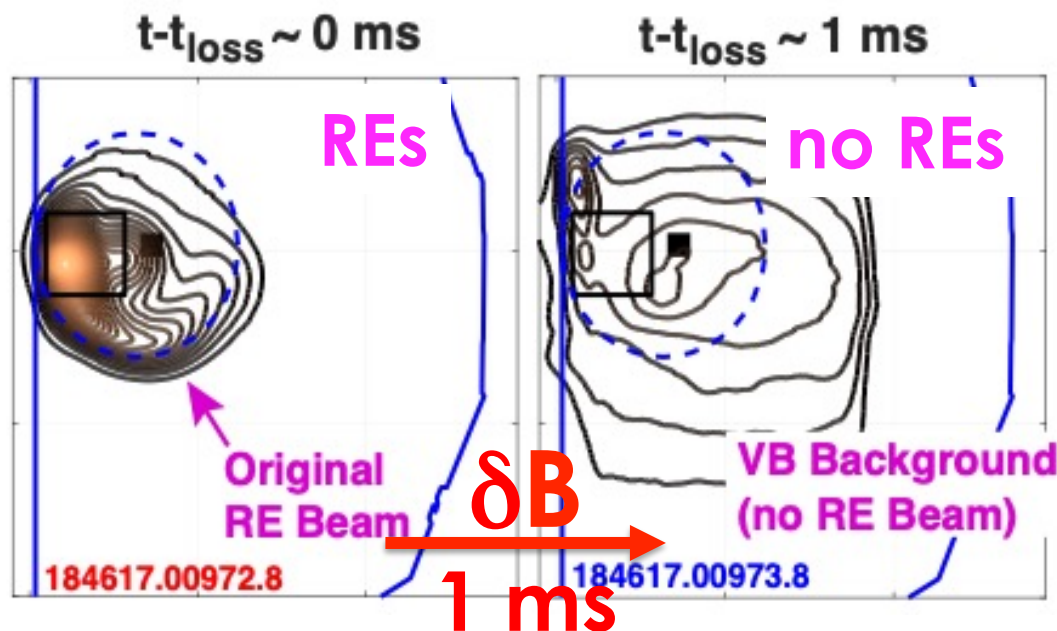
$D_2 +$ Kink: $q \sim 2$

Synchrotron Emission Confirms Full RE Termination on Sub-Millisecond Timescales

- After D_2 injection: REs can persist very long time
 - Up to 5 seconds in DIII-D
- After crossing MHD instability boundary REs vanish in < 1 ms



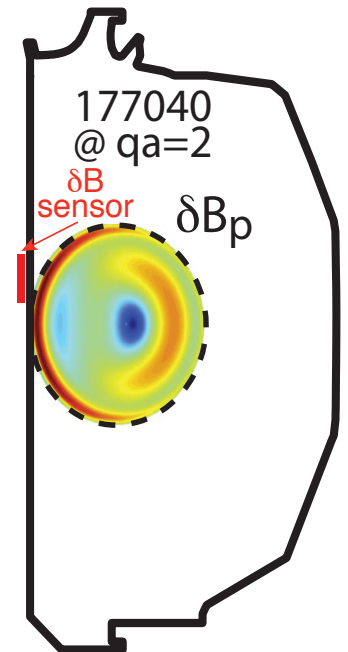
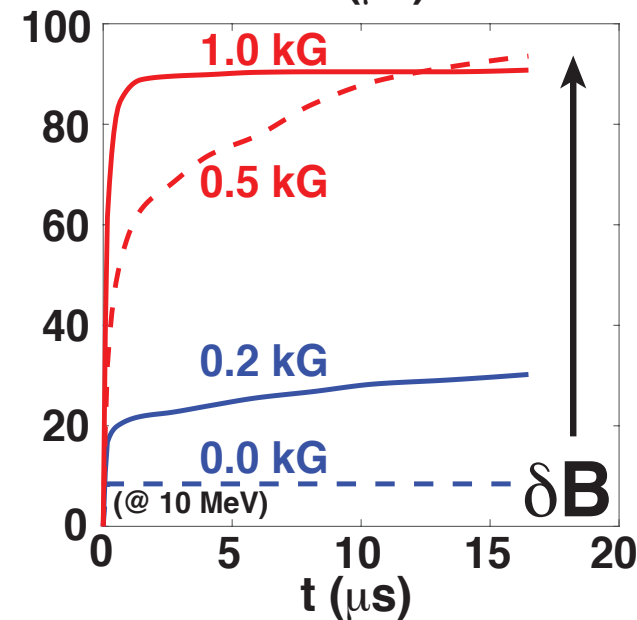
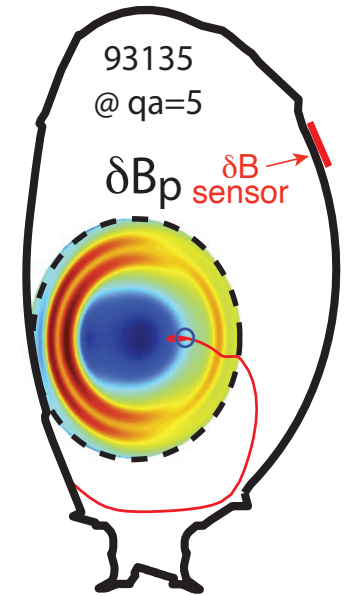
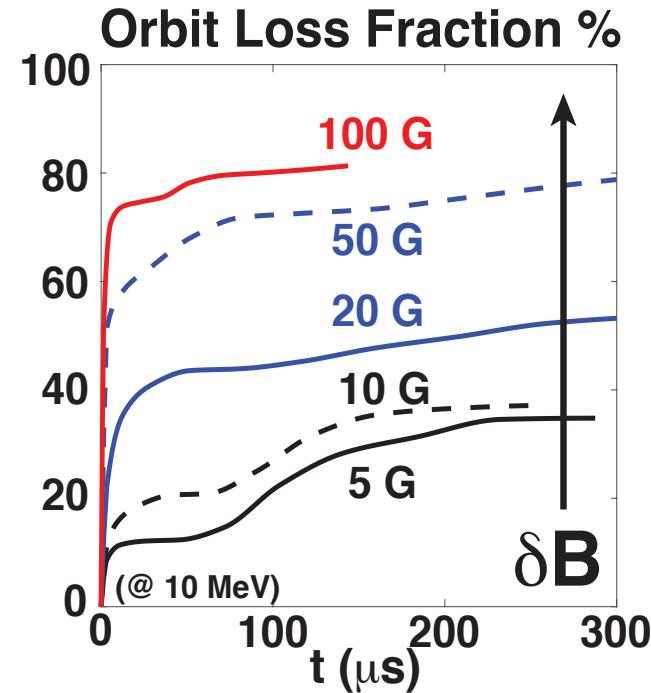
JET IR Synchrotron



DIII-D Vis. Synch.

MHD Model + Orbit Following w/ Observed $\delta B/B$ Levels Confirms Nearly all RE Orbits are Lost to the First Wall

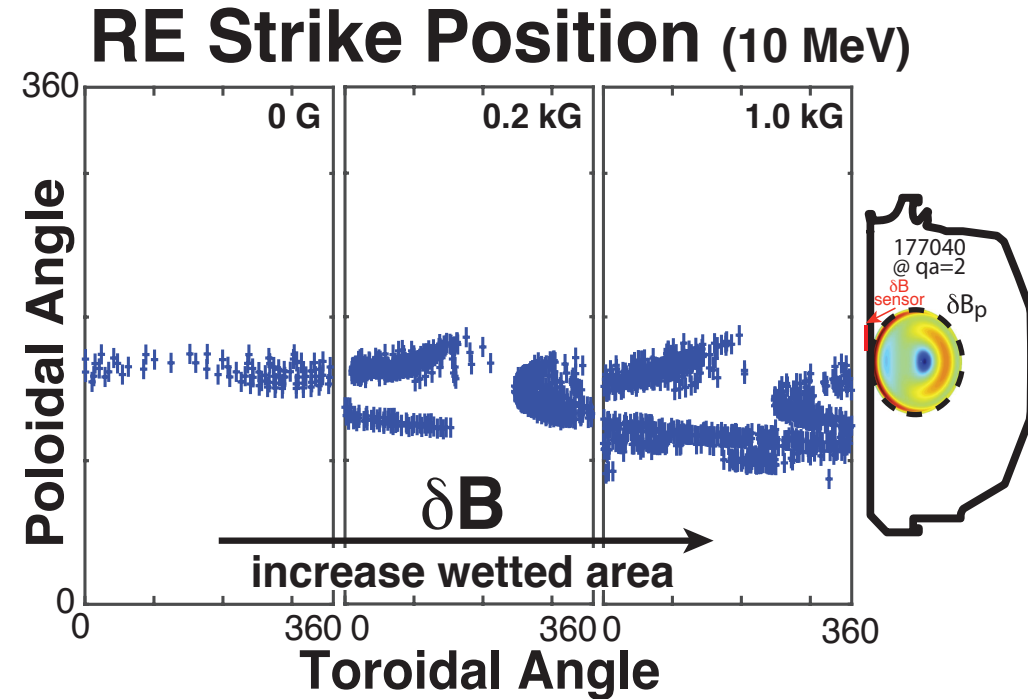
- RE orbits followed in linear MHD eigenmode structure scaled to experimental δB
- $\delta B/B$ at experimentally relevant values ($\sim 5\%$) causes most orbits to be lost to the first wall



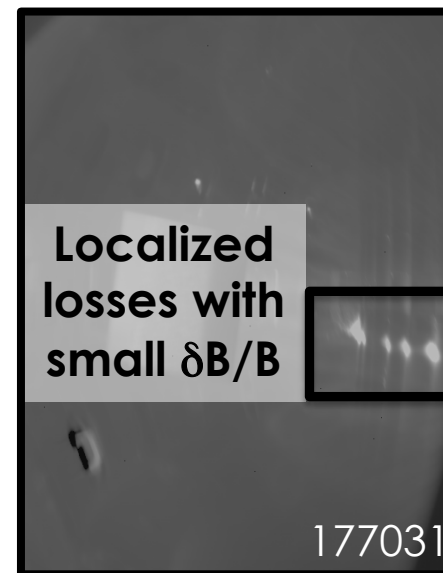
MHD Model + Orbit Following w/ Observed $\delta B/B$ Levels Confirms Nearly all RE Orbits are Lost to the First Wall

- RE orbits followed in linear MHD eigenmode structure scaled to experimental δB
- $\delta B/B$ at experimentally relevant values ($\sim 5\%$) causes most orbits to be lost to the first wall
- RE kinetic energy disperses into a **large wetted area**
 - Reduced peak heat flux

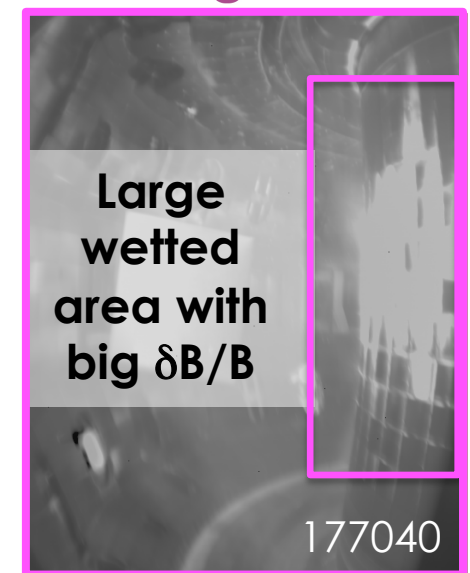
Improved Scenario for Kinetic Energy Handling



small δB

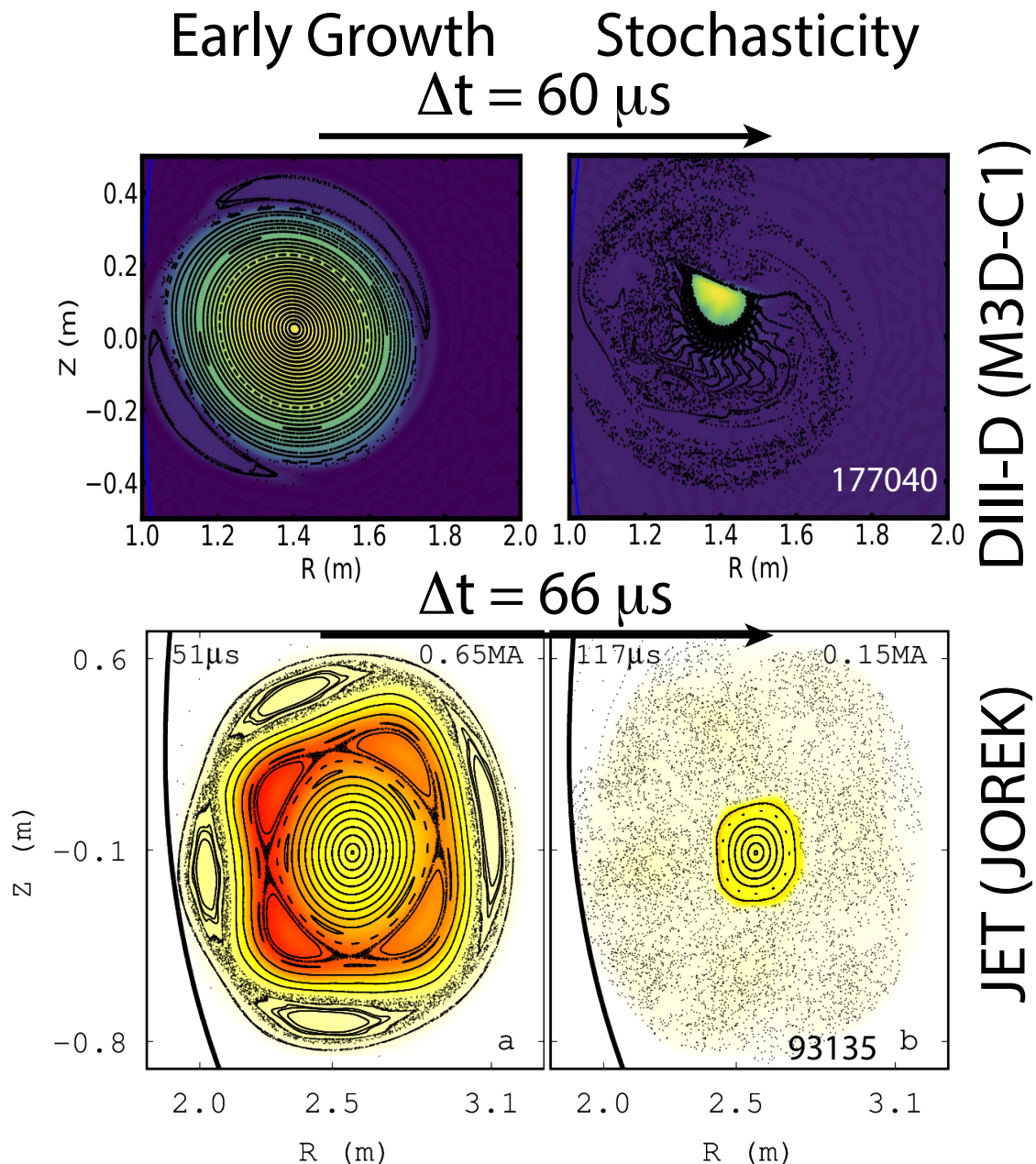


large δB



Extended MHD Modeling Reproduces Total Loss

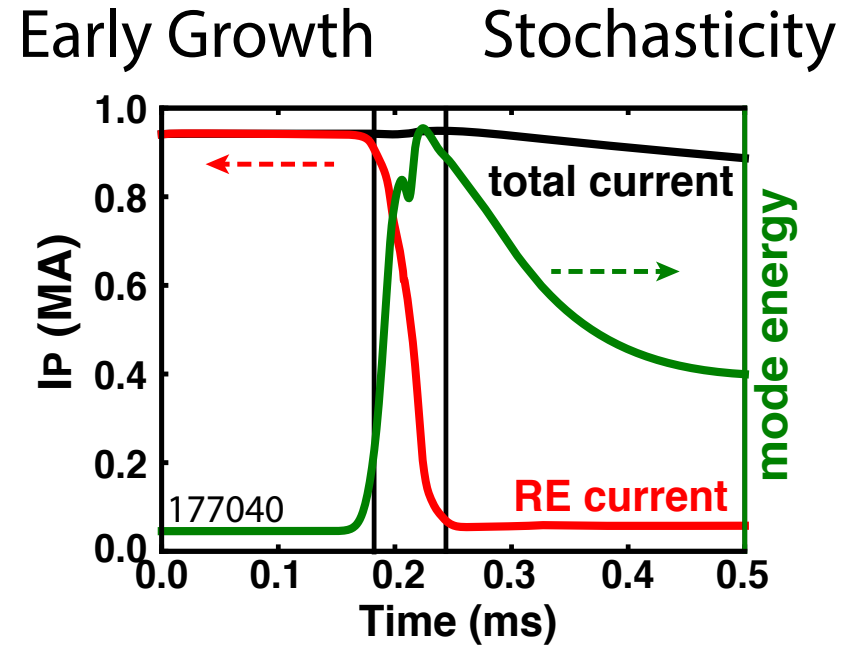
- M3D-C1 and JOREK with RE fluid model deployed
- Near-total stochasticity found in both simulations



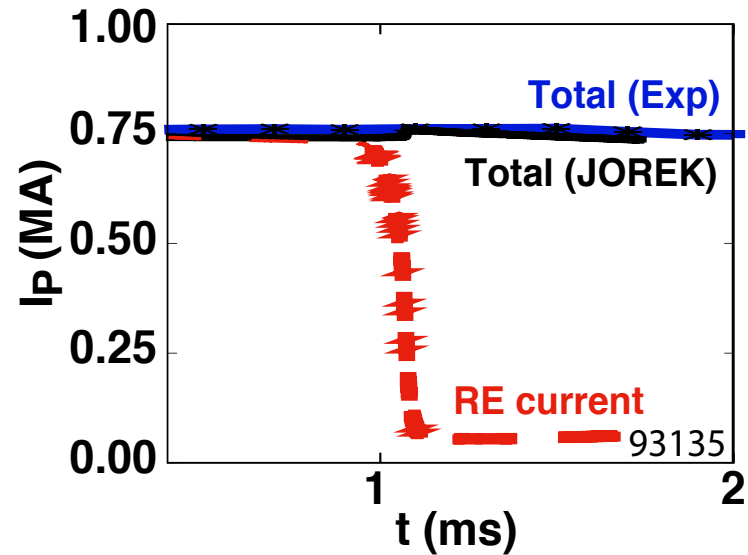
Extended MHD Modeling Reproduces Total Loss Prompt RE → Ohmic Current Transfer in DIII-D and JET

- M3D-C1 and JOREK with RE fluid model deployed
- Near-total stochasticity found in both simulations
- Prompt loss of REs drives current transfer to the bulk
- Dissipation of magnetic energy into line radiation
 - ... Not back into RE energy

Best-Case Scenario for
Magnetic Energy Handling



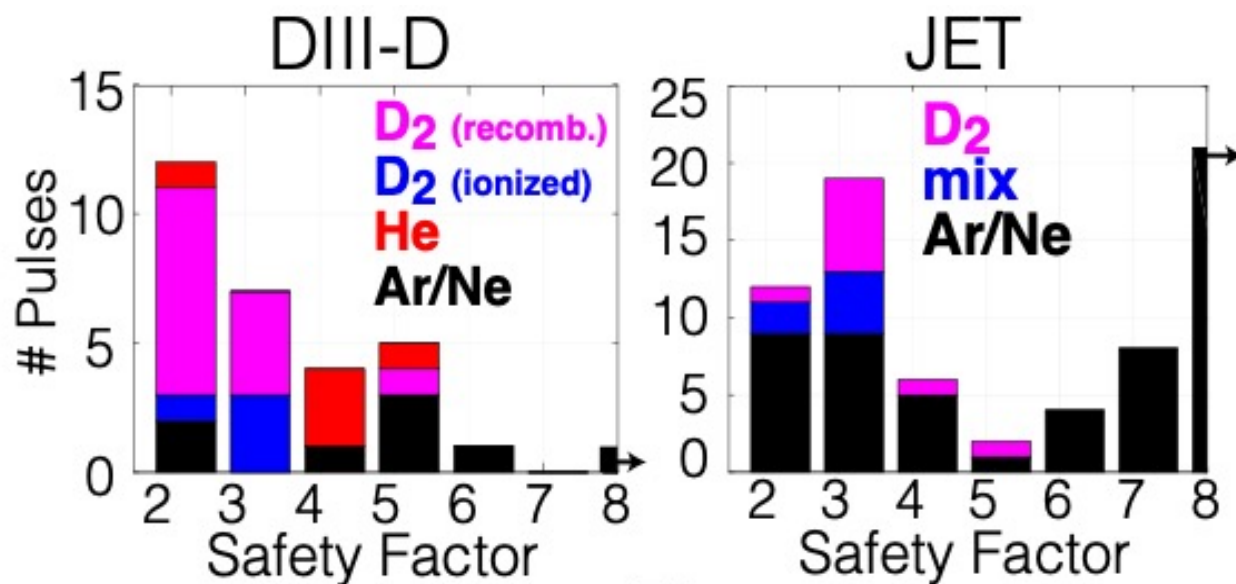
DIII-D (M3D-C1)



JET (JOREK)

D₂ Injection: 1) Facilitates Low Safety Factor Access 2) Accelerates Alfvénic Instability by Reducing Density

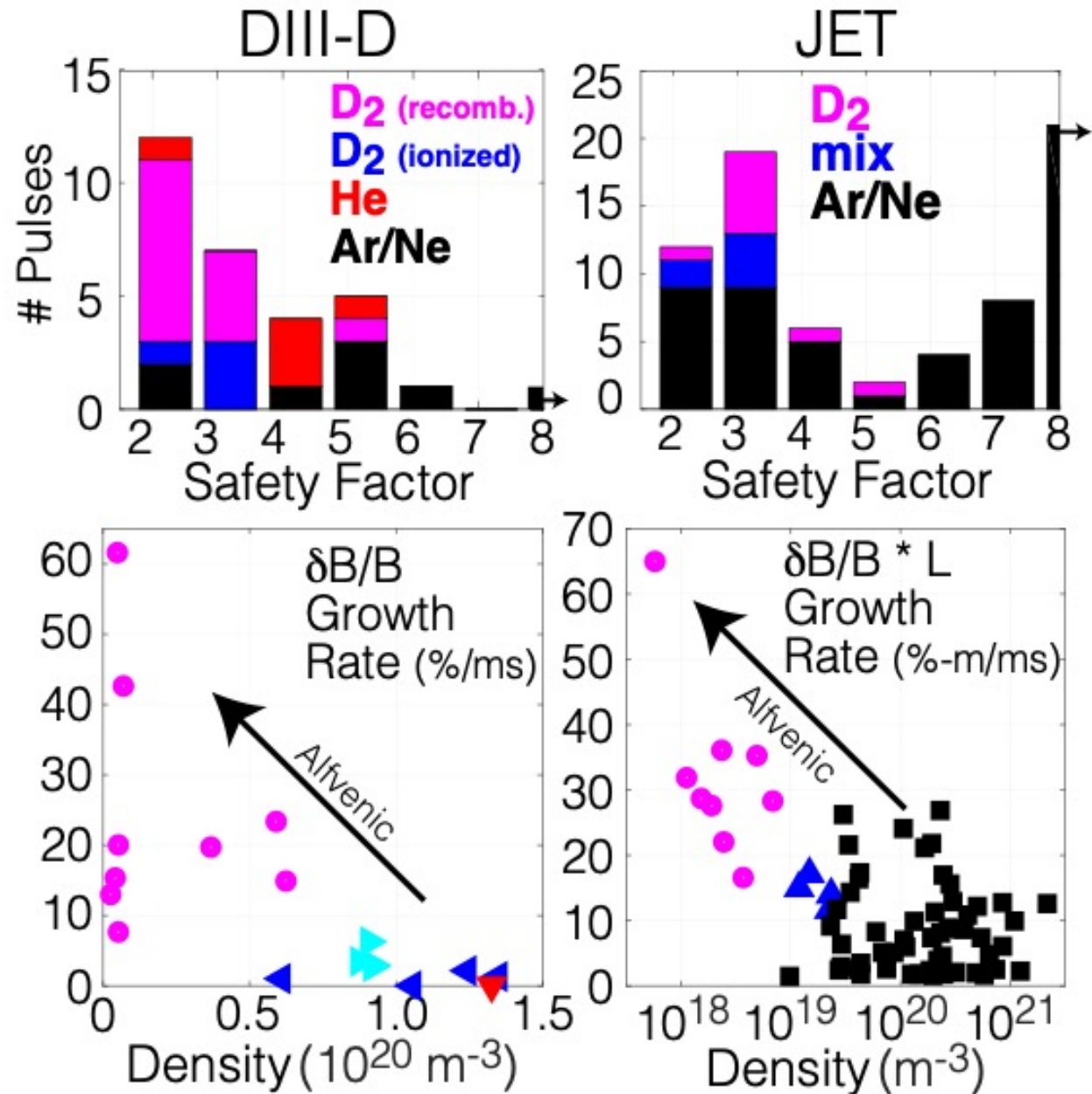
- D₂ cases tend to evolve to lower safety factor (more unstable)
 - ... not guaranteed
 - ... nor essential



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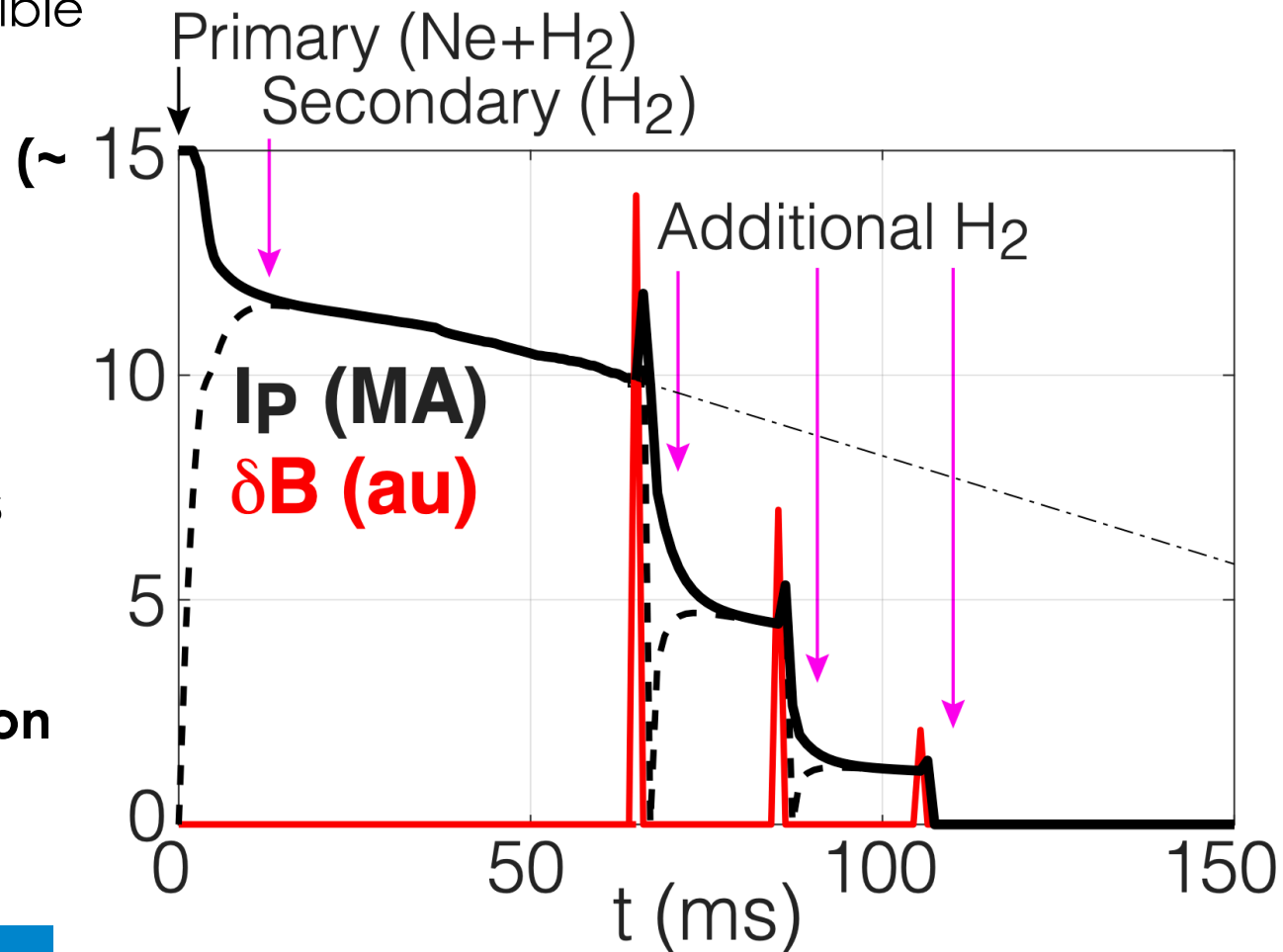
- Key D₂ affect: bulk recombination
 - Decreases density
 - Shortens Alfvén time
 - **Accelerates MHD**



New Approach Deployment in ITER DMS Will Likely Involve Multiple Loss Events (& Pellets?)

- ITER's RE beam expected to access low safety factor
 - Kink should be accessible
- ITER's avalanche gain (10^{20}) still an issue
 - “Remnant” REs will re-avalanche
- Multiple, but benign, loss events are foreseen
- Goal: keep recombination & promote large $\delta B/B$

Candidate ITER DMS Scheme



Validation Needed
@ High RE Current / Gain
... in ITER Pre-FPO Phase