

# Suggestions for Advanced Operating Mode Development in DIII-D

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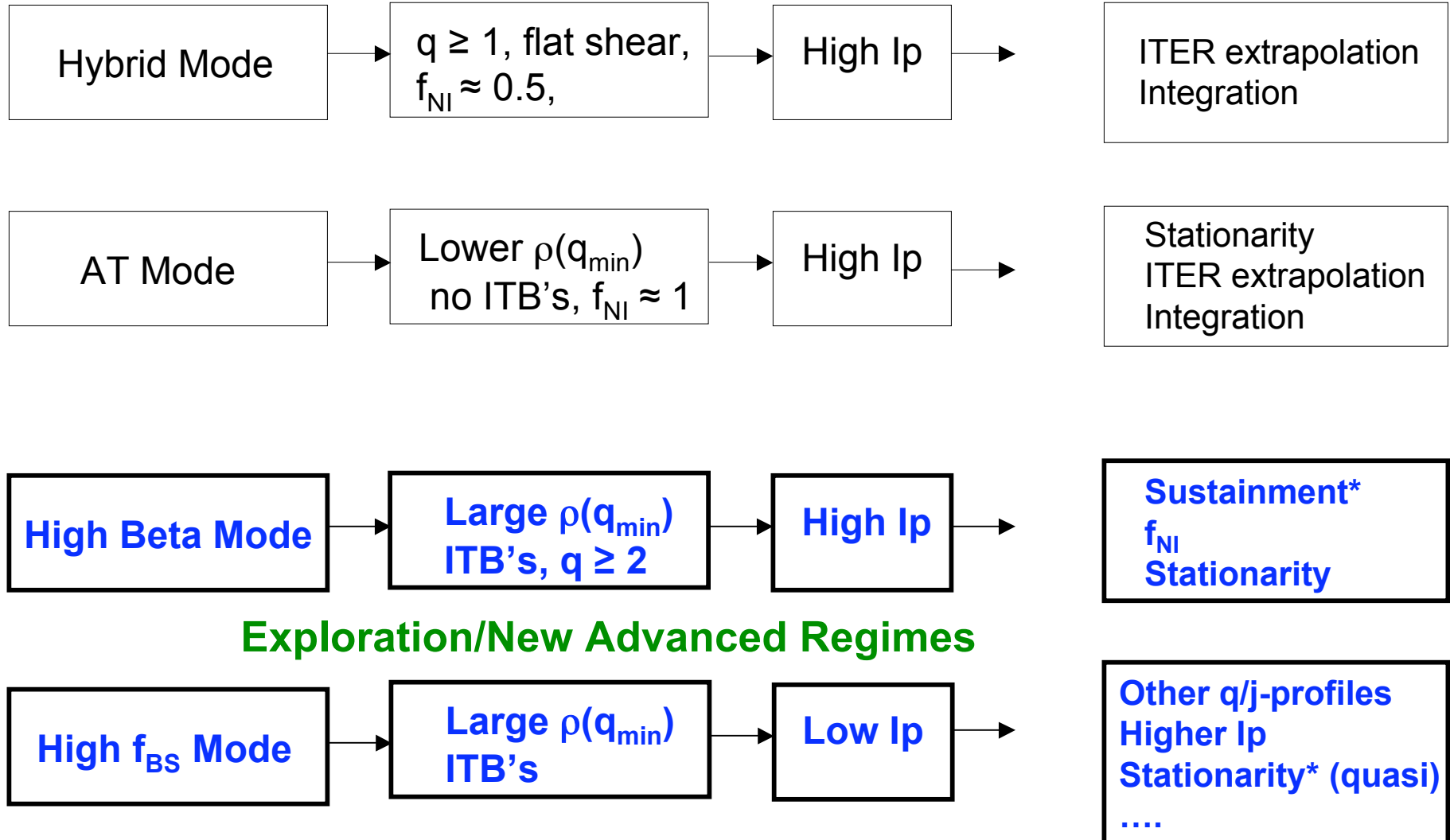
**DIII-D Advanced Tokamak Workshop,  
February 27-28, 2007**

# Major Advanced Operating Modes

**mode**

**characterization**

**issues**

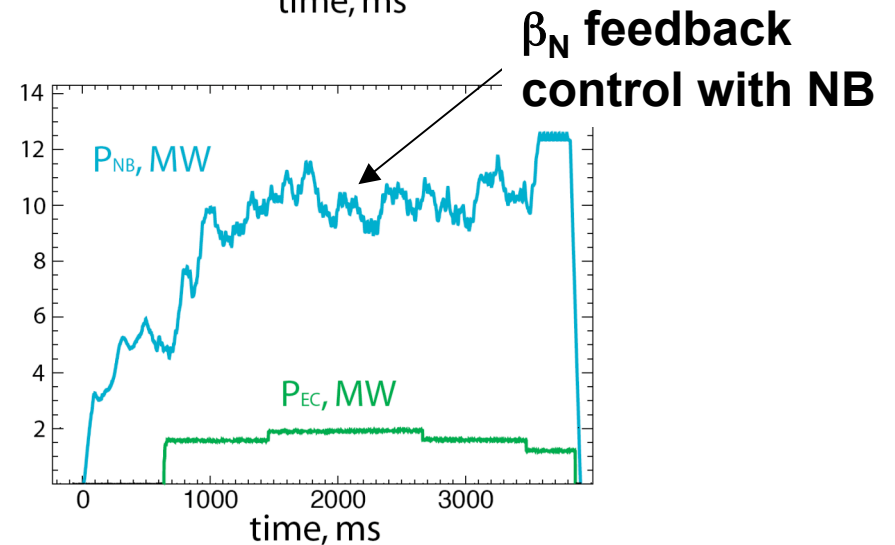
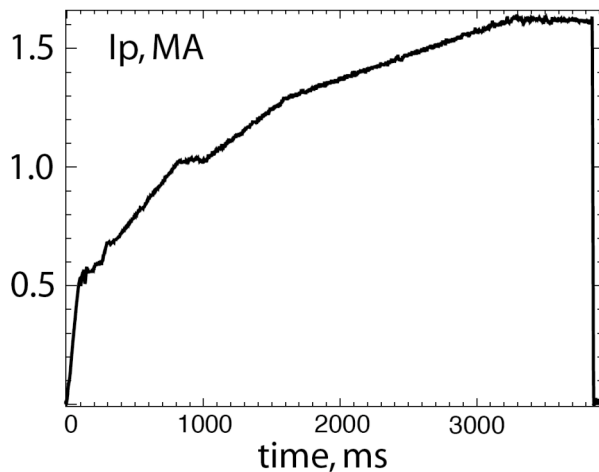
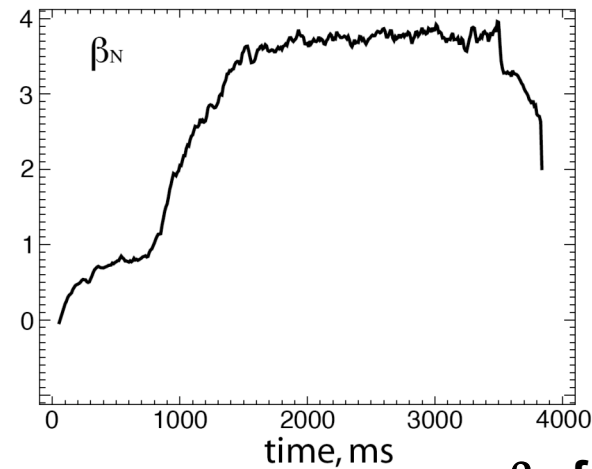
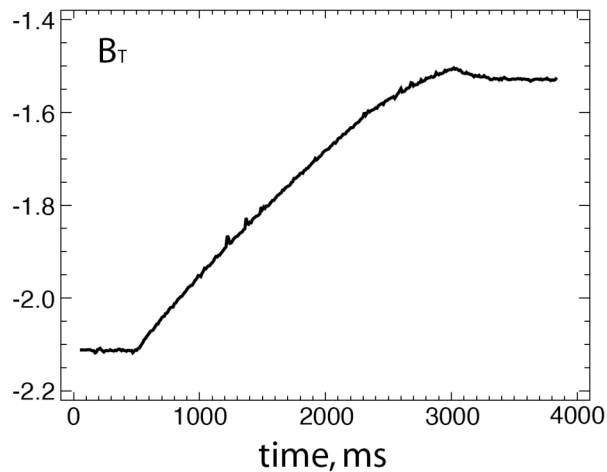


# DIII-D High- $\beta$ Discharge 122976

$I_p$  and  $B_T$  Ramps to Produce High  $q_{\min}$  and large  $\rho(q_{\min})$

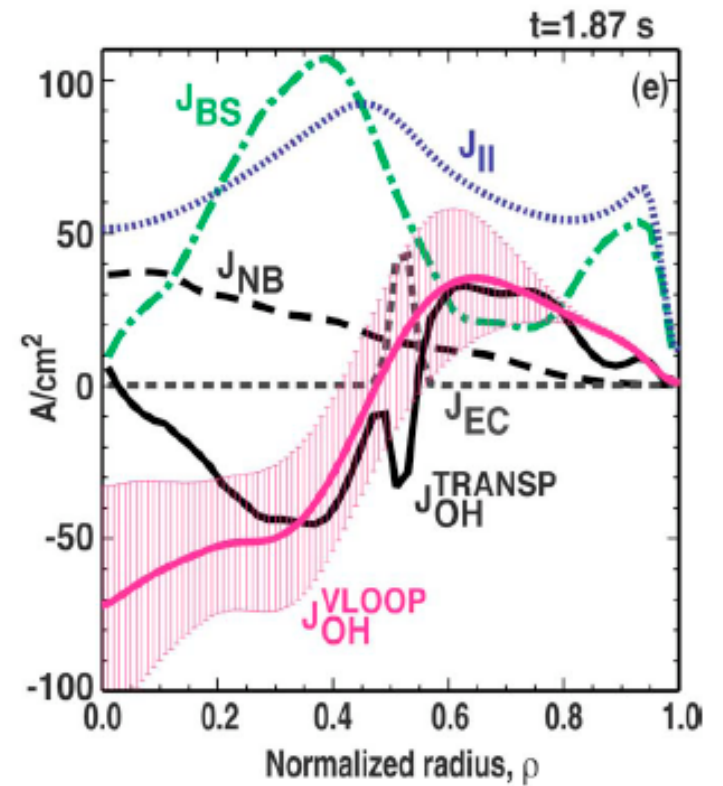
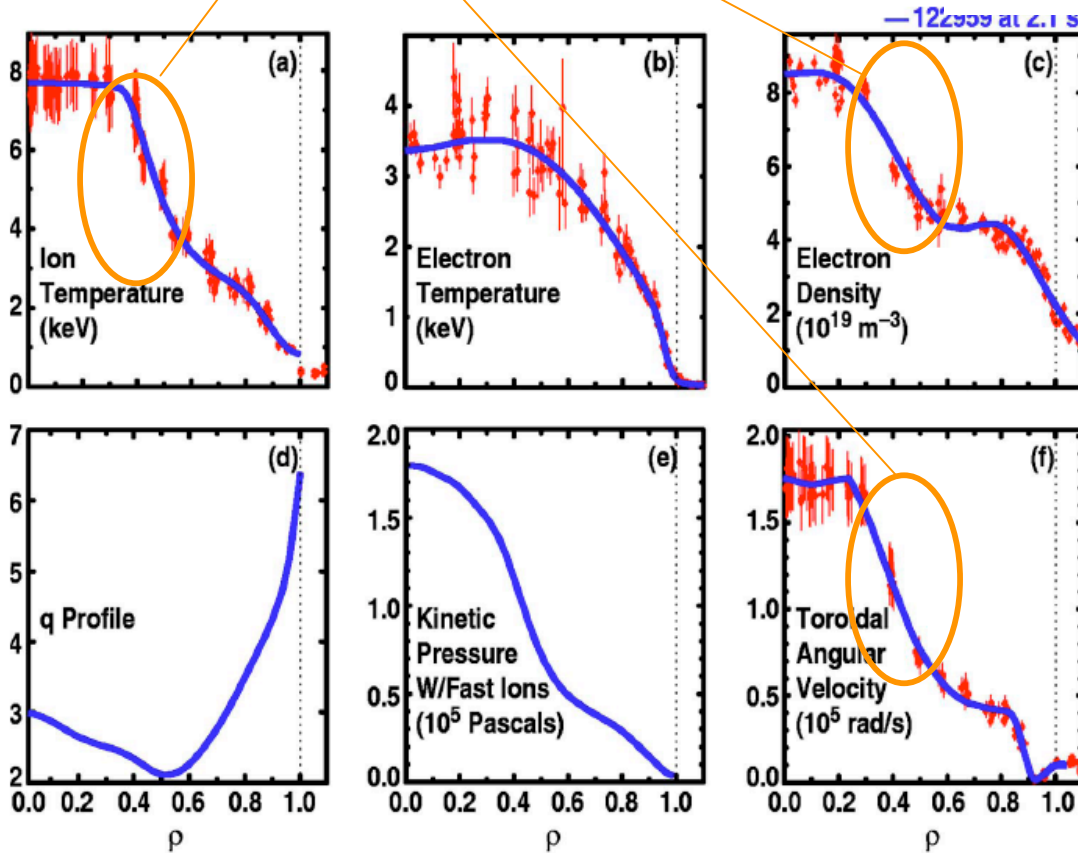
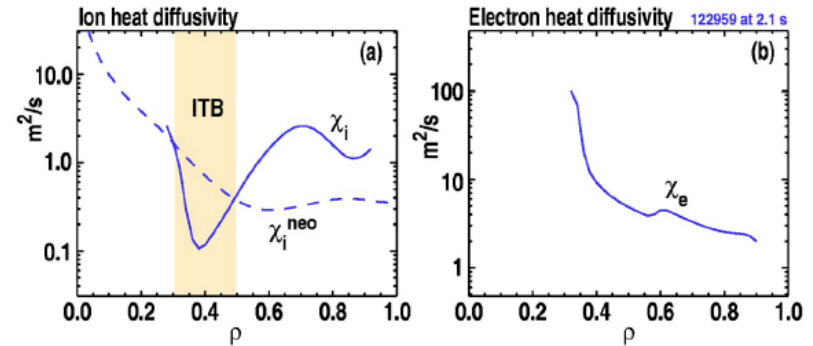
Plasma reaches  $\beta_N \approx 4$  for 2 s

Good target discharge to examine  $q$  control thru source & discharge optimization



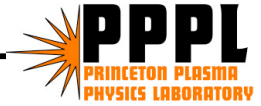
# DIII-D High- $\beta$ Discharge 122976

ITB's in ion energy, density, and momentum channels



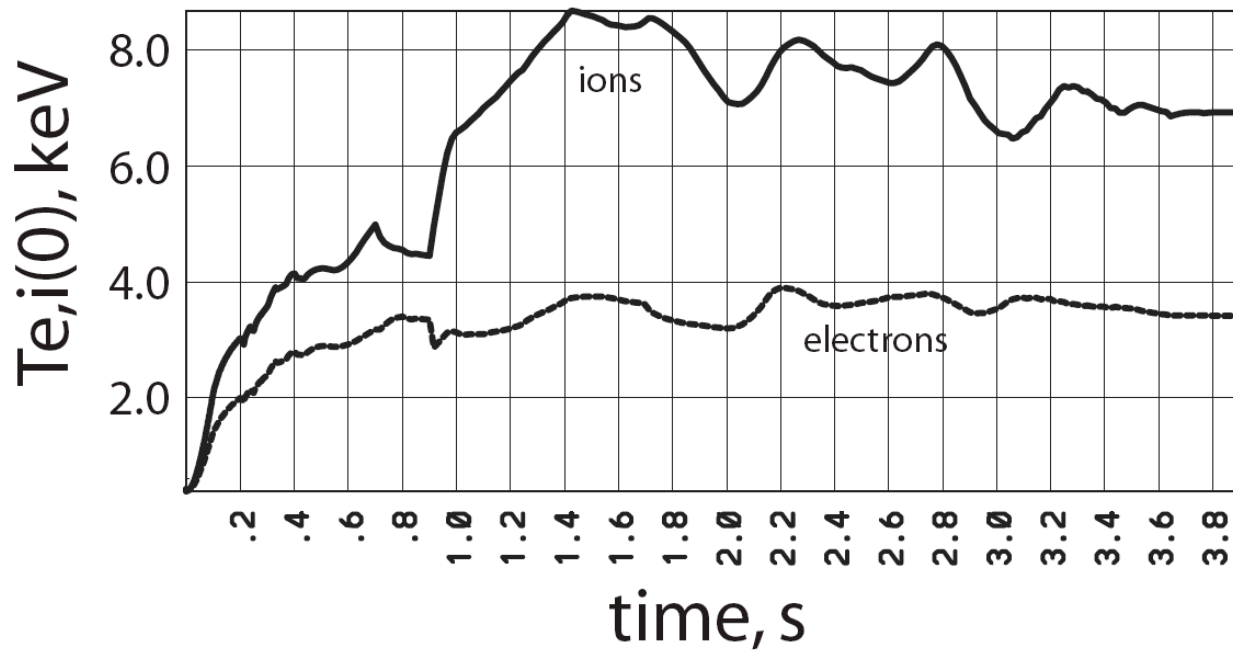
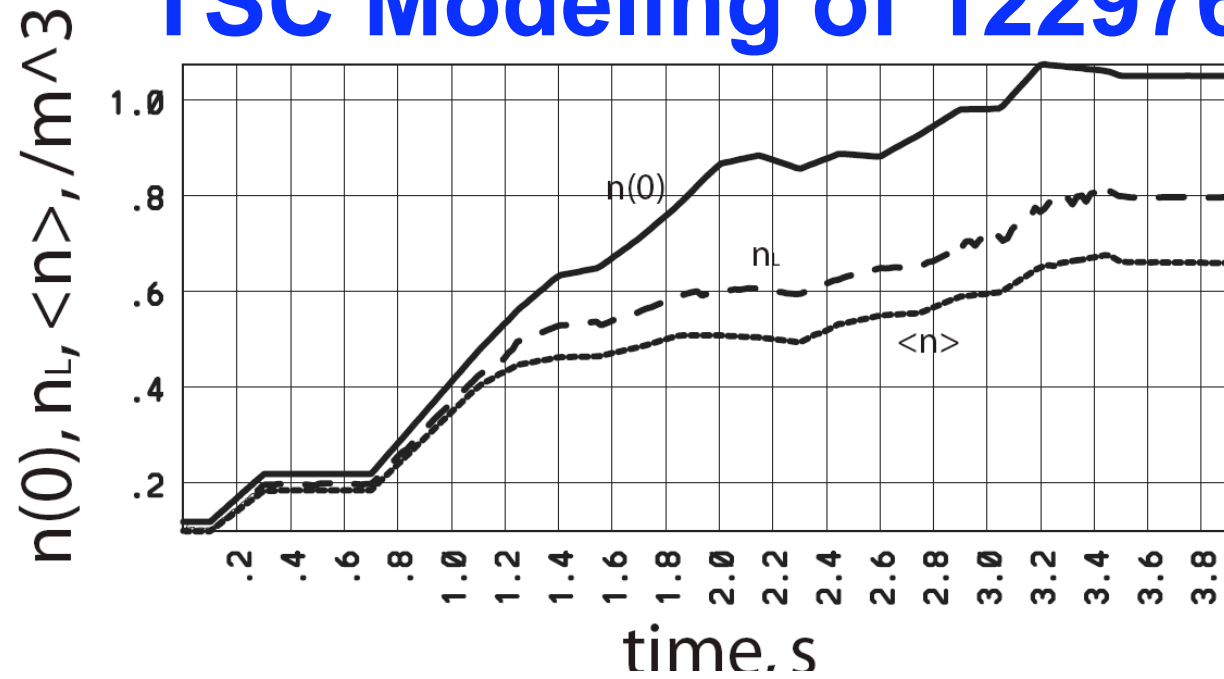
# Simulation of an Experiment to Project to Different Operating Conditions

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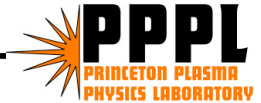


- TRANSP was run on DIII-D discharge 122976, a High Beta plasma
- TRANSP data used in TSC
  - $T_e$ ,  $T_i$ ,  $n$  passed thru
  - Thermal diffusivities, source heating and current depositions
- CURRAY was used offline to calculate the ICRF-FWCD
  - $n_{fast}$  and  $\langle E_{fast} \rangle$  taken from TRANSP to produce effective maxwellian
  - Injected powers based on L-mode and H-mode operating experience
  - Spectra from ORNL
  - Impurities specified as 2% H, 2% C for L-mode, 5% C in H-mode
  - Have 2 frequencies, 60 and 83 MHz launchers on DIII-D
  - $B_T$  varying throughout shot, so ICRF deposition is changing
- TSC was run with TRANSP data, time-slicing out to CURRAY every 250 ms
  - FW was used to replace EC
  - Density evolution was modified and FW added to EC

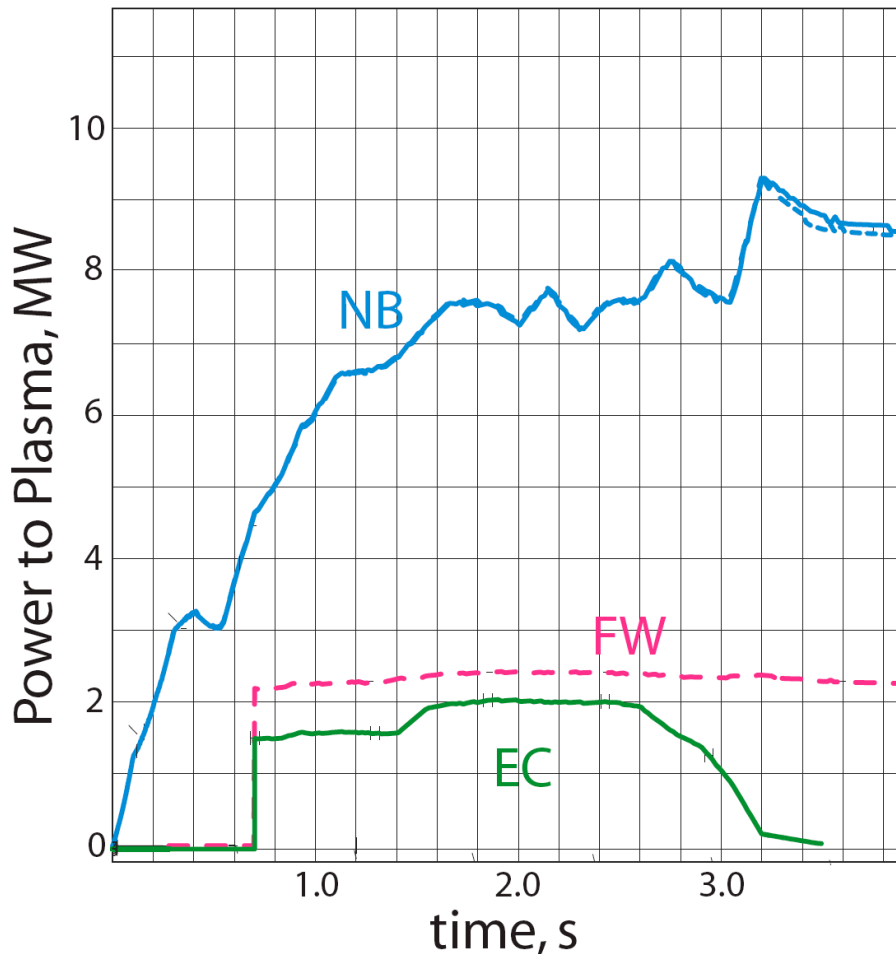
# TSC Modeling of 122976



# TSC Simulation: Compare NB + EC (112976) & NB + FW



— NB + EC (ref)  
- - - NB + FW



Using TSC simulation of 122976 as the basis, which had NB and EC H/CD

Replace EC H/CD with **2.2 MW** of FW heating only

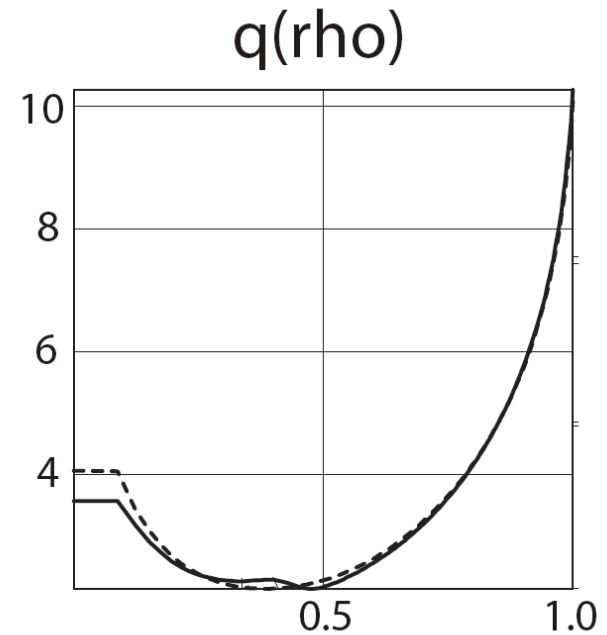
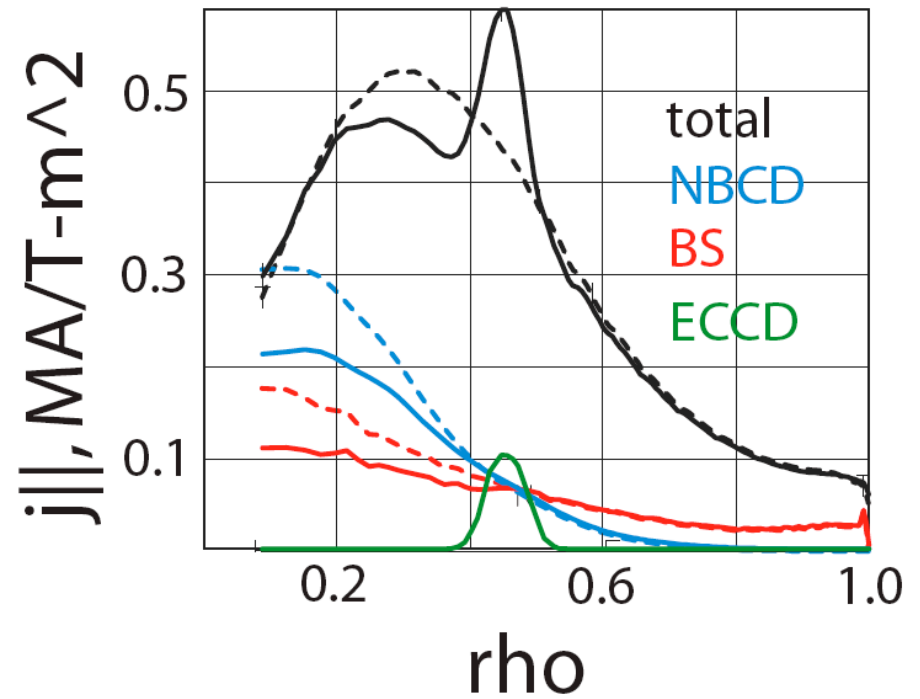
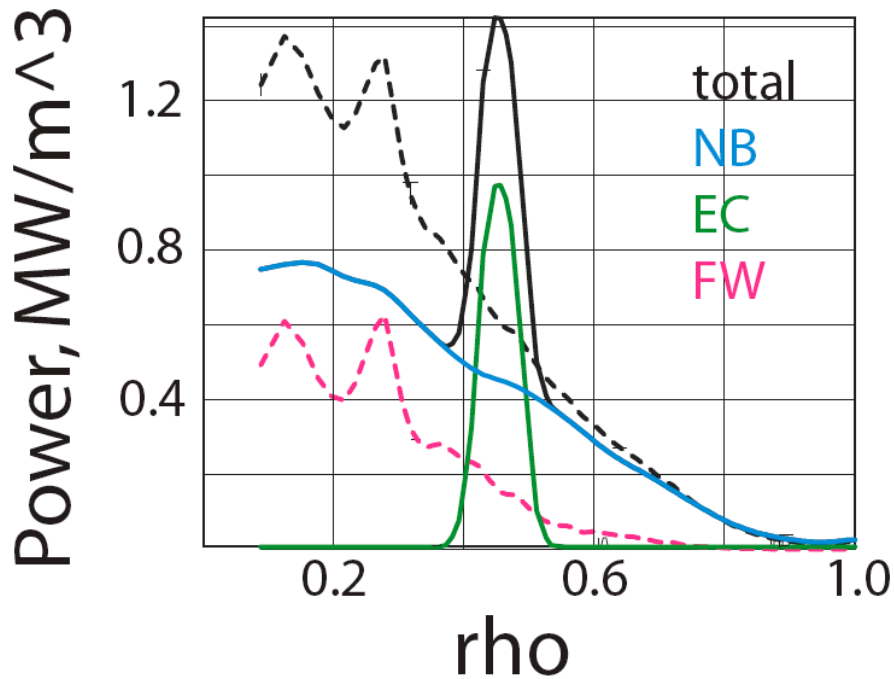
Add **2.2 MW** of FW heating to EC H/CD (not shown)

NB +EC (ref) & NB + FW

## Profiles at $t= 0.9$ s

ECCD creates localized  $q_{\min}$  at slightly larger  $\rho$  than FW

FW heating enhances NBCD and BS in core

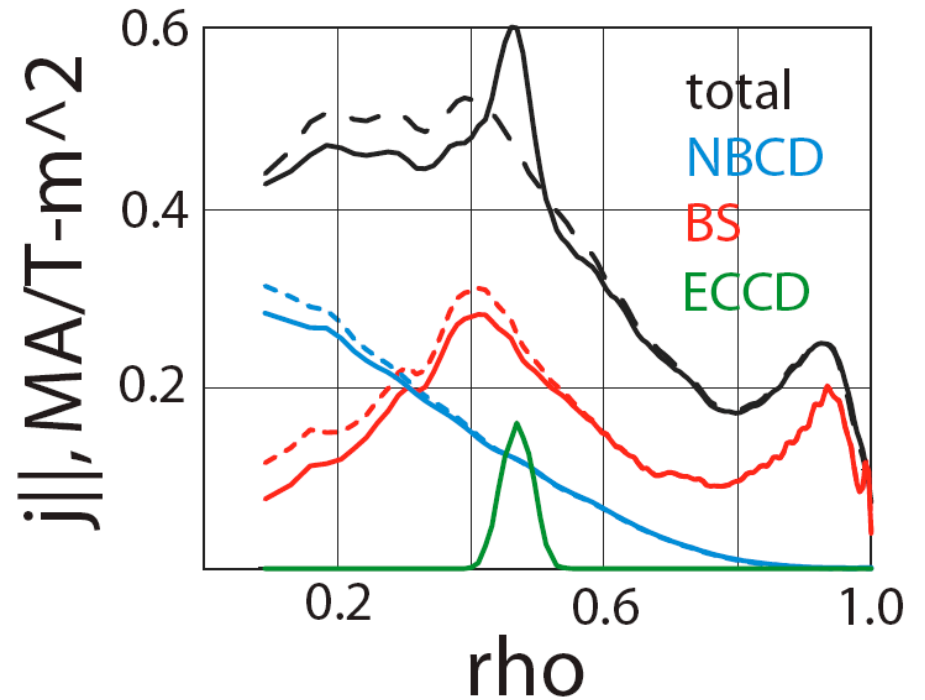
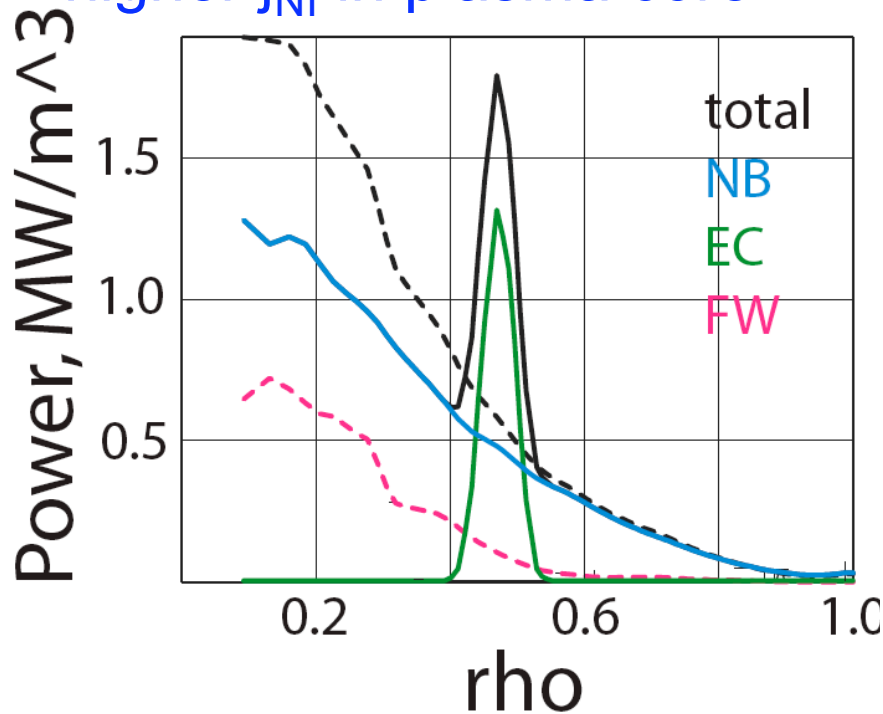
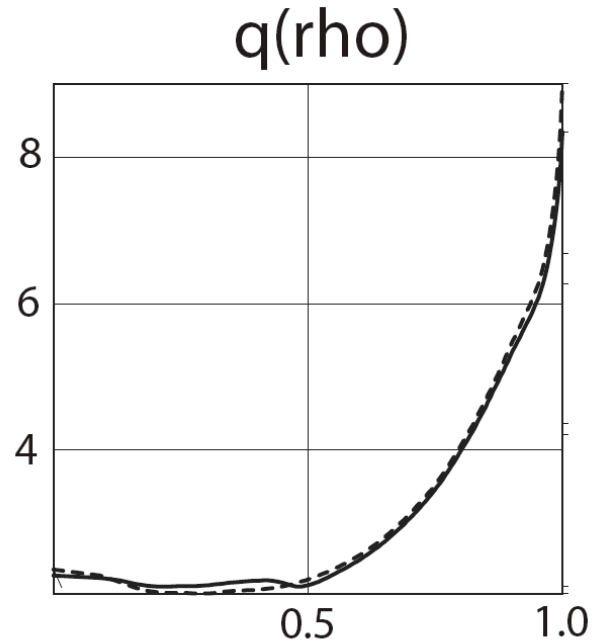


NB + EC (ref) & NB + FW

**Profiles at t = 1.5s**

ECCD producing slightly higher  $q_{\min}$  and larger  $\rho(q_{\min})$

FW heating still providing slightly higher  $j_{NI}$  in plasma core

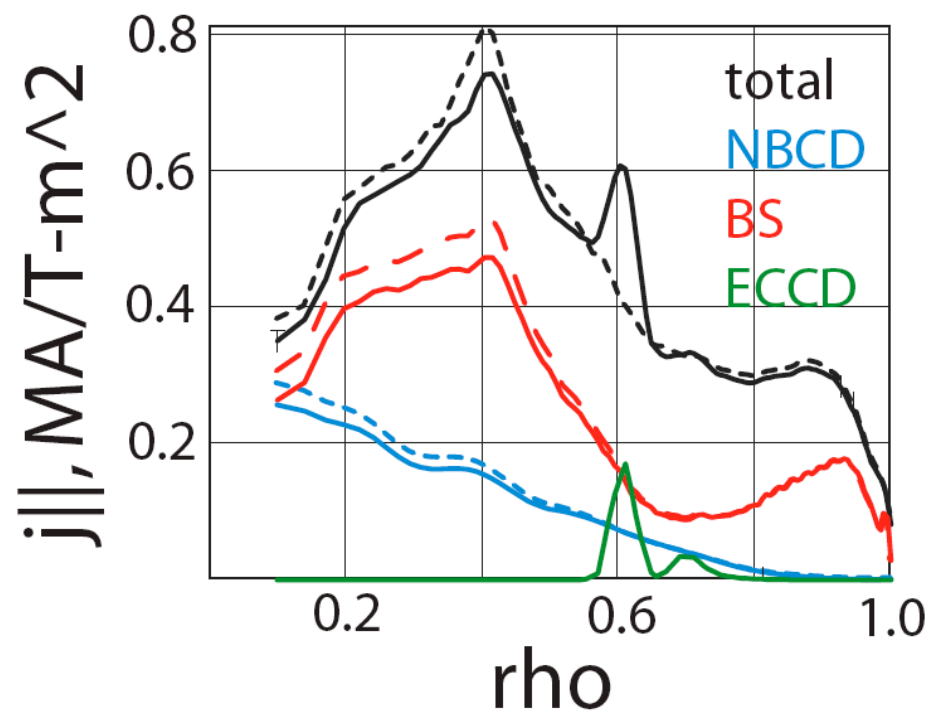
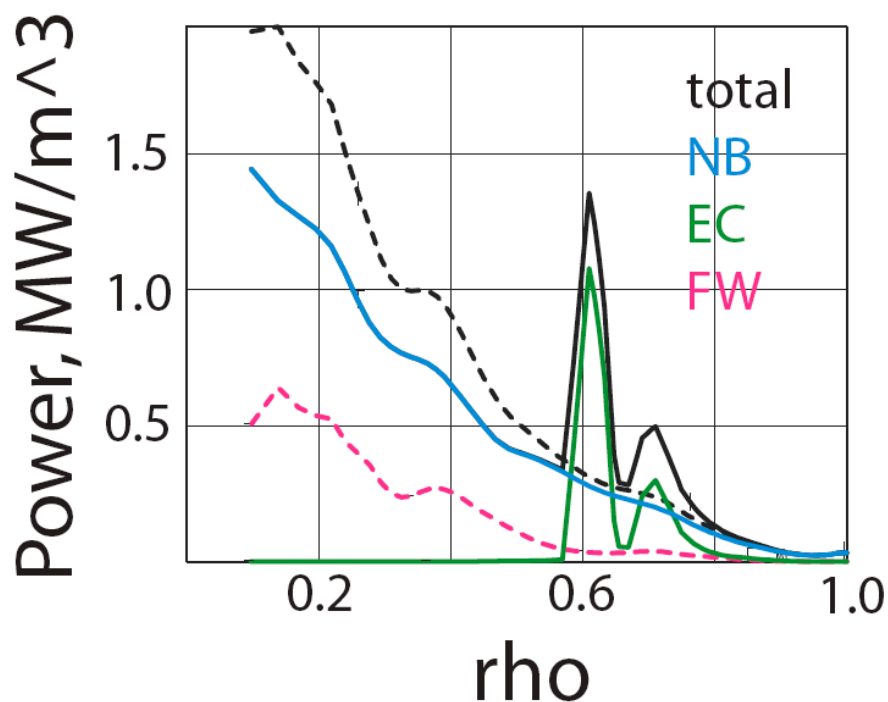
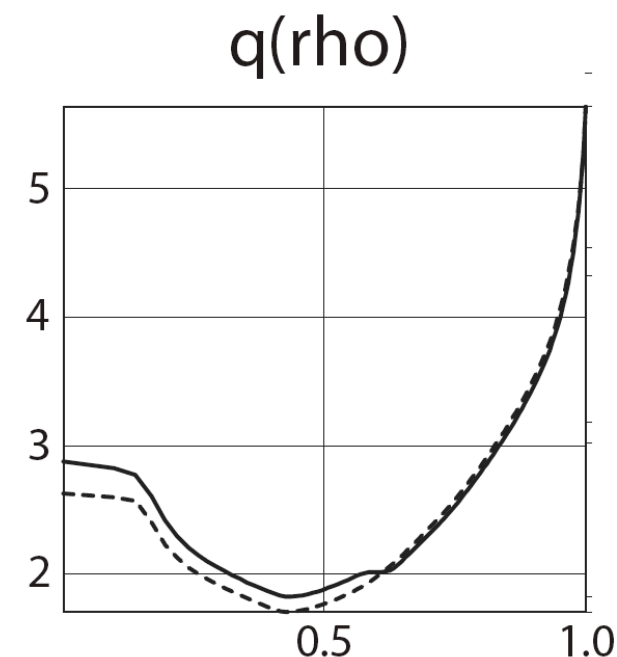


NB + EC (ref) & NB + FW

## Profiles at $t = 2.5$ s

ECCD case has higher  $q_{\min}$ ,  
although CD is diminishing as  
deposition moves to larger  $\rho$

FW & EC cases have same  $\rho(q_{\min})$

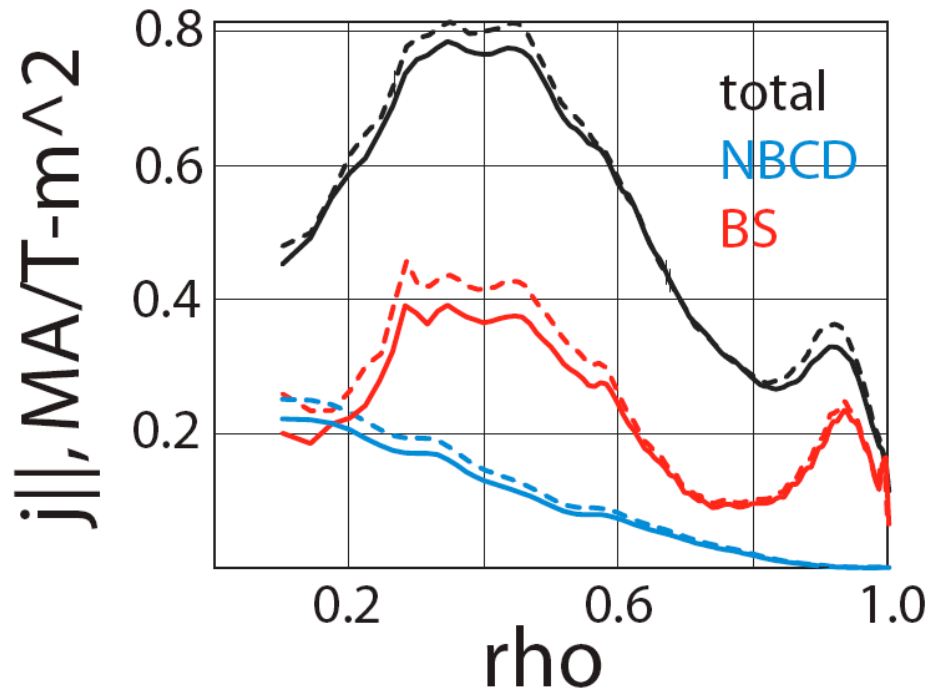
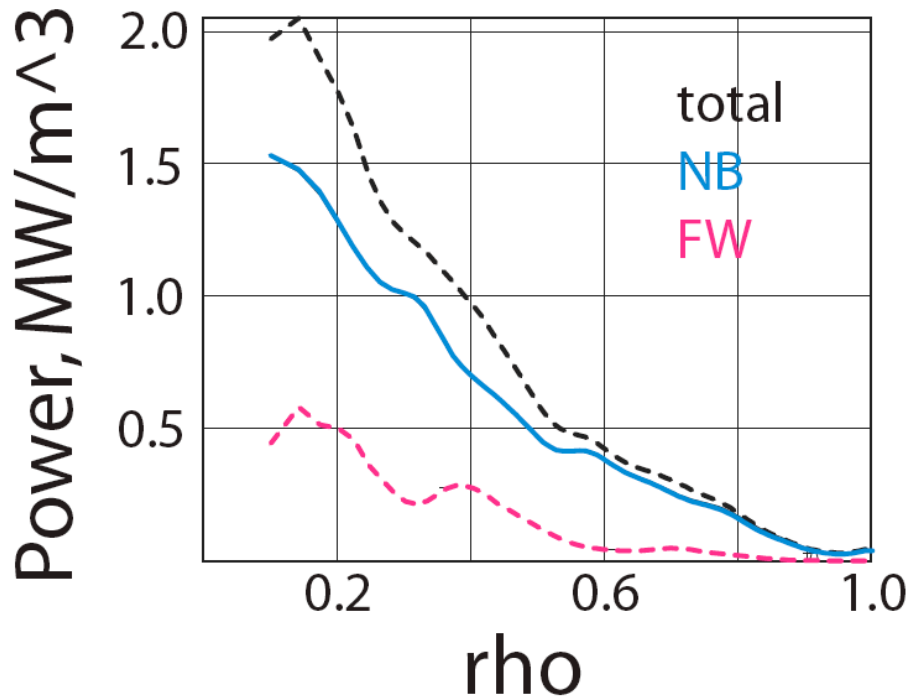
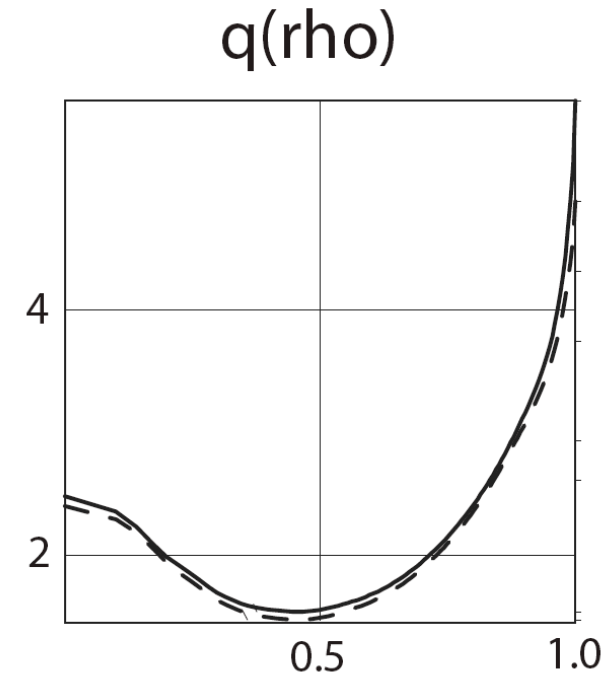


NB + EC (ref) & NB + FW

**Profiles at t = 3.4 s**

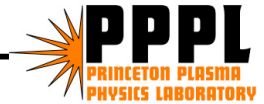
EC has been cutoff due to high density

FW heating continues, but makes little difference to q profile



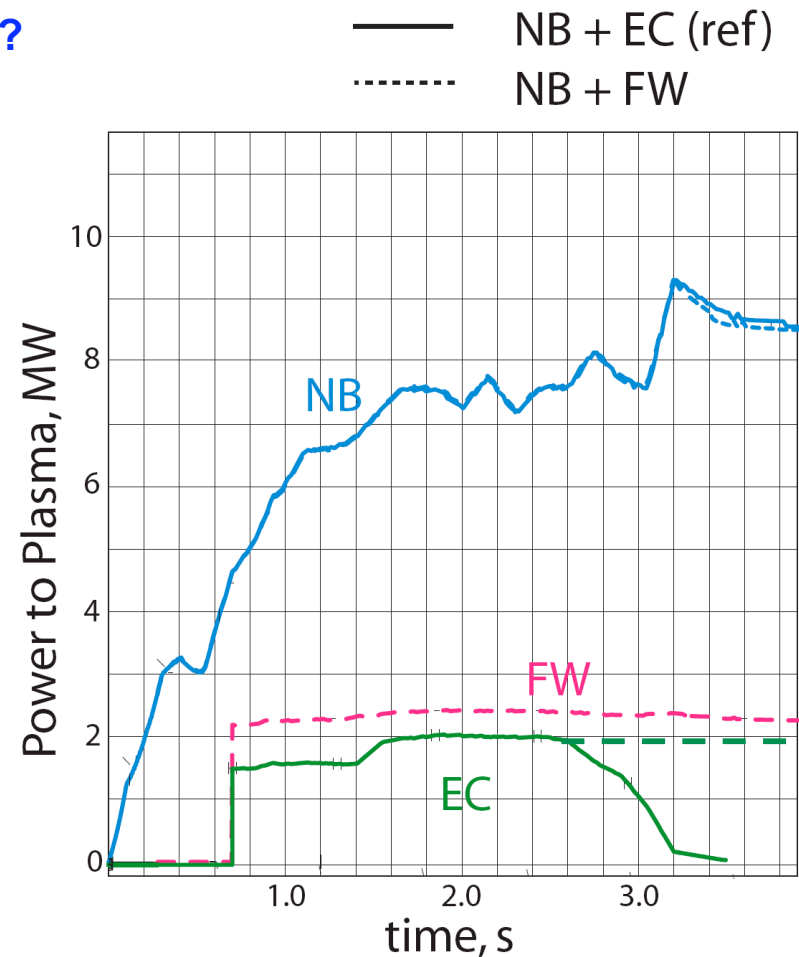
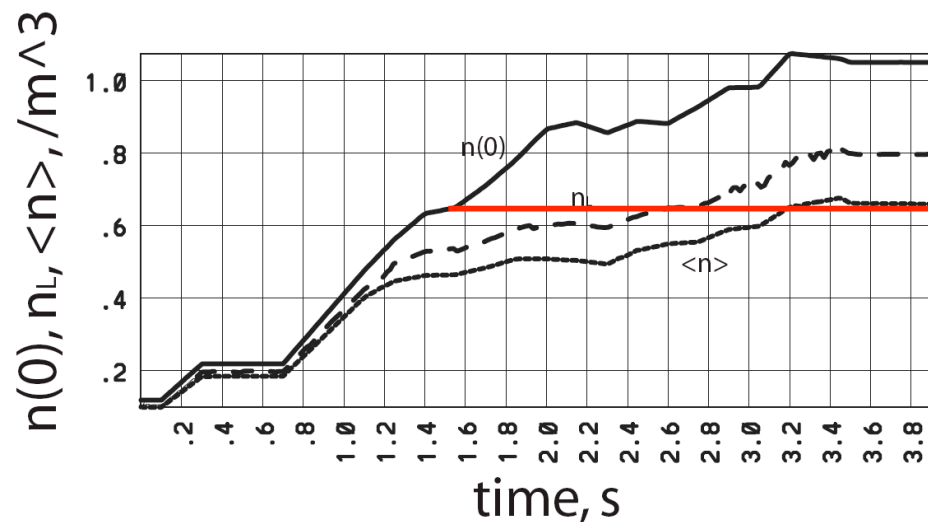
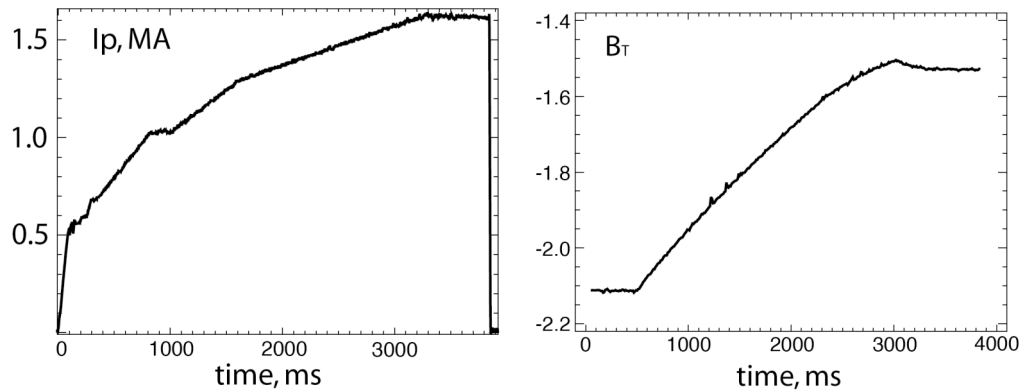
# Simulate High Beta Discharge 122976

## Adding n control and ICRF/FW



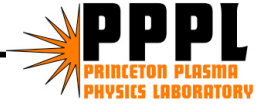
**HOWEVER, n is uncontrolled & EC is eventually cutoff**

**How can FWCD alter the discharge & how can stopping the density rise improve the discharge?**



— NB + EC (ref)  
- - - NB + FW

# Simulations with TSC and TRANSP Examine Effects of ICRF in D3D High Beta Plasmas



Take existing discharge and modify it with FWCD and density control

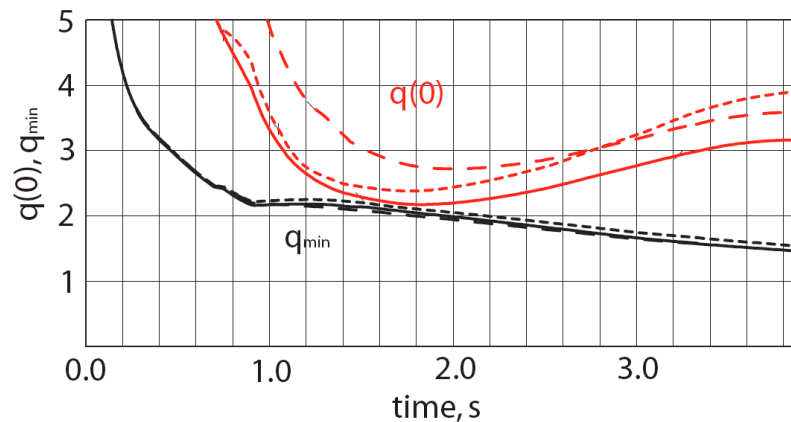
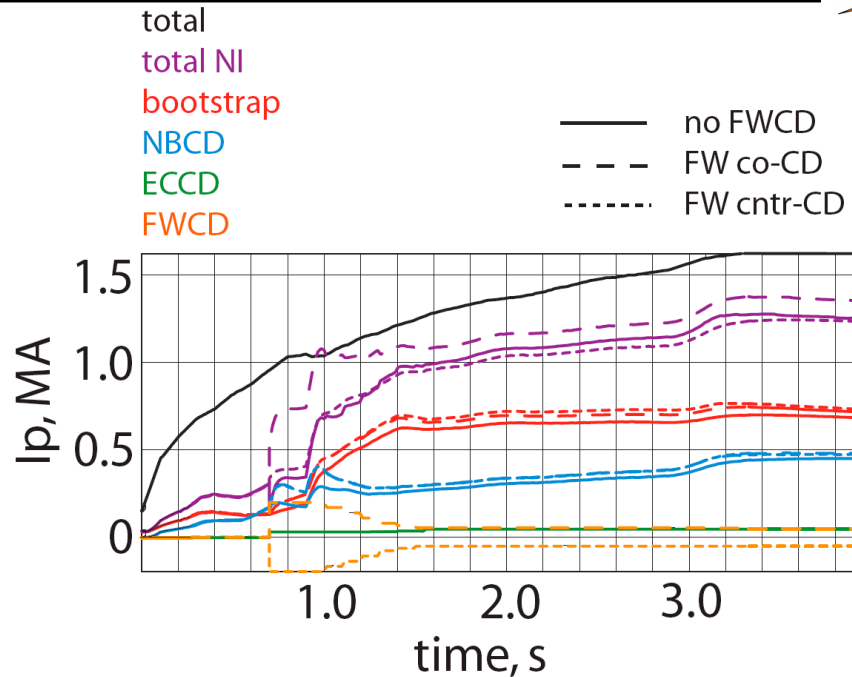
Stop density rise at  $6.5 \times 10^{20} / \text{m}^3$

Inject FWCD at same time as EC

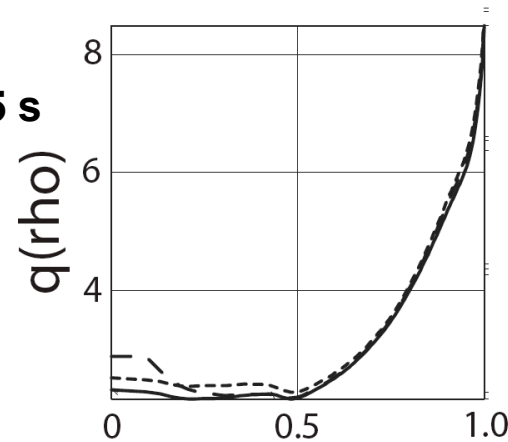
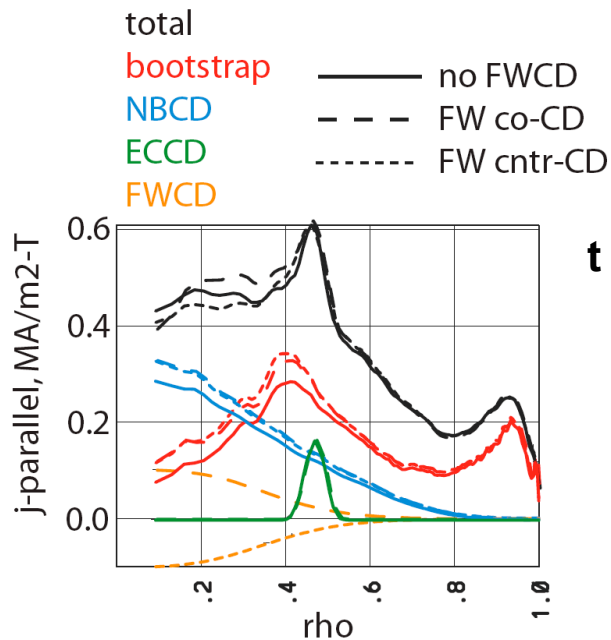
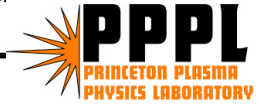
Examine co-CD and cntr-CD

Some improvement in NICD

Changes in  $q(0)$  evolution

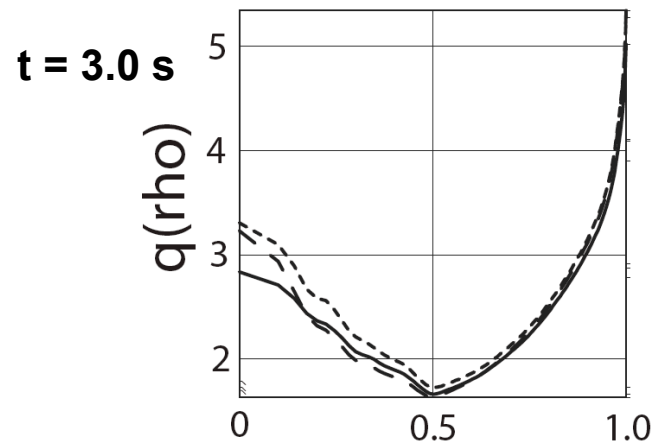
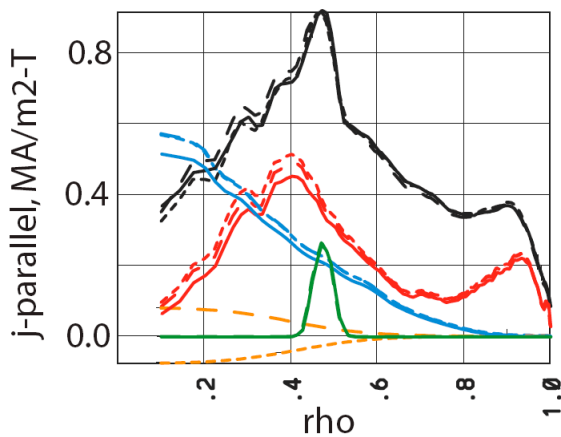


# D3D 122976 is a Complex Discharge with $I_p$ & $B_T$ Ramps, But it Can Be Modeled



Complex and transient discharges are difficult to understand experimentally

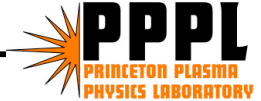
Modeling can provide insights into internal plasma behavior that is difficult to measure



Subtle effects can be important in advanced tokamak scenarios

Learning how to control plasma profiles will require significant time-dependent modeling

# How to Reach $\beta_N \approx 5$



- Primary plasma characteristics that affect ideal MHD stability
  - $p(0)/\langle p \rangle$
  - $\rho(q_{\min})$
  - $q_{\min}$  (dictated by  $\rho(q_{\min})$  and  $I_p$ )
  - Shaping (limited by structures and PF coil/shape control)
  - Wall location (moving plasma closer to wall)
  
- Examine with equilibrium and stability calculations
  - $p(0)/\langle p \rangle \approx 2.15$  and  $2.8$  ( $n(0)/\langle n \rangle = 1.15$  and  $1.35$ )
  - $\rho(q_{\min}) = 0.45$  and  $0.65$
  - $I_p = 1.2, 1.5, 1.7$  MA
  - $B_T = 1.85$
  - Standard shape ( $\kappa=1.85, \delta=0.65, \xi_0=0.1$ , wall varied  $b/a = 1.4-1.5$ )

Transiently = the  $j(\rho)$  and  $p(\rho)$  correlation is broken

# How to Reach $\beta_N \approx 5$



- Prescribe parallel current density analytically to give  $I_p$  and  $\rho(q_{min})$ , model inductive current evolution
- Pressure profile is analytic function with pedestal term
- **Examine  $\beta_N = 5$  first**

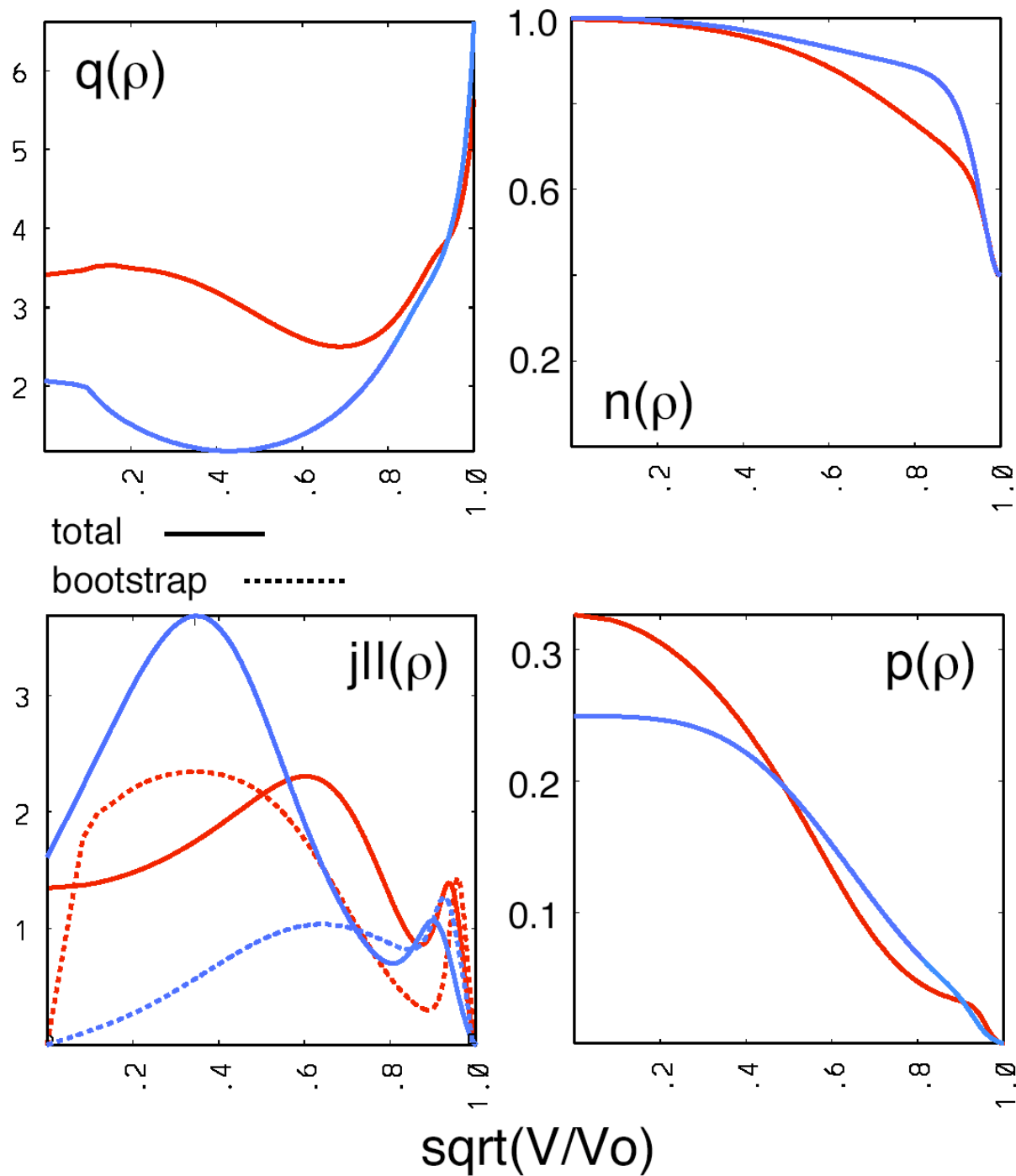
- Best case has  $\rho(q_{min}) = 0.65$  and peaked pressure ( $p(0)/\langle p \rangle = 2.85$ ), wall at  $b/a = 0.5$  stable to  $n = 1, 2, 3$ 
  - $I_p = 1.5$  and  $1.7$  MA are good
  - $I_p = 1.25$  not as good
  - Lowering  $B_T$  appears necessary to get to higher  $\beta_N$ , so re-examine at  $1.65$  and  $1.5$  T ---> same good stability behavior, a little better at  $1.2$  MA



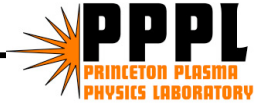
**RED:**  
Peaked pressure with  
large rho(qmin)

**BEST stability at  
high  $\beta_N$**

**BLUE:**  
Broad pressure with  
small rho(qmin)



# High Bootstrap Fraction is an Important Area Integrating Transport, Stability, and Control



*Politzer's experiments are tremendously important* ---> they push in the “other” direction for AT (other than high  $\beta_N$ )

They demonstrate that the transport regime and bootstrap current are strongly coupled at high  $f_{BS}$

- Confinement was good

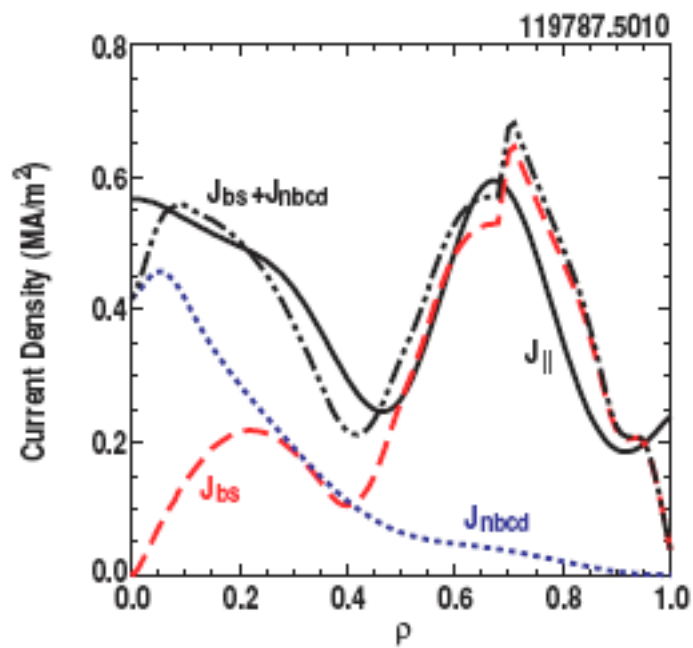
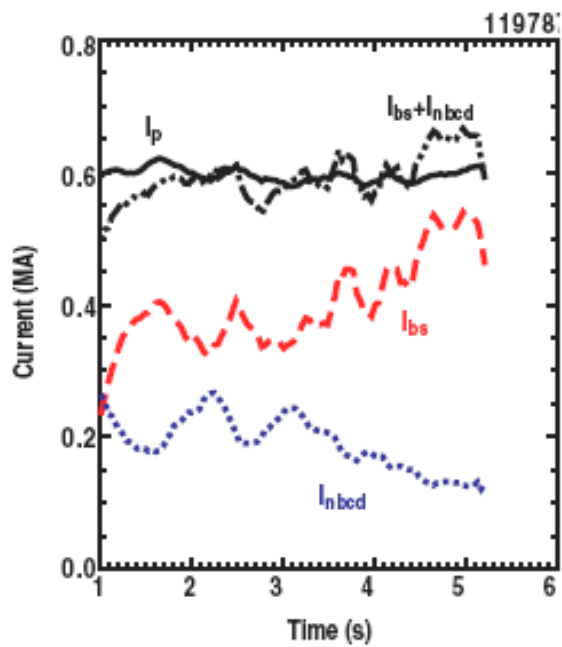
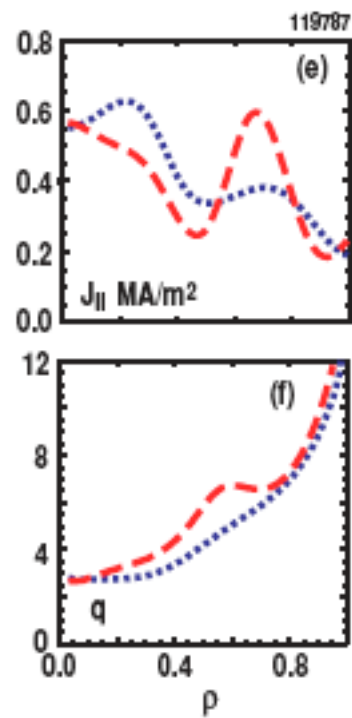
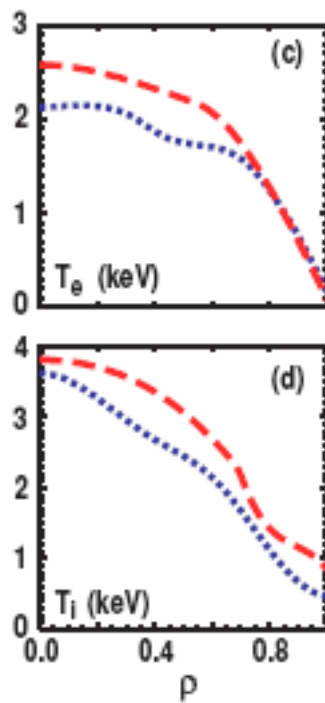
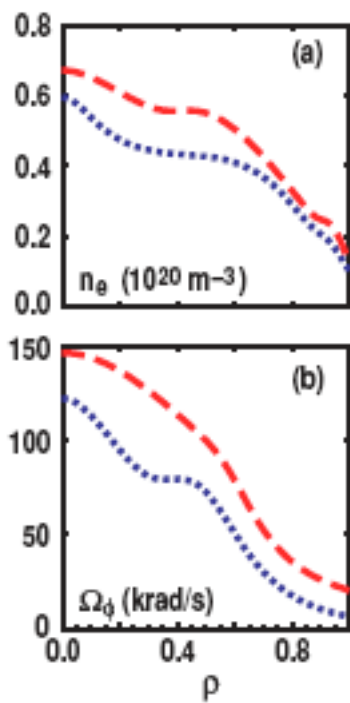
- MHD did not appear to degrade confinement

- ITB is present at large radius, producing large  $j_{BS}$

The oscillations present in these plasmas may be sensitive to  $q/j$ -profile established, use of  $I_p$  control/presence of inductive current,  $f_{BS}$ , etc. ---> are oscillations bad or good?

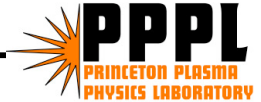
---> I have been interested in oscillating the transport regime between 2 states (ITB and H-mode) to control  $j_{BS}$

The report of this work clearly shows this area has not been examined in great detail ----> *it should be!*



# Very High Bootstrap Current Plasmas

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- Impact of different densities
- Produce different  $q/j$  profiles with available sources, in particular, change shear inside ITB
- Can ICRF/FW be used?
- How is EC being used?
- Ideal MHD analysis, where are the limits
- Plasma configuration as  $I_{NI}/I_p$  goes from 0.75 to 1.0
  - How is transport, MHD,  $q/j$  changing
- ....